

IAEA Safety Standards

for protecting people and the environment

Radiation Protection and Safety of Radiation Sources: International Basic Safety Standards

Jointly sponsored by

EC, FAO, IAEA, ILO, OECD/NEA, PAHO, UNEP, WHO



General Safety Requirements Part 3 No. GSR Part 3



IAEA

International Atomic Energy Agency

IAEA SAFETY STANDARDS AND RELATED PUBLICATIONS

IAEA SAFETY STANDARDS

Under the terms of Article III of its Statute, the IAEA is authorized to establish or adopt standards of safety for protection of health and minimization of danger to life and property, and to provide for the application of these standards.

The publications by means of which the IAEA establishes standards are issued in the **IAEA Safety Standards Series**. This series covers nuclear safety, radiation safety, transport safety and waste safety. The publication categories in the series are **Safety Fundamentals**, **Safety Requirements** and **Safety Guides**.

Information on the IAEA's safety standards programme is available on the IAEA Internet site

<http://www-ns.iaea.org/standards/>

The site provides the texts in English of published and draft safety standards. The texts of safety standards issued in Arabic, Chinese, French, Russian and Spanish, the IAEA Safety Glossary and a status report for safety standards under development are also available. For further information, please contact the IAEA at: Vienna International Centre, PO Box 100, 1400 Vienna, Austria.

All users of IAEA safety standards are invited to inform the IAEA of experience in their use (e.g. as a basis for national regulations, for safety reviews and for training courses) for the purpose of ensuring that they continue to meet users' needs. Information may be provided via the IAEA Internet site or by post, as above, or by email to Official.Mail@iaea.org.

RELATED PUBLICATIONS

The IAEA provides for the application of the standards and, under the terms of Articles III and VIII.C of its Statute, makes available and fosters the exchange of information relating to peaceful nuclear activities and serves as an intermediary among its Member States for this purpose.

Reports on safety in nuclear activities are issued as **Safety Reports**, which provide practical examples and detailed methods that can be used in support of the safety standards.

Other safety related IAEA publications are issued as **Emergency Preparedness and Response** publications, **Radiological Assessment Reports**, the International Nuclear Safety Group's **INSAG Reports**, **Technical Reports** and **TECDOCs**. The IAEA also issues reports on radiological accidents, training manuals and practical manuals, and other special safety related publications.

Security related publications are issued in the **IAEA Nuclear Security Series**.

The **IAEA Nuclear Energy Series** comprises informational publications to encourage and assist research on, and the development and practical application of, nuclear energy for peaceful purposes. It includes reports and guides on the status of and advances in technology, and on experience, good practices and practical examples in the areas of nuclear power, the nuclear fuel cycle, radioactive waste management and decommissioning.

RADIATION PROTECTION AND
SAFETY OF RADIATION SOURCES:
INTERNATIONAL BASIC
SAFETY STANDARDS

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The Agency's Statute was approved on 23 October 1956 by the Conference on the Statute of the IAEA held at United Nations Headquarters, New York; it entered into force on 29 July 1957. The Headquarters of the Agency are situated in Vienna. Its principal objective is "to accelerate and enlarge the contribution of atomic energy to peace, health and prosperity throughout the world".

IAEA SAFETY STANDARDS SERIES No. GSR Part 3

RADIATION PROTECTION AND SAFETY OF RADIATION SOURCES: INTERNATIONAL BASIC SAFETY STANDARDS

GENERAL SAFETY REQUIREMENTS

JOINTLY SPONSORED BY:

EUROPEAN COMMISSION,
FOOD AND AGRICULTURE ORGANIZATION OF
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INTERNATIONAL LABOUR ORGANIZATION,
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WORLD HEALTH ORGANIZATION

This publication includes a CD-ROM containing the IAEA Safety Glossary:
2007 Edition (2007) and the Fundamental Safety Principles (2006),
each in Arabic, Chinese, English, French, Russian and Spanish versions.

The CD-ROM is also available for purchase separately.

See: <http://www-pub.iaea.org/MTCD/publications/publications.asp>

INTERNATIONAL ATOMIC ENERGY AGENCY
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Marketing and Sales Unit, Publishing Section
International Atomic Energy Agency
Vienna International Centre
PO Box 100
1400 Vienna, Austria
fax: +43 1 2600 29302
tel.: +43 1 2600 22417
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FOREWORD

by Yukiya Amano
Director General

The IAEA's Statute authorizes the Agency to "establish or adopt... standards of safety for protection of health and minimization of danger to life and property" — standards that the IAEA must use in its own operations, and which States can apply by means of their regulatory provisions for nuclear and radiation safety. The IAEA does this in consultation with the competent organs of the United Nations and with the specialized agencies concerned. A comprehensive set of high quality standards under regular review is a key element of a stable and sustainable global safety regime, as is the IAEA's assistance in their application.

The IAEA commenced its safety standards programme in 1958. The emphasis placed on quality, fitness for purpose and continuous improvement has led to the widespread use of the IAEA standards throughout the world. The Safety Standards Series now includes unified Fundamental Safety Principles, which represent an international consensus on what must constitute a high level of protection and safety. With the strong support of the Commission on Safety Standards, the IAEA is working to promote the global acceptance and use of its standards.

Standards are only effective if they are properly applied in practice. The IAEA's safety services encompass design, siting and engineering safety, operational safety, radiation safety, safe transport of radioactive material and safe management of radioactive waste, as well as governmental organization, regulatory matters and safety culture in organizations. These safety services assist Member States in the application of the standards and enable valuable experience and insights to be shared.

Regulating safety is a national responsibility, and many States have decided to adopt the IAEA's standards for use in their national regulations. For parties to the various international safety conventions, IAEA standards provide a consistent, reliable means of ensuring the effective fulfilment of obligations under the conventions. The standards are also applied by regulatory bodies and operators around the world to enhance safety in nuclear power generation and in nuclear applications in medicine, industry, agriculture and research.

Safety is not an end in itself but a prerequisite for the purpose of the protection of people in all States and of the environment — now and in the future. The risks associated with ionizing radiation must be assessed and controlled without unduly limiting the contribution of nuclear energy to equitable and sustainable development. Governments, regulatory bodies and operators everywhere must ensure that nuclear material and radiation sources are used beneficially, safely and ethically. The IAEA safety standards are designed to facilitate this, and I encourage all Member States to make use of them.

NOTE BY THE SECRETARIAT

The IAEA safety standards reflect an international consensus on what constitutes a high level of safety for protecting people and the environment from harmful effects of ionizing radiation. The process of developing, reviewing and establishing the IAEA standards involves the IAEA Secretariat and all Member States, many of which are represented on the four IAEA safety standards committees and the IAEA Commission on Safety Standards.

The IAEA standards, as a key element of the global safety regime, are kept under regular review by the Secretariat, the safety standards committees and the Commission on Safety Standards. The Secretariat gathers information on experience in the application of the IAEA standards and information gained from the follow-up of events for the purpose of ensuring that the standards continue to meet users' needs. The present publication reflects feedback and experience accumulated until 2010 and it has been subject to the rigorous review process for standards.

Lessons that may be learned from studying the accident at the Fukushima Daiichi nuclear power plant in Japan following the disastrous earthquake and tsunami of 11 March 2011 will be reflected in this IAEA safety standard as revised and issued in the future.

PREFACE BY THE SPONSORING ORGANIZATIONS

ESTABLISHMENT OF THE INTERNATIONAL BASIC SAFETY STANDARDS BY THE IAEA BOARD OF GOVERNORS AND THEIR APPROVAL BY THE SPONSORING ORGANIZATIONS

Radiation Protection and Safety of Radiation Sources: International Basic Safety Standards are jointly sponsored by the European Commission (EC), the Food and Agriculture Organization of the United Nations (FAO), the IAEA, the International Labour Organization (ILO), the OECD Nuclear Energy Agency (OECD/NEA), the Pan American Health Organization (PAHO), the United Nations Environment Programme (UNEP) and the World Health Organization (WHO) (the Sponsoring Organizations).

The Board of Governors of the IAEA established the draft text in English of the revised International Basic Safety Standards (hereinafter referred to as ‘these Standards’) as a standard of the IAEA at its meeting on 12 September 2011.

The Steering Committee for Nuclear Energy, the governing body of the OECD/NEA, approved these Standards and agreed to jointly sponsor them at its meeting held on 27–28 October 2011.

The Director General of the FAO confirmed its endorsement and co-sponsorship of these Standards on 29 October 2011.

The Executive Director of UNEP confirmed its support for and co-sponsorship of these Standards by letter addressed to the Director General of the IAEA dated 12 March 2012.

The ILO’s Governing Body approved publication of these Standards at its 313th Session on 21 March 2012.

The WHO’s Executive Board noted these Standards on 28 May 2012 at its 131st meeting, and this completed the necessary action by WHO as a co-sponsor.

The EC notified endorsement of these Basic Safety Standards on behalf of the European Atomic Energy Community (Euratom) by letter addressed to the Director General of the IAEA dated 14 August 2012.

For PAHO, the 28th Pan American Sanitary Conference endorsed these Standards on 20 September 2012, and requested PAHO to cooperate with its member states on their implementation. This completed the authorization process for joint sponsorship by the Sponsoring Organizations.

The IAEA, on behalf of the Sponsoring Organizations, is hereby issuing these Standards as General Safety Requirements No. GSR Part 3 in the IAEA Safety Standards Series. These Standards are to be issued as a final publication in Arabic, Chinese, English, French, Russian and Spanish. General Safety Requirements No. GSR Part 3 supersedes General Safety Requirements No. GSR Part 3 (Interim edition) issued in 2011, which itself superseded the

International Basic Safety Standards that were issued as IAEA Safety Series No. 115 in February 1996 (hereinafter referred to as the ‘BSS of 1996’) under the joint sponsorship of the FAO, IAEA, ILO, OECD/NEA, PAHO and the WHO.

BASIS FOR THESE STANDARDS

In the system of international intergovernmental organizations, the first endeavour by the IAEA to establish international standards for radiation protection and safety was made in the 1950s. The Board of Governors of the IAEA first approved health and safety measures in March 1960¹, when it was stated that “The Agency’s basic safety standards...will be based, to the extent possible, on the recommendations of the International Commission on Radiological Protection (ICRP)”.

The Board of Governors of the IAEA first approved Basic Safety Standards in June 1962; they were published by the IAEA as IAEA Safety Series No. 9². A revised edition was issued in 1967³. A third revision was published by the IAEA as the 1982 Edition of IAEA Safety Series No. 9⁴; this edition was jointly sponsored by the IAEA, ILO, OECD/NEA and the WHO. The next edition was International Basic Safety Standards for Protection against Ionizing Radiation and for the Safety of Radiation Sources, published by the IAEA as IAEA Safety Series No. 115 in February 1996⁵, and jointly sponsored by the FAO, IAEA, ILO, OECD/NEA, PAHO and the WHO.

¹ INTERNATIONAL ATOMIC ENERGY AGENCY, The Agency’s Health and Safety Measures, INFCIRC/18, IAEA, Vienna (1960); The Agency’s Safety Standards and Measures, INFCIRC/18/Rev.1, IAEA, Vienna (1976).

² INTERNATIONAL ATOMIC ENERGY AGENCY, Basic Safety Standards for Radiation Protection, IAEA Safety Series No. 9, IAEA, Vienna (1962).

³ INTERNATIONAL ATOMIC ENERGY AGENCY, Basic Safety Standards for Radiation Protection (1967 Edition), IAEA Safety Series No. 9, IAEA, Vienna (1967).

⁴ INTERNATIONAL ATOMIC ENERGY AGENCY, Basic Safety Standards for Radiation Protection (1982 Edition), IAEA Safety Series No. 9, IAEA, Vienna (1982).

⁵ FOOD AND AGRICULTURE ORGANIZATION OF THE UNITED NATIONS, INTERNATIONAL ATOMIC ENERGY AGENCY, INTERNATIONAL LABOUR ORGANISATION, OECD NUCLEAR ENERGY AGENCY, PAN AMERICAN HEALTH ORGANIZATION, WORLD HEALTH ORGANIZATION, International Basic Safety Standards for Protection against Ionizing Radiation and for the Safety of Radiation Sources, IAEA Safety Series No. 115, IAEA, Vienna (1996).

These Standards apply the IAEA Safety Fundamentals: Fundamental Safety Principles (SF-1)⁶, which includes a statement of the fundamental safety objective and a unified set of principles representing a common safety philosophy across all areas of application of the IAEA safety standards. The fundamental safety objective of protecting people — individually and collectively — and the environment from harmful effects of ionizing radiation has to be achieved without unduly limiting the operation of facilities or the conduct of activities that give rise to radiation risks.

SF-1 constitutes the basis on which, under the IAEA's safety standards programme, these Standards establish requirements for the protection of people and the environment from harmful effects of ionizing radiation and for the safety of radiation sources. It also provides the rationale for the IAEA's wider safety related programme. SF-1 was jointly sponsored by Euratom, the FAO, ILO, International Maritime Organization (IMO), OECD/NEA, PAHO, UNEP and the WHO.

These Standards are published in the IAEA Safety Standards Series, which includes other related international standards, such as the Regulations for the Safe Transport of Radioactive Material (the IAEA Transport Regulations (No. SSR-6)); Governmental, Legal and Regulatory Framework for Safety (General Safety Requirements No. GSR Part 1); Preparedness and Response for a Nuclear or Radiological Emergency (No. GS-R-2); The Management System for Facilities and Activities (No. GS-R-3); Safety Assessment for Facilities and Activities (General Safety Requirements No. GSR Part 4); Predisposal Management of Radioactive Waste (General Safety Requirements No. GSR Part 5); and Decommissioning of Facilities (General Safety Requirements No. GSR Part 6).

The other Sponsoring Organizations have also produced standards, codes and guides in their respective spheres of activity. Notably, the ILO has established a Convention concerning the Protection of Workers against Ionising Radiations, 1960 (No. 115) and a Recommendation concerning the Protection of Workers against Ionising Radiations, 1960 (No. 114). The ILO has also issued a code of practice for the radiation protection of workers as well as other relevant publications. The PAHO and the WHO have issued a number of publications relating to the protection and safety of workers and of patients

⁶ EUROPEAN ATOMIC ENERGY COMMUNITY, FOOD AND AGRICULTURE ORGANIZATION OF THE UNITED NATIONS, INTERNATIONAL ATOMIC ENERGY AGENCY, INTERNATIONAL LABOUR ORGANIZATION, INTERNATIONAL MARITIME ORGANIZATION, OECD NUCLEAR ENERGY AGENCY, PAN AMERICAN HEALTH ORGANIZATION, UNITED NATIONS ENVIRONMENT PROGRAMME, WORLD HEALTH ORGANIZATION, Fundamental Safety Principles, IAEA Safety Standards Series No. SF-1, IAEA, Vienna (2006).

in medical applications of radiation. The FAO and the WHO have established, through the Joint FAO/WHO Codex Alimentarius Commission, guideline levels for radioactive substances in foodstuffs moving in international trade. The WHO produces guidelines for drinking water quality that include criteria to assess the safety of drinking water with respect to its radionuclide content.

The OECD/NEA has issued publications on specific topics relating to radiation protection and safety. Euratom, since 20 February 1959, has adopted Basic Safety Standards for protection of the health of the general public and workers against the dangers of ionizing radiation laid down in Council Directives (Euratom), and regularly updated these in close cooperation with the development of international standards. The Euratom Standards are binding on EU member states, and the unique responsibility of Euratom in establishing such standards implies that its sponsorship of international standards encourages their harmonized implementation worldwide without prejudice to Euratom procedures and legislation.

These Standards are to be applied by the Sponsoring Organizations to their own operations, and are recommended for use by States and national authorities and by other international organizations in relation to their own activities. The application of these standards by EU member states is ensured by the implementation of binding Euratom legislation.

International conventions and the IAEA safety standards, appropriately supplemented by industry standards and detailed national requirements, establish a consistent and comprehensive basis for the proper protection of people and the environment from harmful effects of ionizing radiation.

REVISION PROCESS FOR THE INTERNATIONAL BASIC SAFETY STANDARDS

These Standards reflect continuing efforts over several decades towards the harmonization of safety standards internationally. The international effort to review and revise the BSS of 1996 involved hundreds of experts from the member states of the Sponsoring Organizations and from specialized organizations.

At its meeting in November 2004, the IAEA's Commission on Safety Standards requested the IAEA Secretariat to prepare an outline for a possible review and revision of the BSS of 1996 for presentation at its meeting in June 2005.

In September 2005, the General Conference of the IAEA, by way of resolution GC(49)/RES/9A, requested the IAEA Secretariat to undertake a review

of the BSS of 1996. The Inter-Agency Committee for Radiation Safety (IACRS)⁷ agreed, at its meeting in October 2005, to the establishment of a joint secretariat (hereinafter referred to as the 'BSS Secretariat'). In December 2005, the IAEA formally invited representatives of the United Nations and other intergovernmental organizations to jointly coordinate the review and revision of the BSS of 1996 through the establishment of the BSS Secretariat. The BSS Secretariat was coordinated by the IAEA and comprised representatives of the EC, FAO, IAEA, ICRP, ILO, OECD/NEA, PAHO, UNEP and the WHO.

In September 2006, the Director General of the IAEA reported to the IAEA General Conference that the review of the BSS of 1996 had concluded that, while there was no major issue necessitating urgent revision, there was a case to be made for a revision in order to take account of numerous improvements that had been suggested. The IAEA General Conference, by way of resolution GC(50)/RES/10, noted the review of the BSS of 1996 that had been carried out in response to para. 10 of resolution GC(49)/RES/9, and noted that a revision was to be coordinated by the BSS Secretariat. Resolution GC(50)/RES/10 also urged that the BSS Secretariat carefully consider and justify potential changes, taking into account their implications for national regulations.

In the fourth quarter of 2006, an outline for the revision of the BSS of 1996 was approved by the four IAEA Safety Standards Committees and was endorsed by the IAEA Commission on Safety Standards. The IAEA Safety Standards Committees advised the Secretariat that the comprehensive and complete character of the BSS of 1996 should be retained in the new edition, which should continue to underpin the regulatory infrastructure to control sources and practices in radiation safety in all areas, including medicine, general industry, the nuclear industry, radioactive waste management and the transport of radioactive material. They further advised that the new edition should cover the categories of occupational exposure, medical exposure and public exposure, and should cover both normal circumstances of exposure and emergency situations.

The revision of the BSS of 1996 commenced in early 2007 with a series of drafting meetings on thematic areas, hosted by the IAEA, ILO, WHO, OECD/NEA and the PAHO. The draft developed at these meetings provided the basis for discussions at a technical meeting held in July 2007, with the

⁷ The IACRS provides a forum for the exchange of information between the agencies/organizations on their respective activities related to radiation safety. The IACRS is composed of members from the EC, FAO, IAEA, ILO, OECD/NEA, PAHO, United Nations Scientific Committee on the Effects of Atomic Radiation (UNSCEAR) and the WHO. The following non-governmental organizations currently have observer status: International Commission on Radiation Units and Measurements (ICRU), ICRP, International Electrotechnical Commission, International Radiation Protection Association and International Organization for Standardization (ISO).

participation of representatives of the Sponsoring Organizations. The technical meeting concluded that the new edition of the BSS should follow, to the extent possible, the new recommendations of the ICRP, including in particular the designation of situations of exposure in accordance with ICRP Publication 103⁸, i.e. ‘planned exposure situations’, ‘emergency exposure situations’ and ‘existing exposure situations’.

Further, the technical meeting recommended that the structure of the new edition should be based on the designations of ‘planned exposure situations’, ‘emergency exposure situations’ and ‘existing exposure situations’, with corresponding main sections following a similar layout: occupational exposure, public exposure and (for ‘planned exposure situations’ only) medical exposure. The technical meeting further recommended that there would also need to be a main section dealing with generic requirements that are applicable in all exposure situations.

The technical meeting also recommended that the new edition should cover protection of the environment, to be consistent with the Fundamental Safety Principles.

The text of this new edition takes into account the findings of UNSCEAR and also recommendations of the ICRP. The technical meeting noted that since both UNSCEAR and the ICRP had noted that there had been no significant change in the radiobiology underlying radiation protection in terms of nominal risk coefficients of dose, the values for dose limits and related quantities in the new edition would not be changed.

The four IAEA Safety Standards Committees endorsed the decisions made at the technical meeting regarding the proposed structure of the new edition at their meetings in late 2007.

In September 2007, the IAEA General Conference, by way of resolution GC(51)/RES/11, noted the forthcoming publication of the 2007 Recommendations of the ICRP, and again urged the Secretariat to consider carefully and to justify any potential changes to the BSS of 1996, while ensuring consistency with ICRP recommendations and taking into account the implications for national regulations and the importance of maintaining stability in international standards.

Further drafting and review meetings with the Sponsoring Organizations were held from late 2007 to 2009. The IAEA Safety Standards Committees and groups of experts from some of the Sponsoring Organizations provided comments on draft texts of the new edition in 2008 and 2009. A further technical meeting with the participation of the Sponsoring Organizations was held in

⁸ INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, The 2007 Recommendations of the International Commission on Radiological Protection, Publication 103, Elsevier (2007).

December 2009 to discuss a statement on radon that was issued by the ICRP in November 2009, and to assess its implications for the new edition. The technical meeting made recommendations on the text for the new edition on exposure due to radon in dwellings and occupational exposure due to radon.

The draft text of the new edition was submitted for comment to IAEA Member States on 28 January 2010. Each of the Sponsoring Organizations also submitted the draft text to its headquarters or to its member states for comment by their respective processes and procedures. More than 1500 comments were received from 41 IAEA Member States and from Sponsoring Organizations by the closing date of 31 May 2010, and these comments were considered in preparing a revised draft text.

The Nuclear Safety Standards Committee and the Transport Safety Standards Committee approved the draft text of the new edition at their meetings in November 2010. The Radiation Safety Standards Committee and the Waste Safety Standards Committee approved the draft text at their meetings in December 2010. The Committees noted that the ICRP was to release a statement relating to the dose limit for the lens of the eye, which should be taken into account prior to endorsement by the Commission on Safety Standards of the draft text of the new edition.

The ICRP, in April 2011, issued a statement on tissue reactions in which it recommended for occupational exposure a dose limit for equivalent dose to the lens of the eye of 20 mSv in a year, averaged over defined periods of 5 years, with the dose in no single year exceeding 50 mSv. Previously, this dose limit had been 150 mSv per year. The Commission on Safety Standards did endorse the draft text of the new edition at its meeting in May 2011, but requested that Member States be consulted on the proposed new dose limit for the lens of the eye. Member States were invited to provide comment on the proposed new dose limit for the lens of the eye by 7 July 2011. Following a recommendation from the outgoing and incoming Chairs of the Radiation Safety Standards Committee, the Chair of the Commission on Safety Standards approved the new dose limit for the lens of the eye on 12 July 2011.

APPLICATION OF THE INTERNATIONAL BASIC SAFETY STANDARDS

In responding to the Secretariat's submission to the IAEA Board of Governors, GOV/2011/42, the Board, at its meeting on 12 September 2011, established the draft text in English of the new edition "as an Agency safety standard — in accordance with Article III.A.6 of the Statute"⁹ and "authorized the Director General to promulgate these Safety Requirements...and to issue

⁹ Statute of the International Atomic Energy Agency, IAEA, Vienna (1990).

them as a Safety Requirements publication in the Safety Standards Series”. The Board also encouraged Member States “to implement arrangements for meeting these Safety Requirements”. At its 55th regular session, the IAEA General Conference, in resolution GC(55)/RES/9, encouraged Member States “to use safety standards issued by the IAEA in their national regulatory programmes, and noted the need to consider the periodic alignment of national regulations and guidance to internationally established standards and guidance”.

These Standards embody the international benchmark for radiation safety requirements, with major implications for policy making and decision making. Their adoption and application will facilitate the application of international safety standards and will make for greater consistency between the arrangements for protection and safety of different States. It is, therefore, desirable that all Member States adopt and apply these requirements. These safety requirements will be binding on the IAEA in relation to its own operations and they will be binding on States in relation to operations assisted by the IAEA.

These Standards are also to be applied by the Sponsoring Organizations to their own operations. They are recommended for use by States and national authorities and by other international organizations in relation to their own activities. The application of these Standards by EU member states is ensured by the implementation of binding Euratom legislation. Other States that are member states of the other Sponsoring Organizations may adopt the requirements, at their own discretion, or in accordance with their membership obligations, for application to their own activities.

It is recognized that it will take some time to make changes to existing systems of radiation protection in order to fulfil the requirements fully. The IAEA Secretariat envisaged that, for the IAEA’s own operations and for those operations assisted by the IAEA, arrangements needed to be in place to fulfil the requirements within a period of one year from the date of their adoption.

These Standards are intended for application and use by governmental authorities, including regulatory bodies responsible for authorizing facilities and activities; by organizations operating nuclear facilities, some mining and raw material processing facilities such as uranium mines, radioactive waste management facilities, and any other facilities producing or using radiation sources for industrial, research or medical purposes; by organizations transporting radioactive material; by organizations decommissioning facilities; and by staff and technical and scientific support organizations supporting such organizations and authorities.

Member States also subscribe to international conventions relating to nuclear and radiation related activities conducted within their jurisdiction. The Convention on Early Notification of a Nuclear Accident, the Convention on Assistance in the Case of a Nuclear Accident or Radiological Emergency, the

Convention on Nuclear Safety, the Joint Convention on the Safety of Spent Fuel Management and on the Safety of Radioactive Waste Management, and the ILO Convention concerning the Protection of Workers against Ionising Radiations, 1960 (No. 115) all place specific obligations on the Contracting Parties. These Basic Safety Standards and other IAEA safety standards constitute a useful tool for Contracting Parties to assess their performance under these international conventions. International conventions and IAEA safety standards, appropriately supplemented by industry standards and detailed national requirements, establish a consistent and comprehensive basis for the proper protection of people and the environment from harmful effects of ionizing radiation.

These Standards and other IAEA safety standards also support the application of the Code of Conduct on the Safety and Security of Radioactive Sources, the Code of Conduct on the Safety of Research Reactors, and the International Health Regulations, an international instrument that is legally binding on WHO member states.

LEGAL NATURE OF IAEA SAFETY STANDARDS

The statutory basis for the IAEA safety standards is to be found in Article III.A.6 of the Statute. By virtue of that provision the Agency is authorized:

“To establish or adopt, in consultation and, where appropriate, in collaboration with the competent organs of the United Nations and with the specialized agencies concerned, standards of safety for protection of health and minimization of danger to life and property (including such standards for labour conditions), and to provide for the application of these standards to its own operations as well as to the operations making use of materials, services, equipment, facilities, and information made available by the Agency or at its request or under its control or supervision; and to provide for the application of these standards, at the request of the parties, to operations under any bilateral or multilateral arrangement, or, at the request of a State, to any of that State’s activities in the field of atomic energy.”

Pursuant to that authorization, the first step in establishing IAEA safety standards was taken when the IAEA Board of Governors, on 31 March 1960, approved the Health and Safety Document: “The Agency’s Safety Standards and Measures” (INFCIRC/18). Thereafter, various safety standards (e.g. the BSS and the IAEA Transport Regulations) were established in accordance with Article III.A.6 and “The Agency’s Safety Standards and Measures”

(INFCIRC/18), which was revised in 1975 and approved by the IAEA Board in February 1976 (reproduced in INFCIRC/18/Rev.1).

Paragraph 1 of INFCIRC/18/Rev.1 contains the following relevant definitions:

“1.1 “Safety standards” means standards, regulations, rules or codes of practice established to protect man and the environment against ionizing radiation and to minimize danger to life and property.

“1.2 “The Agency’s safety standards” means safety standards established by the Agency under the authority of the Board of Governors. Such standards comprise:

- (a) The Agency’s basic safety standards for radiation protection, which prescribe maximum permissible doses and dose limits;
- (b) The Agency’s specialized regulations, which are safety prescriptions relating to particular fields of operation; and
- (c) The Agency’s codes of practice, which establish for particular activities the minimum requirements which, in the light of experience and the current state of technology, must be fulfilled to ensure adequate safety. Codes of practice are, as appropriate, supplemented by safety guides recommending a procedure or procedures that might be followed in implementing them.

“1.3 “Safety measure” means any action, condition or procedure to ensure the observance of safety standards.”

INTERPRETATION OF THE TEXT

A list of definitions that apply for the purposes of these Standards is included. The list of definitions includes: definitions of new terms that are not included in entries in the IAEA Safety Glossary: Terminology Used in Nuclear Safety and Radiation Protection (2007 Edition); revised definitions of terms that are defined in entries in the IAEA Safety Glossary (2007 Edition); and existing terms and their existing definitions from entries in the IAEA Safety Glossary (2007 Edition), included here for ease of reference. The entries for the new terms and for the revised definitions will be included in the next revision of the IAEA Safety Glossary and the revised definitions will, thus, supersede the pre-existing definitions for the purposes of the IAEA safety standards. Further relevant definitions not included here are provided in the IAEA Safety Glossary. See also: <http://www-ns.iaea.org/standards/safety-glossary.asp>

Also included is a CD-ROM containing the IAEA Safety Glossary (2007 Edition) and the Fundamental Safety Principles (2006), each in Arabic, Chinese, English, French, Russian and Spanish versions. The CD-ROM is also available for purchase separately. See: <http://www-pub.iaea.org/MTCD/publications/publications.asp>

The text of these Standards was developed in drafting and review meetings and technical meetings which were held in English only. The IAEA Secretariat made official translations into Arabic, Chinese, French, Russian and Spanish of the draft standard for the purpose of submission of the draft new edition to the IAEA Board of Governors for approval. The approved text was issued in November 2011 as a Safety Requirements publication in an Interim Edition in English only in the IAEA Safety Standards Series. This Interim Edition was submitted to the other Sponsoring Organizations for their approval. Versions of the IAEA Secretariat's official translations of the draft new edition of these Standards into Arabic, Chinese, French, Russian and Spanish for the IAEA Board of Governors were included in the Interim Edition on an accompanying CD-ROM.

The references included herein are to editions that were current as of the time of establishment of these Standards. Editions that supersede these references may be adopted under national legislation. In the event that the publications referenced here are superseded, please refer to the most recent editions. See also: <http://www-ns.iaea.org/standards/>

The IAEA Secretariat is to issue this safety standard as a jointly sponsored IAEA Safety Requirements publication in Arabic, Chinese, French, Russian and Spanish, as well as in English. Comments and queries on matters of technical content, terminology and language, and notification of any apparent errors, omissions or mistranslations, may be submitted by email to Rad.prot.unit@iaea.org and Safety.Standards@iaea.org, or via the IAEA safety standards web site, <http://www-ns.iaea.org/standards/>, for consideration by the IAEA Secretariat in a future review process.

The IAEA, on behalf of all the Sponsoring Organizations, wishes to express its sincere appreciation to all those who assisted in the process of reaching and formulating a consensus, and in the drafting, review and revision of these Standards.

THE IAEA SAFETY STANDARDS

BACKGROUND

Radioactivity is a natural phenomenon and natural sources of radiation are features of the environment. Radiation and radioactive substances have many beneficial applications, ranging from power generation to uses in medicine, industry and agriculture. The radiation risks to workers and the public and to the environment that may arise from these applications have to be assessed and, if necessary, controlled.

Activities such as the medical uses of radiation, the operation of nuclear installations, the production, transport and use of radioactive material, and the management of radioactive waste must therefore be subject to standards of safety.

Regulating safety is a national responsibility. However, radiation risks may transcend national borders, and international cooperation serves to promote and enhance safety globally by exchanging experience and by improving capabilities to control hazards, to prevent accidents, to respond to emergencies and to mitigate any harmful consequences.

States have an obligation of diligence and duty of care, and are expected to fulfil their national and international undertakings and obligations.

International safety standards provide support for States in meeting their obligations under general principles of international law, such as those relating to environmental protection. International safety standards also promote and assure confidence in safety and facilitate international commerce and trade.

A global nuclear safety regime is in place and is being continuously improved. IAEA safety standards, which support the implementation of binding international instruments and national safety infrastructures, are a cornerstone of this global regime. The IAEA safety standards constitute a useful tool for contracting parties to assess their performance under these international conventions.

THE IAEA SAFETY STANDARDS

The status of the IAEA safety standards derives from the IAEA's Statute, which authorizes the IAEA to establish or adopt, in consultation and, where appropriate, in collaboration with the competent organs of the United Nations and with the specialized agencies concerned, standards of safety for protection of health and minimization of danger to life and property, and to provide for their application.

With a view to ensuring the protection of people and the environment from harmful effects of ionizing radiation, the IAEA safety standards establish fundamental safety principles, requirements and measures to control the radiation exposure of people and the release of radioactive material to the environment, to restrict the likelihood of events that might lead to a loss of control over a nuclear reactor core, nuclear chain reaction, radioactive source or any other source of radiation, and to mitigate the consequences of such events if they were to occur. The standards apply to facilities and activities that give rise to radiation risks, including nuclear installations, the use of radiation and radioactive sources, the transport of radioactive material and the management of radioactive waste.

Safety measures and security measures¹ have in common the aim of protecting human life and health and the environment. Safety measures and security measures must be designed and implemented in an integrated manner so that security measures do not compromise safety and safety measures do not compromise security.

The IAEA safety standards reflect an international consensus on what constitutes a high level of safety for protecting people and the environment from harmful effects of ionizing radiation. They are issued in the IAEA Safety Standards Series, which has three categories (see Fig. 1).

Safety Fundamentals

Safety Fundamentals present the fundamental safety objective and principles of protection and safety, and provide the basis for the safety requirements.

Safety Requirements

An integrated and consistent set of Safety Requirements establishes the requirements that must be met to ensure the protection of people and the environment, both now and in the future. The requirements are governed by the objective and principles of the Safety Fundamentals. If the requirements are not met, measures must be taken to reach or restore the required level of safety. The format and style of the requirements facilitate their use for the establishment, in a harmonized manner, of a national regulatory framework. Requirements, including numbered ‘overarching’ requirements, are expressed as ‘shall’ statements. Many requirements are not addressed to a specific party, the implication being that the appropriate parties are responsible for fulfilling them.

¹ See also publications issued in the IAEA Nuclear Security Series.

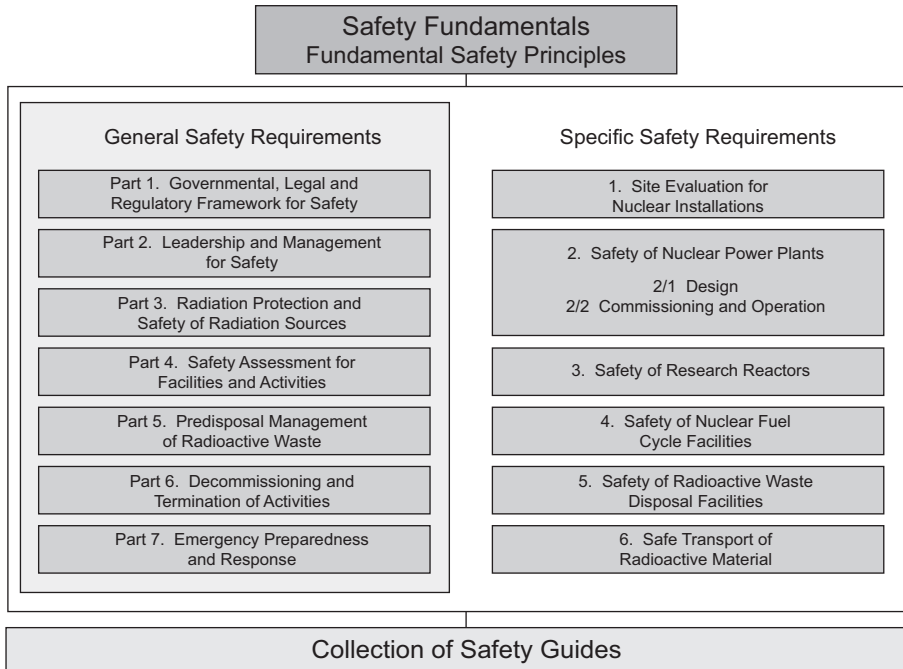


FIG. 1. The long term structure of the IAEA Safety Standards Series.

Safety Guides

Safety Guides provide recommendations and guidance on how to comply with the safety requirements, indicating an international consensus that it is necessary to take the measures recommended (or equivalent alternative measures). The Safety Guides present international good practices, and increasingly they reflect best practices, to help users striving to achieve high levels of safety. The recommendations provided in Safety Guides are expressed as ‘should’ statements.

APPLICATION OF THE IAEA SAFETY STANDARDS

The principal users of safety standards in IAEA Member States are regulatory bodies and other relevant national authorities. The IAEA safety standards are also used by co-sponsoring organizations and by many organizations that design, construct and operate nuclear facilities, as well as organizations involved in the use of radiation and radioactive sources.

The IAEA safety standards are applicable, as relevant, throughout the entire lifetime of all facilities and activities — existing and new — utilized for peaceful purposes and to protective actions to reduce existing radiation risks. They can be used by States as a reference for their national regulations in respect of facilities and activities.

The IAEA's Statute makes the safety standards binding on the IAEA in relation to its own operations and also on States in relation to IAEA assisted operations.

The IAEA safety standards also form the basis for the IAEA's safety review services, and they are used by the IAEA in support of competence building, including the development of educational curricula and training courses.

International conventions contain requirements similar to those in the IAEA safety standards and make them binding on contracting parties. The IAEA safety standards, supplemented by international conventions, industry standards and detailed national requirements, establish a consistent basis for protecting people and the environment. There will also be some special aspects of safety that need to be assessed at the national level. For example, many of the IAEA safety standards, in particular those addressing aspects of safety in planning or design, are intended to apply primarily to new facilities and activities. The requirements established in the IAEA safety standards might not be fully met at some existing facilities that were built to earlier standards. The way in which IAEA safety standards are to be applied to such facilities is a decision for individual States.

The scientific considerations underlying the IAEA safety standards provide an objective basis for decisions concerning safety; however, decision makers must also make informed judgements and must determine how best to balance the benefits of an action or an activity against the associated radiation risks and any other detrimental impacts to which it gives rise.

DEVELOPMENT PROCESS FOR THE IAEA SAFETY STANDARDS

The preparation and review of the safety standards involves the IAEA Secretariat and four safety standards committees, for nuclear safety (NUSSC), radiation safety (RASSC), the safety of radioactive waste (WASSC) and the safe transport of radioactive material (TRANSSC), and a Commission on Safety Standards (CSS) which oversees the IAEA safety standards programme (see Fig. 2).

All IAEA Member States may nominate experts for the safety standards committees and may provide comments on draft standards. The membership of the Commission on Safety Standards is appointed by the Director General and

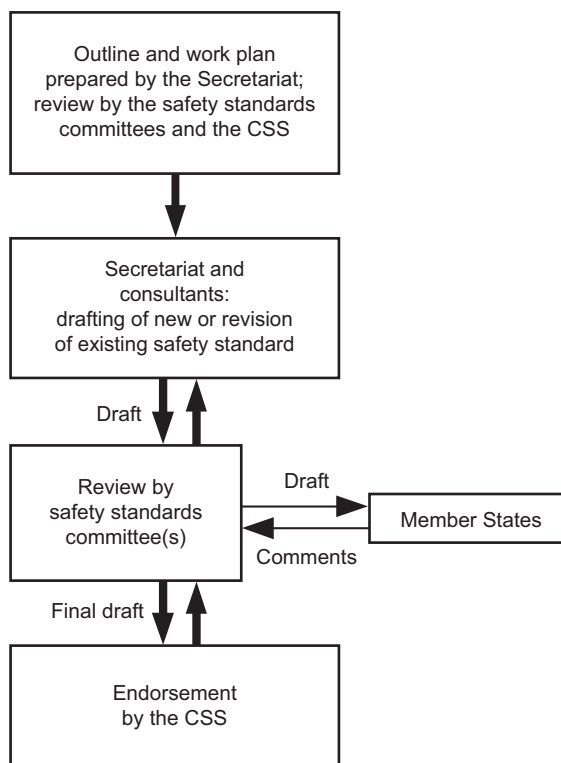


FIG. 2. The process for developing a new safety standard or revising an existing standard.

includes senior governmental officials having responsibility for establishing national standards.

A management system has been established for the processes of planning, developing, reviewing, revising and establishing the IAEA safety standards. It articulates the mandate of the IAEA, the vision for the future application of the safety standards, policies and strategies, and corresponding functions and responsibilities.

INTERACTION WITH OTHER INTERNATIONAL ORGANIZATIONS

The findings of the United Nations Scientific Committee on the Effects of Atomic Radiation (UNSCEAR) and the recommendations of international expert bodies, notably the International Commission on Radiological Protection (ICRP), are taken into account in developing the IAEA safety standards. Some

safety standards are developed in cooperation with other bodies in the United Nations system or other specialized agencies, including the Food and Agriculture Organization of the United Nations, the United Nations Environment Programme, the International Labour Organization, the OECD Nuclear Energy Agency, the Pan American Health Organization and the World Health Organization.

INTERPRETATION OF THE TEXT

Safety related terms are to be understood as defined in the IAEA Safety Glossary (see <http://www-ns.iaea.org/standards/safety-glossary.htm>). Otherwise, words are used with the spellings and meanings assigned to them in the latest edition of The Concise Oxford Dictionary. For Safety Guides, the English version of the text is the authoritative version.

The background and context of each standard in the IAEA Safety Standards Series and its objective, scope and structure are explained in Section 1, Introduction, of each publication.

Material for which there is no appropriate place in the body text (e.g. material that is subsidiary to or separate from the body text, is included in support of statements in the body text, or describes methods of calculation, procedures or limits and conditions) may be presented in appendices or annexes.

An appendix, if included, is considered to form an integral part of the safety standard. Material in an appendix has the same status as the body text, and the IAEA assumes authorship of it. Annexes and footnotes to the main text, if included, are used to provide practical examples or additional information or explanation. Annexes and footnotes are not integral parts of the main text. Annex material published by the IAEA is not necessarily issued under its authorship; material under other authorship may be presented in annexes to the safety standards. Extraneous material presented in annexes is excerpted and adapted as necessary to be generally useful.

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1. INTRODUCTION

BACKGROUND

1.1. This General Safety Requirements publication, IAEA Safety Standards Series No. GSR Part 3, Radiation Protection and Safety of Radiation Sources: International Basic Safety Standards (hereinafter referred to as ‘these Standards’), is issued in the IAEA Safety Standards Series. It supersedes International Basic Safety Standards for Protection against Ionizing Radiation and for the Safety of Radiation Sources issued in 1996 (the ‘BSS of 1996’)¹. Section 1 does not include requirements, but explains the context, concepts and principles for the requirements, which are established in Sections 2–5 and in the schedules.

1.2. Radioactivity is a natural phenomenon and natural sources of radiation are features of the environment. Radiation² and radioactive material may also be of artificial origin and they have many beneficial applications, including uses in medicine, industry, agriculture and research as well as for nuclear power generation. The radiation risks to people and the environment that may arise from the use of radiation and radioactive material must be assessed and must be controlled by means of the application of standards of safety³.

1.3. Exposure of human tissues or organs to radiation can induce the death of cells on a scale that can be extensive enough to impair the function of the exposed tissue or organ. Effects of this type, which are called ‘deterministic effects’, are clinically observable in an individual only if the radiation dose exceeds a certain threshold level. Above this threshold level of dose, a deterministic effect is more severe for a higher dose.

¹ FOOD AND AGRICULTURE ORGANIZATION OF THE UNITED NATIONS, INTERNATIONAL ATOMIC ENERGY AGENCY, INTERNATIONAL LABOUR ORGANISATION, OECD NUCLEAR ENERGY AGENCY, PAN AMERICAN HEALTH ORGANIZATION, WORLD HEALTH ORGANIZATION, International Basic Safety Standards for Protection against Ionizing Radiation and for the Safety of Radiation Sources, IAEA Safety Series No. 115, IAEA, Vienna (1996).

² The term ‘radiation’ in the context of these Standards means ionizing radiation.

³ Obligations expressed as ‘must’ statements in Section 1 are quoted from the Fundamental Safety Principles [1].

1.4. Exposure to radiation can also induce the non-lethal transformation of cells, which may still retain their capacity for cell division. The human body's immune system is very effective at detecting and destroying abnormal cells. However, there is a possibility that the non-lethal transformation of a cell could lead, after a latency period, to cancer in the individual exposed, if the cell is a somatic cell; or such a transformation of a cell could lead to hereditary effects, if the cell is a germ cell. Such effects are called 'stochastic' effects. For the purposes of these Standards, it is assumed that the probability of the eventual occurrence of a stochastic effect is proportional to the dose received, with no threshold. The 'detriment-adjusted nominal risk coefficient of dose', which includes the risks of all cancers and the risks of hereditary effects, is 5% per sievert [2]. This risk coefficient may need to be adjusted as new scientific knowledge becomes available.

1.5. The requirements established in these Standards are governed by the objectives, concepts and principles of the Fundamental Safety Principles [1]. These Standards draw upon information derived from the experience of States in applying the requirements of the BSS of 1996⁴, and from experience in many States in the use of radiation and nuclear techniques. These Standards draw upon extensive research and development work by national and international scientific and engineering organizations on the health effects of radiation exposure and on measures and techniques for the safe design and use of radiation sources. These Standards also take account of the findings of the United Nations Scientific Committee on the Effects of Atomic Radiation (UNSCEAR) [3] and the Recommendations of the International Commission on Radiological Protection (ICRP) [2]. As scientific considerations are only part of the basis for making decisions on protection and safety, these Standards also address the use of value judgements relating to the management of risks.

The system of protection and safety

1.6. As stated in the Fundamental Safety Principles [1], "The fundamental safety objective is to protect people and the environment from harmful effects of ionizing radiation." This objective must be achieved without unduly limiting the

⁴ FOOD AND AGRICULTURE ORGANIZATION OF THE UNITED NATIONS, INTERNATIONAL ATOMIC ENERGY AGENCY, INTERNATIONAL LABOUR ORGANISATION, OECD NUCLEAR ENERGY AGENCY, PAN AMERICAN HEALTH ORGANIZATION, WORLD HEALTH ORGANIZATION, International Basic Safety Standards for Protection against Ionizing Radiation and for the Safety of Radiation Sources, IAEA Safety Series No. 115, IAEA, Vienna (1996).

operation of facilities or the conduct of activities that give rise to radiation risks⁵. Therefore, the system of protection and safety aims to assess, manage and control exposure to radiation so that radiation risks, including risks of health effects and risks to the environment, are reduced to the extent reasonably achievable.

1.7. These Standards are based on the following safety principles stated in the Fundamental Safety Principles [1]:

Principle 1: Responsibility for safety

The prime responsibility for safety must rest with the person or organization responsible for facilities and activities that give rise to radiation risks.

Principle 2: Role of government

An effective legal and governmental framework for safety, including an independent regulatory body, must be established and sustained.

Principle 3: Leadership and management for safety

Effective leadership and management for safety must be established and sustained in organizations concerned with, and facilities and activities that give rise to, radiation risks.

Principle 4: Justification of facilities and activities

Facilities and activities that give rise to radiation risks must yield an overall benefit.

⁵ The term ‘radiation risks’ is used in a general sense to refer to:

- Detrimental health effects of radiation exposure (including the likelihood of such effects occurring).
- Any other safety related risks (including those to the environment) that might arise as a direct consequence of:
 - Exposure to radiation;
 - The presence of radioactive material (including radioactive waste) or its release to the environment;
 - A loss of control over a nuclear reactor core, nuclear chain reaction, radioactive source or any other source of radiation.

Principle 5: Optimization of protection

Protection must be optimized to provide the highest level of safety that can reasonably be achieved.

Principle 6: Limitation of risks to individuals

Measures for controlling radiation risks must ensure that no individual bears an unacceptable risk of harm.

Principle 7: Protection of present and future generations

People and the environment, present and future, must be protected against radiation risks.

Principle 8: Prevention of accidents

All practical efforts must be made to prevent and mitigate nuclear or radiation accidents.

Principle 9: Emergency preparedness and response

Arrangements must be made for emergency preparedness and response for nuclear or radiation incidents.

Principle 10: Protective actions to reduce existing or unregulated radiation risks

Protective actions to reduce existing or unregulated radiation risks must be justified and optimized.

The three general principles of radiation protection, which concern justification, optimization of protection and application of dose limits, are expressed in Safety Principles 4, 5, 6 and also 10 [1].

1.8. The prime responsibility for safety must rest with the person or organization responsible for facilities and activities⁶ that give rise to radiation risks [1]. Other parties also bear certain responsibilities. For instance, suppliers of radiation generators and radioactive sources have responsibilities in relation to their design and manufacture and operating instructions for their safe use. In the case of medical exposures, because of the medical setting in which such exposures occur, primary responsibility for protection and safety for patients lies with the health professional responsible for administration of the radiation dose, who is referred to in these Standards as the ‘radiological medical practitioner’. Other types of health professional may be involved in the preparation for, and the conduct of, radiological procedures, and each type has specific responsibilities, as established in these Standards.

1.9. A properly established governmental, legal and regulatory framework for safety provides for the regulation of facilities and activities that give rise to radiation risks. There is a hierarchy of responsibilities within this framework, from governments to regulatory bodies to the organizations responsible for, and the persons engaged in, activities involving radiation exposure. The government is responsible for the adoption within its national legal system of such legislation, regulations, and standards and measures as may be necessary to fulfil all its national and international obligations effectively, and for the establishment of an independent regulatory body. In some cases, more than one governmental organization may have the functions of a regulatory body for activities within their jurisdictions relating to the control of radiation and radioactive material.

1.10. Both the government and the regulatory body have important responsibilities in establishing the regulatory framework for protecting people and the environment from harmful effects of radiation, including establishing standards. These Standards require the government to ensure that there is coordination of government departments and agencies that have responsibilities

⁶ The term ‘facilities and activities’ is a general term encompassing any human activity that may cause people to be exposed to radiation risks arising from naturally occurring or artificial sources. The term ‘facilities’ includes: nuclear facilities; irradiation installations; some mining and raw material processing facilities such as uranium mines; radioactive waste management facilities; and any other places where radioactive material is produced, processed, used, handled, stored or disposed of — or where radiation generators are installed — on such a scale that consideration of protection and safety is required. The term ‘activities’ includes: the production, use, import and export of radiation sources for industrial, research and medical purposes; the transport of radioactive material; the decommissioning of facilities; radioactive waste management activities such as the discharge of effluents; and some aspects of the remediation of sites affected by residues from past activities.

for protection and safety, including the regulatory body, and of departments and agencies concerned with public health, the environment, labour, mining, science and technology, agriculture and education. Standards have to be developed by means of consultation with those who are or who could be required to apply them.

1.11. The government is also responsible for ensuring, as necessary, that provision is made for support services, such as education and training, and technical services. If these services are not available within the State, other mechanisms to provide them may have to be considered. The regulatory body is responsible for carrying out its required regulatory functions, such as the establishment of requirements and guidelines, the authorization and inspection of facilities and activities, and the enforcement of legislative and regulatory provisions.

1.12. Leadership in safety matters has to be demonstrated at the highest levels in an organization, and safety has to be achieved and maintained by means of an effective management system. This system has to integrate all elements of management so that requirements for protection and safety are established and applied coherently with other requirements, including those for health, human performance, quality, protection of the environment and security, together with economic considerations. The application of the management system also has to ensure the promotion of safety culture, the regular assessment of safety performance and the application of lessons learned from experience. Safety culture includes individual and collective commitment to safety on the part of the leadership, the management and personnel at all levels. The term 'management system' reflects and includes the concept of 'quality control' (controlling the quality of products) and its evolution through 'quality assurance' (the system for ensuring the quality of products) and 'quality management system' (the system for managing quality).

1.13. The operation of facilities or the conduct of activities that introduce a new source of radiation, that change exposures or that change the likelihood of exposures has to be justified in the sense that the detriments that may be caused are outweighed by the individual and societal benefits that are expected. The comparison of detriments and benefits often goes beyond the consideration of protection and safety, and involves the consideration of economic, societal and environmental factors also.

1.14. The application of the justification principle to medical exposures requires a special approach. As an overarching justification of medical exposures, it is accepted that the use of radiation in medicine does more good than harm. However, at the next level, there is a need for generic justification, to be carried

out by the health authority in conjunction with appropriate professional bodies, of a given radiological procedure. This applies to the justification of new technologies and techniques as they evolve. For the final level of justification, the application of the radiological procedure to a given individual has to be considered. The specific objectives of the exposure, the clinical circumstances and the characteristics of the individual involved have to be taken into account by means of referral guidelines developed by professional bodies and the health authority.

1.15. The optimization of protection and safety, when applied to the exposure of workers and members of the public, and carers and comforters of patients undergoing radiological procedures, is a process for ensuring that the likelihood and magnitude of exposures and the number of individuals exposed are as low as reasonably achievable, with economic, societal and environmental factors taken into account. This means that the level of protection would be the best possible under the prevailing circumstances. Optimization is a prospective and iterative process that requires both qualitative and quantitative judgements to be made.

1.16. As is the case with justification, the application of the optimization principle to the medical exposure of patients, and to that of volunteers as part of a programme of biomedical research, requires a special approach. Too low a radiation dose could be as bad as too high a radiation dose, in that the consequence could be that a cancer is not cured or the images obtained are not of suitable diagnostic quality. It is of paramount importance that the medical exposure leads to the required outcome.

1.17. For planned exposure situations, exposures and risks are subject to control to ensure that the specified dose limits for occupational exposure and those for public exposure are not exceeded, and optimization is applied to attain the desired level of protection and safety.

1.18. All practical efforts must be made to prevent and mitigate nuclear or radiological accidents. The most harmful consequences arising from facilities and activities have come from the loss of control over a nuclear reactor core, nuclear chain reaction, radioactive source or other source of radiation. Consequently, to ensure that the likelihood of an accident having harmful consequences is extremely low, measures have to be taken:

- To prevent the occurrence of failures or abnormal conditions (including breaches of security) that could lead to such a loss of control;

- To prevent the escalation of any such failures or abnormal conditions that do occur;
- To prevent the loss of, or the loss of control over, a radioactive source or other source of radiation.

1.19. Arrangements must be made for emergency preparedness and response for nuclear or radiological incidents. The primary goals of preparedness and response for a nuclear or radiological emergency are:

- To ensure that arrangements are in place for an effective response at the scene and, as appropriate, at the local, regional, national and international levels, to a nuclear or radiation emergency;
- To ensure that, for reasonably foreseeable incidents, radiation risks would be minor;
- For any incidents that do occur, to take practical measures to mitigate any consequences for human life and health and the environment.

Types of exposure situation

1.20. For the purpose of establishing practical requirements for protection and safety, these Standards distinguish between three different types of exposure situation: planned exposure situations, emergency exposure situations and existing exposure situations [2]. Together, these three types of exposure situation cover all situations of exposure for which these Standards apply:

- (a) A *planned exposure situation* is a situation of exposure that arises from the planned operation of a source or from a planned activity that results in an exposure due to a source. Since provision for protection and safety can be made before embarking on the activity concerned, the associated exposures and their likelihood of occurrence can be restricted from the outset. The primary means of controlling exposure in planned exposure situations is by good design of facilities, equipment and operating procedures, and by training. In planned exposure situations, exposure at some level can be expected to occur. If exposure is not expected to occur with certainty, but could result from an accident or from an event or a sequence of events that may occur but is not certain to occur, this is referred to as ‘potential exposure’.
- (b) An *emergency exposure situation* is a situation of exposure that arises as a result of an accident, a malicious act or any other unexpected event, and requires prompt action in order to avoid or to reduce adverse consequences. Preventive measures and mitigatory actions have to be considered before

an emergency exposure situation arises. However, once an emergency exposure situation actually arises, exposures can be reduced only by implementing protective actions.

- (c) An *existing exposure situation* is a situation of exposure that already exists when a decision on the need for control needs to be taken. Existing exposure situations include situations of exposure to natural background radiation. They also include situations of exposure due to residual radioactive material that derives from past practices that were not subject to regulatory control or that remains after an emergency exposure situation.

If an event or a sequence of events that has been considered in the assessment of potential exposure does actually occur, it may be treated either as a planned exposure situation or, if an emergency has been declared, as an emergency exposure situation.

1.21. The descriptions that are given in para. 1.20 of the three types of exposure situation are not always sufficient to determine unequivocally which type of exposure situation applies for particular circumstances. For instance, the transitions from an emergency exposure situation to an existing exposure situation may occur progressively over time; and some exposures due to natural sources may have some characteristics of both planned exposure situations and existing exposure situations. In these Standards, the most appropriate type of exposure situation for particular circumstances has been determined by taking practical considerations into account. For the purposes of these Standards, the exposure of aircrew to cosmic radiation is considered under existing exposure situations in Section 5. The exposure of space crew to cosmic radiation presents exceptional circumstances and these are considered separately in Section 5.

Dose constraints and reference levels

1.22. Dose constraints and reference levels are used for optimization of protection and safety, the intended outcome of which is that all exposures are controlled to levels that are as low as reasonably achievable, economic, societal and environmental factors being taken into account. Dose constraints are applied to occupational exposure and to public exposure in planned exposure situations. Dose constraints are set separately for each source under control and they serve as boundary conditions in defining the range of options for the purposes of optimization of protection and safety. Dose constraints are not dose limits: exceeding a dose constraint does not represent non-compliance with regulatory requirements, but it could result in follow-up actions.

1.23. While the objectives of the use of dose constraints for controlling occupational exposure and public exposure are similar, the dose constraints are applied in different ways. For occupational exposure, the dose constraint is a tool to be established and used in the optimization of protection and safety by the person or organization responsible for a facility or an activity. For public exposure in planned exposure situations, the government or the regulatory body ensures the establishment or approval of dose constraints, taking into account the characteristics of the site and of the facility or activity, the scenarios for exposure and the views of interested parties. After exposures have occurred, the dose constraint may be used as a benchmark for assessing the suitability of the optimized strategy for protection and safety (referred to as the protection strategy) that has been implemented and for making adjustments as necessary. The setting of the dose constraint needs to be considered in conjunction with other health and safety provisions and the technology available.

1.24. Reference levels are used for optimization of protection and safety in emergency exposure situations and in existing exposure situations. They are established or approved by the government, the regulatory body or another relevant authority. For occupational exposure and public exposure in emergency exposure situations and in existing exposure situations, a reference level serves as a boundary condition in identifying the range of options for the purposes of optimization in implementing protective actions. The reference level represents the level of dose or the level of risk above which it is judged to be inappropriate to plan to allow exposures to occur, and below which the optimization of protection and safety is implemented. The value chosen for the reference level will depend on the prevailing circumstances for the exposures under consideration. The optimized protection strategies are intended to keep doses below the reference level. When an emergency exposure situation has arisen or an existing exposure situation has been identified, actual exposures could be above or below the reference level. The reference level would be used as a benchmark for judging whether further protective actions are necessary and, if so, in prioritizing their application. Optimization of protection and safety is to be applied in emergency exposure situations and in existing exposure situations, even if the doses initially received are below the reference level.

1.25. The ICRP recommends a range of doses spanning two orders of magnitude within which the value of a dose constraint or reference level would usually be chosen [2]. At the lower end of this range, the dose constraint or reference level represents an increase, of up to about 1 mSv, over the dose received in a year

from exposure due to naturally occurring radiation sources⁷. It would be used when individuals are exposed to radiation from a source that yields little or no benefit for them, but which may benefit society in general. This would be the case, for instance, in establishing dose constraints for public exposure in planned exposure situations.

1.26. Dose constraints or reference levels of 1–20 mSv would be used when the exposure situation — but not necessarily the exposure itself — usually benefits individuals. This would be the case, for instance, when establishing dose constraints for occupational exposure in planned exposure situations or reference levels for exposure of a member of the public in existing exposure situations.

1.27. Reference levels of 20–100 mSv would be used where individuals are exposed to radiation from sources that are not under control or where actions to reduce doses would be disproportionately disruptive. This would be the case, for instance, in establishing reference levels for the residual dose after a nuclear or radiological emergency. Any situation that resulted in a dose of greater than 100 mSv being incurred within a short period of time or in one year would be considered unacceptable, except under the circumstances relating to exposure of emergency workers that are addressed specifically in these Standards.

1.28. The selection of the value for the dose constraint or the reference level would be based on the characteristics of the exposure situation, including:

- The nature of the exposure and the practicability of reducing or preventing the exposure;
- The expected benefits of the exposure for individuals and society, or the benefits of avoiding preventive measures or protective actions that would be detrimental to living conditions, as well as other societal criteria relating to the management of the exposure situation;
- National or regional factors, together with a consideration of international guidance and good practice elsewhere.

1.29. The system of protection and safety required by these Standards includes criteria for protection against exposure due to radon that are based on the average

⁷ According to UNSCEAR [4], the worldwide average annual radiation dose from exposure due to naturally occurring radiation sources, including radon, is 2.4 mSv. In any large population, about 65% would be expected to have annual doses of between 1 and 3 mSv. About 25% of the population would be expected to have annual doses of less than 1 mSv, and about 10% would be expected to have annual doses greater than 3 mSv.

level of risk to a population with typical but various smoking habits. Owing to the synergistic effects of smoking and exposure due to radon, the absolute risk of lung cancer resulting from unit dose from exposure due to radon for people who are smokers is substantially greater than for those who have never smoked [3, 5, 6]. Information provided to people on the risks associated with exposure due to radon needs to highlight this increased risk for smokers.

1.30. Dose constraints are used in optimization of protection and safety for carers and comforters and for volunteers subject to exposure as part of a programme of biomedical research. Dose constraints are not applicable to the exposure of patients in radiological procedures for the purposes of medical diagnosis or treatment.

1.31. In X ray medical imaging, image guided interventional procedures and diagnostic nuclear medicine, a diagnostic reference level is used to indicate the need for an investigation. Periodic assessments are performed of typical doses or activity of the radiopharmaceuticals administered in a medical facility. If comparison with established diagnostic reference levels shows that the typical doses or activity of the radiopharmaceuticals administered are either too high or unusually low, a local review is to be initiated to ascertain whether protection and safety has been optimized and whether any corrective action is required.

Protection of the environment

1.32. In a global and long term perspective, protection of people and the environment against radiation risks associated with the operation of facilities and the conduct of activities — and in particular, protection against such risks that may transcend national borders and may persist for long periods of time — is important for achieving equitable and sustainable development.

1.33. The system of protection and safety required by these Standards generally provides for appropriate protection of the environment from harmful effects of radiation. Nevertheless, international trends in this field show an increasing awareness of the vulnerability of the environment. Trends also indicate the need to be able to demonstrate (rather than to assume) that the environment is being protected against effects of industrial pollutants, including radionuclides, in a wider range of environmental situations, irrespective of any human connection. This is usually accomplished by means of a prospective environmental assessment to identify impacts on the environment, to define the appropriate criteria for protection of the environment, to assess the impacts and to compare

the expected results of the available options for protection. Methods and criteria for such assessments are being developed and will continue to evolve.

1.34. Radiological impacts in a particular environment constitute only one type of impact and, in most cases, may not be the dominant impacts of a particular facility or activity. Furthermore, the assessment of impacts on the environment needs to be viewed in an integrated manner with other features of the system of protection and safety to establish the requirements applicable to a particular source. Since there are complex interrelations, the approach to the protection of people and the environment is not limited to the prevention of radiological effects on humans and on other species. When establishing regulations, an integrated perspective has to be adopted to ensure the sustainability, now and in the future, of agriculture, forestry, fisheries and tourism, and of the use of natural resources. Such an integrated perspective also has to take into account the need to prevent unauthorized acts with potential consequences for and via the environment, including, for example, the illicit dumping of radioactive material and the abandonment of radiation sources. Consideration also needs to be given to the potential for buildup and accumulation of long lived radionuclides released to the environment.

1.35. These Standards are designed to identify the protection of the environment as an issue necessitating assessment, while allowing for flexibility in incorporating into decision making processes the results of environmental assessments that are commensurate with the radiation risks.

Interfaces between safety and security

1.36. Safety measures and security measures have in common the aim of protecting human life and health and the environment. In addition, safety measures and security measures must be designed and implemented in an integrated manner, so that security measures do not compromise safety and safety measures do not compromise security.

1.37. Security infrastructure and safety infrastructure need to be developed, as far as possible, in a well coordinated manner. All the organizations involved need to be made aware of the commonalities and the differences between safety and security so as to be able to factor both into development plans. The synergies between safety and security have to be developed so that safety and security complement and enhance one another.

OBJECTIVE

1.38. These Standards establish requirements for the protection of people and the environment from harmful effects of ionizing radiation and for the safety of radiation sources.

SCOPE

1.39. These Standards apply for protection against ionizing radiation only, which includes gamma rays, X rays and particles such as beta particles, neutrons, protons, alpha particles and heavier ions. While these Standards do not specifically address the control of non-radiological aspects of health, safety and the environment, these aspects also need to be considered. Protection from harmful effects of non-ionizing radiation is outside the scope of these Standards.

1.40. These Standards are intended primarily for use by governments and regulatory bodies. Requirements also apply to principal parties and other parties as specified in Section 2, health authorities, professional bodies and service providers such as technical support organizations.

1.41. These Standards do not deal with security measures. The IAEA issues recommendations on nuclear security in the IAEA Nuclear Security Series.

1.42. These Standards apply to all situations involving radiation exposure that is amenable to control. Exposures deemed to be not amenable to control are excluded from the scope of these Standards.⁸

1.43. These Standards establish requirements to be fulfilled in all facilities and activities giving rise to radiation risks. For certain facilities and activities, such as nuclear installations, radioactive waste management facilities and the transport of radioactive material, other safety requirements, complementary to these Standards, also apply. The IAEA issues Safety Guides to assist in the application of these Standards.

1.44. These Standards apply to three categories of exposure: occupational exposure, public exposure and medical exposure.

⁸ It is generally accepted, for example, that it is not feasible to control ⁴⁰K in the body or cosmic radiation at the surface of the Earth.

1.45. These Standards apply to human activities involving radiation exposure that are:

- Carried out in a State which decides to adopt these Standards or which requests any of the Sponsoring Organizations to provide for the application of these Standards;
- Undertaken by States with the assistance of the Food and Agriculture Organization of the United Nations, the IAEA, the International Labour Organization, the Pan American Health Organization, the United Nations Environment Programme or the World Health Organization, in the light of relevant national rules and regulations;
- Carried out by the IAEA or involving the use of materials, services, equipment, facilities and non-published information made available by the IAEA or at its request or under its control or supervision; or
- Carried out under any bilateral or multilateral arrangement whereby the parties request the IAEA to provide for the application of these Standards.

1.46. Quantities and units used in these Standards are in accordance with the recommendations of the International Commission on Radiation Units and Measurements [7].

STRUCTURE

1.47. The requirements of these Standards are grouped into requirements applicable for all exposure situations and separate sets of requirements for planned exposure situations, emergency exposure situations and existing exposure situations. For each of the three types of exposure situation, the requirements are further grouped into requirements for occupational exposure, public exposure and (for planned exposure situations only) medical exposure.

1.48. The requirements established by these Standards, both numbered ‘overarching’ requirements in bold with titles and other requirements, are expressed as ‘shall’ statements. Each individual overarching requirement is followed by associated requirements.

1.49. Section 2 sets out the requirements that apply generally for all exposure situations and for all of the three categories of exposure (occupational exposure, public exposure and medical exposure). These requirements include the assignment of responsibilities to the government, the regulatory body, and principal parties and other parties with respect to the implementation of a

protection and safety programme and a management system, the promotion of safety culture and the consideration of human factors.

1.50. Section 3 sets out the requirements — in addition to those of Section 2 — for planned exposure situations. Section 3 includes requirements applicable to all three categories of exposure, requirements for the safety of sources, and separate sets of requirements in respect of occupational exposure, public exposure and medical exposure.

1.51. Section 4 sets out the requirements — in addition to those of Section 2 — for emergency exposure situations. Section 4 includes requirements in respect of public exposure and occupational exposure (i.e. exposure of emergency workers) in emergency exposure situations. It also includes requirements on the transitions from an emergency exposure situation to an existing exposure situation.

1.52. Section 5 sets out the requirements — in addition to those of Section 2 — for existing exposure situations. Section 5 includes requirements in respect of public exposure and occupational exposure in existing exposure situations. It includes requirements in respect of remediation of sites and habitation in areas with residual radioactive material, radon in homes and in workplaces, radionuclides in commodities, and exposure of aircrew and of space crew.

1.53. The organization of the requirements in these Standards for the relevant categories of exposure in each type of exposure situation is as shown in Table 1. General requirements for all exposure situations are given in Section 2, and requirements for different exposure situations are given in Sections 3–5. Thus, for any particular facility or activity, more than one section of these Standards will be relevant, as illustrated by the following examples:

- (a) The requirements for the regulatory body given in Section 2 are applicable for all exposure situations and all categories of exposure. They provide the regulatory framework within which persons or organizations responsible for facilities and activities have to comply with requirements placed on them. These requirements, thus, establish the general regulatory responsibilities of the regulatory body. Any further requirements on the regulatory body that apply for one type of exposure situation are given in Sections 3–5. These further requirements are in addition to the requirements given in Section 2.

TABLE 1. ORGANIZATION OF THE REQUIREMENTS OF THESE STANDARDS

	Occupational exposure	Public exposure	Medical exposure
Planned exposure situations	Section 2; Section 3: paras 3.5–3.67 and paras 3.68–3.116	Section 2; Section 3: paras 3.5–3.67 and paras 3.117–3.144	Section 2; Section 3: paras 3.5–3.67 and paras 3.145–3.185
Emergency exposure situations	Section 2; Section 4	Section 2; Section 4	Not applicable
Existing exposure situations	Section 2; Section 5	Section 2; Section 5	Not applicable

- (b) Persons or organizations responsible for a medical facility in which radiation generators or radioactive sources are used are subject to the requirements given in Section 2 for all exposure situations and all categories of exposure, and also to those requirements given in Section 3 that are common to all planned exposure situations (paras 3.5–3.67). In addition, such persons or organizations are subject to the separate requirements given in Section 3 for occupational exposure (such as exposure of medical staff operating medical devices that emit radiation) (paras 3.68–3.116), public exposure (such as exposure in rooms adjacent to rooms containing equipment that generates radiation) (paras 3.117–3.144) and medical exposure (such as exposure of patients) (paras 3.145–3.185).

1.54. Schedules I–IV provide numerical values in support of the requirements, covering exemption and clearance, categorization of sealed sources, dose limits for planned exposure situations and criteria for use in emergency preparedness and response.

1.55. Definitions of terms used are included in these Standards.

2. GENERAL REQUIREMENTS FOR PROTECTION AND SAFETY

DEFINITIONS

2.1. Terms used have the meanings given under Definitions.

INTERPRETATION

2.2. Except as specifically authorized by the statutory governing body of a Sponsoring Organization, no interpretation of these Standards by any officer or employee of the Sponsoring Organization other than a written interpretation by the Director General of the Sponsoring Organization will be binding on the Sponsoring Organization.

RESOLUTION OF CONFLICTS

2.3. The requirements of these Standards are in addition to and not in place of other applicable requirements, such as those of relevant binding conventions and national laws and regulations.

2.4. In cases of conflict between the requirements of these Standards and other applicable requirements, the government or the regulatory body, as appropriate, shall determine which requirements are to be enforced.

2.5. Nothing in these Standards shall be construed as restricting any actions that may otherwise be necessary for protection and safety or as relieving the parties referred to in paras 2.40 and 2.41 from complying with applicable laws and regulations.

ENTRY INTO FORCE

2.6. These Standards shall enter into force one year after the date of their adoption or acknowledgement, as appropriate, by the Sponsoring Organization.

2.7. If a State decides to adopt these Standards, these Standards shall come into force at the time indicated in the formal adoption by that State.

APPLICATION OF THE PRINCIPLES OF RADIATION PROTECTION

Requirement 1: Application of the principles of radiation protection

Parties with responsibilities for protection and safety shall ensure that the principles of radiation protection are applied for all exposure situations.

2.8. For planned exposure situations, each party with responsibilities for protection and safety shall ensure, when relevant requirements apply to that party, that no practice is undertaken unless it is justified.

2.9. For emergency exposure situations and existing exposure situations, each party with responsibilities for protection and safety shall ensure, when relevant requirements apply to that party, that protective actions or remedial actions are justified and are undertaken in such a way as to achieve the objectives set out in a protection strategy.

2.10. For all exposure situations, each party with responsibilities for protection and safety shall ensure, when relevant requirements apply to that party, that protection and safety is optimized⁹.

2.11. For planned exposure situations other than for medical exposure, each party with responsibilities for protection and safety shall ensure that, when relevant requirements apply to that party, specified dose limits are not exceeded.

2.12. The application of the requirements for the system of protection and safety shall be commensurate with the radiation risks associated with the exposure situation.

⁹ 'Protection and safety is optimized' means that optimization of protection and safety has been applied and the result of that process has been implemented.

RESPONSIBILITIES OF THE GOVERNMENT¹⁰

Requirement 2: Establishment of a legal and regulatory framework

The government shall establish and maintain a legal and regulatory framework for protection and safety and shall establish an effectively independent regulatory body with specified responsibilities and functions.

2.13. The government shall establish and maintain an appropriate and effective legal and regulatory framework for protection and safety in all exposure situations.¹¹ This framework shall encompass both the assignment and the discharge of governmental responsibilities, and the regulatory control of facilities and activities that give rise to radiation risks. The framework shall allow for the fulfilment of international obligations.

2.14. The government shall ensure that adequate arrangements are in place for the protection of people and the environment, both now and in the future, against harmful effects of ionizing radiation, without unduly limiting the operation of facilities or the conduct of activities that give rise to radiation risks. This shall include arrangements for the protection of people of present and future generations and populations remote from present facilities and activities.

2.15. The government shall establish legislation that, among other things:

- (a) Provides the statutory basis for requirements for protection and safety for all exposure situations;
- (b) Specifies that the prime responsibility for protection and safety rests with the person or organization responsible for facilities and activities that give rise to radiation risks;
- (c) Specifies the scope of its applicability;
- (d) Establishes and provides for maintaining an independent regulatory body with clearly specified functions and responsibilities for the regulation of protection and safety;
- (e) Provides for coordination between authorities with responsibilities relevant to protection and safety for all exposure situations.

¹⁰ States have different legal structures, and, therefore, the term ‘government’ as used in the IAEA safety standards is to be understood in a broad sense, and is accordingly interchangeable here with the term ‘State’.

¹¹ Requirements on the governmental, legal and regulatory framework for safety of facilities and activities are established in Ref. [8].

2.16. The government shall ensure that the regulatory body is effectively independent, in making decisions relating to protection and safety, of persons and organizations using or promoting the use of radiation and radioactive material, so that it is free from any undue influence by interested parties and from any conflicts of interest; and shall ensure that it has functional separation from entities having responsibilities or interests that could unduly influence its decision making.

2.17. The government shall ensure that the regulatory body has the legal authority, competence and resources necessary to fulfil its statutory functions and responsibilities.

2.18. The government shall ensure that a graded approach is taken to the regulatory control of radiation exposure, so that the application of regulatory requirements is commensurate with the radiation risks associated with the exposure situation.

2.19. The government shall establish mechanisms to ensure that:

- (a) The activities of the regulatory body are coordinated with those of other governmental authorities, in accordance with para. 2.15(e), and with national and international organizations that have related responsibilities;
- (b) Interested parties are involved as appropriate in regulatory decision making processes or regulatory decision aiding processes.

2.20. The government shall ensure that arrangements are in place at the national level for making decisions relating to protection and safety that fall outside the authority of the regulatory body.

2.21. The government shall ensure that requirements are established for:

- (a) Education, training, qualification and competence in protection and safety of all persons engaged in activities relevant to protection and safety;
- (b) The formal recognition¹² of qualified experts;
- (c) The competence of organizations that have responsibilities relating to protection and safety.

¹² 'Formal recognition' means documented acknowledgement by the relevant authority that a person has the qualifications and expertise required for the responsibilities that he or she will bear in the conduct of the authorized activity.

2.22. The government shall ensure that arrangements are in place for the provision of the education and training services required for building and maintaining the competence of persons and organizations that have responsibilities relating to protection and safety.

2.23. The government shall ensure that arrangements are in place for the provision of technical services relating to protection and safety, such as services for personal dosimetry, environmental monitoring and the calibration of monitoring and measuring equipment.

2.24. The government shall ensure that arrangements are in place for the safe decommissioning of facilities [9], the safe management of radioactive waste [10, 11] and the safe management of spent fuel.

2.25. The government shall ensure that the transport of radioactive material is in accordance with the IAEA Regulations for the Safe Transport of Radioactive Material (the IAEA Transport Regulations) [12] and with any applicable international conventions, taking into consideration other internationally endorsed standards and recommendations derived from the IAEA Transport Regulations.¹³

2.26. The government shall ensure that arrangements are in place for regaining control over radioactive sources that have been abandoned, lost, misplaced, stolen or otherwise transferred without proper authorization.

2.27. The government shall ensure that infrastructural arrangements are in place for the interfaces between safety and the security of radioactive sources.

2.28. In establishing the legal and regulatory framework for protection and safety, the government:

- (a) Shall fulfil its respective international obligations;
- (b) Shall allow for participation in relevant international arrangements, including international peer reviews;
- (c) Shall promote international cooperation to enhance safety globally.

¹³ Additional measures are taken for security in the transport of radioactive material. The IAEA issues guidance on security in the transport of radioactive material in the IAEA Nuclear Security Series of publications.

RESPONSIBILITIES OF THE REGULATORY BODY

Requirement 3: Responsibilities of the regulatory body

The regulatory body shall establish or adopt regulations and guides for protection and safety and shall establish a system to ensure their implementation.

2.29. The regulatory body shall establish requirements for the application of the principles of radiation protection specified in paras 2.8–2.12 for all exposure situations and shall establish or adopt regulations and guides for protection and safety.

2.30. The regulatory body shall establish a regulatory system for protection and safety that includes [8]:

- (a) Notification and authorization;
- (b) Review and assessment of facilities and activities;
- (c) Inspection of facilities and activities;
- (d) Enforcement of regulatory requirements;
- (e) The regulatory functions relevant to emergency exposure situations and existing exposure situations;
- (f) Provision of information to, and consultation with, parties affected by its decisions and, as appropriate, the public and other interested parties.

2.31. The regulatory body shall adopt a graded approach to the implementation of the system of protection and safety, such that the application of regulatory requirements is commensurate with the radiation risks associated with the exposure situation.

2.32. The regulatory body shall ensure the application of the requirements for education, training, qualification and competence in protection and safety of all persons engaged in activities relevant to protection and safety.

2.33. The regulatory body shall ensure that mechanisms are in place for the timely dissemination of information to relevant parties, such as suppliers of and users of sources, on lessons learned for protection and safety from regulatory experience and operating experience, and from incidents and accidents and the related findings. The mechanisms established shall, as appropriate, be used to provide relevant information to other relevant organizations at the national and international level.

2.34. The regulatory body, in conjunction with other relevant authorities, shall specify requirements for acceptance and for performance, by regulation or by the application of published standards, for any manufactured or constructed source, device, equipment or facility that, when in use, has implications for protection and safety.

2.35. The regulatory body shall make provision for establishing, maintaining and retrieving adequate records relating to facilities and activities. These records shall include:

- Registers of sealed sources and radiation generators¹⁴;
- Records of doses from occupational exposure;
- Records relating to the safety of facilities and activities;
- Records that might be necessary for the shutdown and decommissioning or closure of facilities;
- Records of events, including non-routine releases of radioactive material to the environment;
- Inventories of radioactive waste and of spent fuel.

2.36. The regulatory body shall establish mechanisms for communication and discussion that involve professional and constructive interactions with relevant parties for all protection and safety related issues.

2.37. The regulatory body, in consultation with the health authority, shall ensure that provisions are in place for ensuring protection and safety in the handling of deceased persons or human remains that are known to contain sealed or unsealed radioactive sources, either as a result of radiological procedures for medical treatment of patients or as a consequence of an emergency.

2.38. The regulatory body shall establish, implement, assess and strive to continually improve a management system that is aligned with the goals of the regulatory body and that contributes to the achievement of those goals.

¹⁴ The regulatory body specifies which sources are to be included in the registers and inventories, with due consideration given to the associated risks.

RESPONSIBILITIES FOR PROTECTION AND SAFETY

Requirement 4: Responsibilities for protection and safety

The person or organization responsible for facilities and activities that give rise to radiation risks shall have the prime responsibility for protection and safety. Other parties shall have specified responsibilities for protection and safety.

2.39. The person or organization responsible for any facility or activity that gives rise to radiation risks shall have the prime responsibility for protection and safety, which cannot be delegated.

2.40. The principal parties responsible for protection and safety are:

- (a) Registrants or licensees, or the person or organization responsible for facilities and activities for which notification only is required;
- (b) Employers, in relation to occupational exposure;
- (c) Radiological medical practitioners, in relation to medical exposure;
- (d) Those persons or organizations designated to deal with emergency exposure situations or existing exposure situations.

2.41. Other parties shall have specified responsibilities in relation to protection and safety. These other parties include:

- (a) Suppliers of sources, providers of equipment and software, and providers of consumer products;
- (b) Radiation protection officers;
- (c) Referring medical practitioners;
- (d) Medical physicists;
- (e) Medical radiation technologists;
- (f) Qualified experts or any other party to whom a principal party has assigned specific responsibilities;
- (g) Workers other than workers listed in (a)–(f) in this paragraph;
- (h) Ethics committees.

2.42. The relevant principal parties shall establish and implement a protection and safety programme that is appropriate for the exposure situation. The protection and safety programme:

- (a) Shall adopt objectives for protection and safety in accordance with the requirements of these Standards;
- (b) Shall apply measures for protection and safety that are commensurate with the radiation risks associated with the exposure situation and that are adequate to ensure compliance with the requirements of these Standards.

2.43. The relevant principal parties shall ensure that, in the implementation of the protection and safety programme:

- (a) The measures and resources that are necessary for achieving the objectives for protection and safety have been determined and are duly provided;
- (b) The programme is periodically reviewed to assess its effectiveness and its continued fitness for purpose;
- (c) Any failures or shortcomings in protection and safety are identified and corrected, and steps are taken to prevent their recurrence;
- (d) Arrangements are made to consult with interested parties;
- (e) Appropriate records are maintained.

2.44. The relevant principal parties and other parties having specified responsibilities in relation to protection and safety shall ensure that all personnel engaged in activities relevant to protection and safety have appropriate education, training and qualification so that they understand their responsibilities and can perform their duties competently, with appropriate judgement and in accordance with procedures.

2.45. The relevant principal parties shall permit access by authorized representatives of the regulatory body to carry out inspections of their facilities and activities and of their protection and safety records, and shall cooperate in the conduct of inspections.

2.46. The relevant principal parties shall ensure that qualified experts are identified and are consulted as necessary on the proper observance of these Standards.

MANAGEMENT REQUIREMENTS

Requirement 5: Management for protection and safety

The principal parties shall ensure that protection and safety are effectively integrated into the overall management system of the organizations for which they are responsible.

Protection and safety elements of the management system

2.47. The principal parties shall demonstrate commitment to protection and safety at the highest levels within the organizations for which they are responsible.

2.48. The principal parties shall ensure that the management system¹⁵ is designed and applied to enhance protection and safety by:

- (a) Applying the requirements for protection and safety coherently with other requirements, including requirements for operational performance, and coherently with guidelines for security;
- (b) Describing the planned and systematic actions necessary to provide adequate confidence that the requirements for protection and safety are fulfilled;
- (c) Ensuring that protection and safety are not compromised by other requirements;
- (d) Providing for the regular assessment of performance for protection and safety, and the application of lessons learned from experience;
- (e) Promoting safety culture.

2.49. The principal parties shall ensure that protection and safety elements of the management system are commensurate with the complexity of and the radiation risks associated with the activity.

2.50. The principal parties shall be able to demonstrate the effective fulfilment of the requirements for protection and safety in the management system.

¹⁵ Requirements on the management system for facilities and activities are established in Ref. [13].

Safety culture

2.51. The principal parties shall promote and maintain safety culture by:

- (a) Promoting individual and collective commitment to protection and safety at all levels of the organization;
- (b) Ensuring a common understanding of the key aspects of safety culture within the organization;
- (c) Providing the means by which the organization supports individuals and teams in carrying out their tasks safely and successfully, with account taken of the interactions between individuals, technology and the organization;
- (d) Encouraging the participation of workers and their representatives and other relevant persons in the development and implementation of policies, rules and procedures dealing with protection and safety;
- (e) Ensuring accountability of the organization and of individuals at all levels for protection and safety;
- (f) Encouraging open communication with regard to protection and safety within the organization and with relevant parties, as appropriate;
- (g) Encouraging a questioning and learning attitude, and discouraging complacency, with regard to protection and safety;
- (h) Providing means by which the organization continually seeks to develop and strengthen its safety culture.

Human factors

2.52. The principal parties and other parties having specified responsibilities in relation to protection and safety, as appropriate, shall take into account human factors and shall support good performance and good practices to prevent human and organizational failures, by ensuring among other things that:

- (a) Sound ergonomic principles are followed in the design of equipment and the development of operating procedures, so as to facilitate the safe operation and use of equipment, to minimize the possibility that operator errors could lead to accidents, and to reduce the possibility that indications of normal conditions and abnormal conditions could be misinterpreted.
- (b) Appropriate equipment, safety systems and procedural requirements are provided, and other necessary provision is made:
 - (i) To reduce, as far as practicable, the possibility that human errors or inadvertent actions could give rise to accidents or to other incidents leading to the exposure of any person;

- (ii) To provide means for detecting human errors and for correcting them or compensating for them;
- (iii) To facilitate protective actions and corrective actions in the event of failures of safety systems or failures of measures for protection and safety.

3. PLANNED EXPOSURE SITUATIONS

SCOPE

3.1. The requirements for planned exposure situations apply to the following practices:

- (a) The production, supply, provision and transport of radioactive material and of devices that contain radioactive material, including sealed sources and unsealed sources, and of consumer products;
- (b) The production and supply of devices that generate radiation, including linear accelerators, cyclotrons, and fixed and mobile radiography equipment;
- (c) The generation of nuclear power, including any activities within the nuclear fuel cycle that involve or that could involve exposure to radiation or exposure due to radioactive material;
- (d) The use of radiation or radioactive material for medical, industrial, veterinary, agricultural, legal or security purposes, including the use of associated equipment, software or devices where such use could affect exposure to radiation;
- (e) The use of radiation or radioactive material for education, training or research, including any activities relating to such use that involve or could involve exposure to radiation or exposure due to radioactive material;
- (f) The mining and processing of raw materials that involve exposure due to radioactive material;
- (g) Any other practice as specified by the regulatory body.

3.2. The requirements for planned exposure situations apply to exposure due to sources within practices¹⁶, as follows:

- (a) Facilities that contain radioactive material and facilities that contain radiation generators, including nuclear installations, medical radiation facilities, veterinary radiation facilities, facilities for the management of radioactive waste, installations for the processing of radioactive material, irradiation facilities, and mineral extraction and mineral processing facilities that involve or could involve exposure to radiation or exposure due to radioactive material;
- (b) Individual sources of radiation, including sources within the types of facility mentioned in para. 3.2(a), as appropriate, in accordance with the requirements of the regulatory body.

3.3. The requirements for planned exposure situations apply for any occupational exposure, medical exposure or public exposure due to any practice or due to a source within a practice as specified in paras 3.1 and 3.2.

3.4. Exposure due to natural sources is, in general, considered an existing exposure situation and is subject to the requirements in Section 5. However, the relevant requirements in Section 3 for planned exposure situations apply to:

- (a) Exposure due to material¹⁷ in any practice specified in para. 3.1 where the activity concentration in the material of any radionuclide in the uranium decay chain or the thorium decay chain is greater than 1 Bq/g or the activity concentration of ⁴⁰K is greater than 10 Bq/g;
- (b) Public exposure due to discharges or due to the management of radioactive waste arising from a practice involving material as specified in (a) above;

¹⁶ For example, a sterilization gamma irradiation unit is a source for the practice of radiation preservation of food. An X ray unit may be a source for the practice of radiodiagnosis. A nuclear power plant is part of the practice of generating electricity by nuclear fission, and may be regarded as a single source (e.g. with respect to discharges) or as a collection of sources (e.g. for purposes of occupational radiation protection). A complex or multiple installation situated at one location or site may, as appropriate, be considered a single source for the purposes of application of these Standards.

¹⁷ A situation of exposure due to radionuclides of natural origin in food, feed, drinking water, agricultural fertilizer and soil amendments, construction materials and residual radioactive material in the environment is treated as an existing exposure situation regardless of the activity concentrations of the radionuclides concerned.

- (c) Exposure due to ^{222}Rn and to ^{222}Rn progeny and due to ^{220}Rn and to ^{220}Rn progeny in workplaces in which occupational exposure due to other radionuclides in the uranium decay chain or the thorium decay chain is controlled as a planned exposure situation;
- (d) Exposure due to ^{222}Rn and to ^{222}Rn progeny where the annual average activity concentration of ^{222}Rn in air in workplaces remains above the reference level established in accordance with para. 5.27 after the fulfilment of the requirement in para. 5.28.

GENERIC REQUIREMENTS

3.5. No person or organization shall adopt, introduce, conduct, discontinue or cease a practice, or shall, as applicable, mine, extract, process, design, manufacture, construct, assemble, install, acquire, import, export, supply, provide, distribute, loan, hire, receive, site, locate, commission, possess, use, operate, maintain, repair, transfer, decommission, disassemble, transport, store or dispose of a source within a practice other than in accordance with the requirements of these Standards.

Requirement 6: Graded approach

The application of the requirements of these Standards in planned exposure situations shall be commensurate with the characteristics of the practice or the source within a practice, and with the likelihood and magnitude of exposures.

3.6. The application of the requirements of these Standards shall be in accordance with the graded approach and shall also conform to any requirements specified by the regulatory body. Not all the requirements of these Standards are relevant for every practice or source, or for all the actions specified in para. 3.5.

Requirement 7: Notification and authorization

Any person or organization intending to operate a facility or to conduct an activity shall submit to the regulatory body a notification and, as appropriate, an application for authorization.

Notification

3.7. Any person or organization intending to carry out any of the actions specified in para. 3.5 shall submit a notification to the regulatory body of such an intention¹⁸. Notification alone is sufficient provided that the exposures expected to be associated with the practice or action are unlikely to exceed a small fraction, as specified by the regulatory body, of the relevant limits, and that the likelihood and magnitude of potential exposures and any other potential detrimental consequences are negligible. Notification is required for consumer products only with respect to manufacture, maintenance, import, export, provision, distribution and, in some cases, disposal.

Authorization: Registration or licensing

3.8. Any person or organization intending to carry out any of the actions specified in para. 3.5 shall, unless notification alone is sufficient, apply to the regulatory body for authorization¹⁸, which shall take the form of either registration¹⁹ or licensing.

3.9. Any person or organization applying for authorization:

- (a) Shall submit to the regulatory body the relevant information necessary to support the application;
- (b) Shall refrain from carrying out any of the actions specified in para. 3.5 until the registration or the licence has been issued;
- (c) Shall assess the nature, likelihood and magnitude of the expected exposures due to the source and shall take all necessary measures for protection and safety;
- (d) Shall, if there is a possibility for an exposure to be greater than a level as specified by the regulatory body, have a safety assessment made and submitted to the regulatory body as part of the application;

¹⁸ With regard to material being transported in accordance with the IAEA Transport Regulations [12], the requirements of these Standards for notification and authorization are fulfilled by means of compliance with the requirements of the IAEA Transport Regulations.

¹⁹ Typical practices that are suitable for registration are those for which: (i) safety can largely be ensured by the design of the facilities and equipment; (ii) the operating procedures are simple to follow; (iii) the training requirements for safety are minimal; and (iv) there is a history of few problems relating to safety in operations. Registration is best suited to those practices for which operations do not vary significantly.

- (e) Shall, as required by the regulatory body, have an appropriate prospective assessment made for radiological environmental impacts, commensurate with the radiation risks associated with the facility or activity.

Requirement 8: Exemption and clearance

The government or the regulatory body shall determine which practices or sources within practices are to be exempted from some or all of the requirements of these Standards. The regulatory body shall approve which sources, including materials and objects, within notified practices or authorized practices may be cleared from regulatory control.

Exemption

3.10. The government or the regulatory body shall determine which practices or sources within practices are to be exempted from some or all of the requirements of these Standards, including the requirements for notification, registration or licensing, using as the basis for this determination the criteria for exemption specified in Schedule I or any exemption levels specified by the regulatory body on the basis of these criteria.

3.11. Exemption shall not be granted for practices deemed to be not justified.

Clearance

3.12. The regulatory body shall approve which sources, including materials and objects, within notified or authorized practices may be cleared from regulatory control, using as the basis for such approval the criteria for clearance specified in Schedule I or any clearance levels specified by the regulatory body on the basis of these criteria. By means of this approval, the regulatory body shall ensure that sources that have been cleared from regulatory control do not again become subject to the requirements for notification, registration or licensing unless it so specifies.

Requirement 9: Responsibilities of registrants and licensees in planned exposure situations

Registrants and licensees shall be responsible for protection and safety in planned exposure situations.

3.13. Registrants and licensees shall bear the responsibility for setting up and implementing the technical and organizational measures that are necessary for protection and safety for the practices and sources for which they are authorized. Registrants and licensees may designate suitably qualified persons to carry out tasks relating to these responsibilities, but they shall retain the prime responsibility for protection and safety. Registrants and licensees shall document the names and responsibilities of persons designated to ensure compliance with the requirements of these Standards.

3.14. Registrants and licensees shall notify the regulatory body of any intention to introduce modifications to any practice or source for which they are authorized, whenever the modifications could have significant implications for protection and safety, and they shall not carry out any such modification unless it is specifically authorized by the regulatory body.

3.15. Registrants and licensees:

- (a) Shall establish clear lines of responsibility and accountability for protection and safety for the sources for which they are authorized, and shall establish organizational arrangements for protection and safety;
- (b) Shall ensure that any delegation of responsibilities by a principal party is documented;
- (c) Shall, for the sources for which they are authorized and for which a safety assessment is required in para. 3.9(d), conduct such a safety assessment and keep it up to date in accordance with para. 3.35;
- (d) Shall, for the sources for which they are authorized and for which the regulatory body requires a prospective assessment to be made for radiological environmental impacts (see para. 3.9(e)), conduct such an assessment and keep it up to date;
- (e) Shall assess the likelihood and magnitude of potential exposures, their likely consequences and the number of individuals who may be affected by them;
- (f) Shall have in place operating procedures and arrangements for protection and safety that are subject to periodic review and updating under a management system;
- (g) Shall establish procedures for reporting on and learning from accidents and other incidents;
- (h) Shall establish arrangements for the periodic review of the overall effectiveness of the measures for protection and safety;

- (i) Shall ensure that adequate maintenance, testing and servicing are carried out as necessary so that sources remain capable of fulfilling their design requirements for protection and safety throughout their lifetime;
- (j) Shall ensure safe management of and control over all radioactive waste that is generated, and shall dispose of such waste in accordance with the regulatory requirements.

Requirement 10: Justification of practices

The government or the regulatory body shall ensure that only justified practices are authorized.

3.16. The government or the regulatory body, as appropriate, shall ensure that provision²⁰ is made for the justification of any type of practice²¹ and for review of the justification, as necessary, and shall ensure that only justified practices are authorized.

3.17. The following practices are deemed to be not justified:

- (a) Practices, except for justified practices involving medical exposure²², that result in an increase in activity, by the deliberate addition of radioactive substances or by activation²³, in food, feed, beverages, cosmetics or any other commodity or product intended for ingestion, inhalation or percutaneous intake by, or application to, a person;
- (b) Practices involving the frivolous use of radiation or radioactive substances in commodities or in consumer products such as toys and personal jewellery or adornments, which result in an increase in activity, by the deliberate addition of radioactive substances or by activation²³;
- (c) Human imaging using radiation that is performed as a form of art or for publicity purposes.

²⁰ Such provision may involve several governmental authorities not necessarily having direct responsibility for protection and safety, such as ministries of health, justice, immigration and security.

²¹ This provision for the justification of any type of practice includes practices for which notification alone is sufficient.

²² Particular requirements for the justification of medical exposure are specified in paras 3.155–3.161.

²³ This requirement is not intended to prohibit those practices that may involve the short term activation of commodities or products, for which there is no increase in radioactivity in the commodity or product as made available.

3.18. Human imaging using radiation that is performed for occupational, legal or health insurance purposes²⁴, and is undertaken without reference to clinical indication, shall normally be deemed to be not justified. If, in exceptional circumstances, the government or the regulatory body decides that the justification of such human imaging for specific practices is to be considered, the requirements of paras 3.61–3.64 and 3.66 shall apply.

3.19. Human imaging using radiation for theft detection purposes shall be deemed to be not justified.

3.20. Human imaging using radiation for the detection of concealed objects for anti-smuggling purposes shall normally be deemed to be not justified. If, in exceptional circumstances, the government or the regulatory body decides that the justification of such human imaging is to be considered, the requirements of paras 3.61–3.67 shall apply.

3.21. Human imaging using radiation for the detection of concealed objects that can be used for criminal acts that pose a national security threat shall be justified only by the government. If the government decides that the justification of such human imaging is to be considered, the requirements of paras 3.61–3.67 shall apply.

Requirement 11: Optimization of protection and safety

The government or the regulatory body shall establish and enforce requirements for the optimization of protection and safety, and registrants and licensees shall ensure that protection and safety is optimized.

3.22. The government or the regulatory body:

- (a) Shall establish and enforce requirements for the optimization of protection and safety;
- (b) Shall require documentation addressing the optimization of protection and safety;

²⁴ Such purposes for performing human imaging using radiation include: assessment of fitness for employment (prior to employment or periodically during employment); assessment of physiological suitability for a career or a sport; assessment of athletes before a selection or transfer; determination of age for legal purposes; obtaining evidence for legal purposes; detection of drugs concealed within the body; immigration or emigration requirements; pre-insurance checks; and obtaining evidence for the purposes of a compensation claim.

- (c) Shall establish or approve constraints²⁵ on dose and on risk, as appropriate, or shall establish or approve a process for establishing such constraints, to be used in the optimization of protection and safety.

3.23. Registrants and licensees shall ensure that protection and safety is optimized.

3.24. For occupational exposure and public exposure²⁶, registrants and licensees shall ensure that all relevant factors are taken into account in a coherent way in the optimization of protection and safety to contribute to achieving the following objectives:

- (a) To determine measures for protection and safety that are optimized for the prevailing circumstances, with account taken of the available options for protection and safety as well as the nature, likelihood and magnitude of exposures;
- (b) To establish criteria, on the basis of the results of the optimization, for the restriction of the likelihood and magnitudes of exposures by means of measures for preventing accidents and for mitigating the consequences of those that do occur.

3.25. For occupational exposure and public exposure, registrants and licensees shall ensure, as appropriate, that relevant constraints are used in the optimization of protection and safety for any particular source within a practice.²⁵

Requirement 12: Dose limits

The government or the regulatory body shall establish dose limits for occupational exposure and public exposure, and registrants and licensees shall apply these limits.

²⁵ For occupational exposure, the relevant dose constraint is on individual doses to workers, established and used by registrants and licensees to set the range of options in optimizing protection and safety for the source. For public exposure, the relevant dose constraint is a source related value established or approved by the government or the regulatory body, with account taken of the doses from planned operations of all sources under control. The dose constraint for each particular source is intended, among other things, to ensure that the sum of doses from planned operations for all sources under control remains within the dose limit.

²⁶ Requirements for the optimization of medical exposure are specified in paras 3.162–3.177.

3.26. The government or the regulatory body shall establish and the regulatory body shall enforce compliance with the dose limits specified in Schedule III for occupational exposures and public exposures in planned exposure situations.

3.27. The government or the regulatory body shall determine what additional restrictions, if any, are required to be complied with by registrants and licensees to ensure that the dose limits specified in Schedule III are not exceeded owing to possible combinations of doses from exposures due to different authorized practices.

3.28. Registrants and licensees shall ensure that the exposures of individuals due to the practices for which the registrants and licensees are authorized are restricted, so that neither the effective dose nor the equivalent dose to tissues or organs exceeds any relevant dose limit specified in Schedule III.²⁷

Requirement 13: Safety assessment

The regulatory body shall establish and enforce requirements for safety assessment, and the person or organization responsible for a facility or activity that gives rise to radiation risks shall conduct an appropriate safety assessment of this facility or activity.

3.29. The regulatory body shall establish requirements for persons or organizations responsible for facilities and activities that give rise to radiation risks to conduct an appropriate safety assessment²⁸. Prior to the granting of an authorization, the responsible person or organization shall be required to submit a safety assessment, which shall be reviewed and assessed by the regulatory body.

3.30. The person or organization, as required under para. 3.9(d), or registrants and licensees, as appropriate, shall conduct a safety assessment that is either generic or specific to the practice or source for which they are responsible.²⁹

²⁷ Dose limits do not apply to medical exposures.

²⁸ Requirements on safety assessment for facilities and activities are established in Ref. [14].

²⁹ A generic safety assessment is usually sufficient for types of source with a high degree of uniformity in design. A specific safety assessment is usually required in other cases; however, the specific safety assessment need not include those aspects covered by a generic safety assessment, if a generic safety assessment has been conducted for the type of source.

3.31. Safety assessments shall be conducted at different stages, including the stages of siting, design, manufacture, construction, assembly, commissioning, operation, maintenance and decommissioning (or closure) of facilities or parts thereof, as appropriate, so as:

- (a) To identify the ways in which exposures could be incurred, account being taken of the effects of external events as well as of events directly involving the sources and associated equipment;
- (b) To determine the expected likelihood and magnitudes of exposures in normal operation and, to the extent reasonable and practicable, to make an assessment of potential exposures;
- (c) To assess the adequacy of the provisions for protection and safety.

3.32. The safety assessment shall include, as appropriate, a systematic critical review of:

- (a) The operational limits and conditions for the operation of the facility;
- (b) The ways in which structures, systems and components, including software, and procedures relating to protection and safety might fail, singly or in combination, or might otherwise give rise to exposures, and the consequences of such events;
- (c) The ways in which external factors could affect protection and safety;
- (d) The ways in which operating procedures relating to protection and safety might be erroneous, and the consequences of such errors;
- (e) The implications for protection and safety of any modifications;
- (f) The implications for protection and safety of security measures or of any modifications to security measures;
- (g) Any uncertainties or assumptions and their implications for protection and safety.

3.33. The registrant or licensee shall take into account in the safety assessment:

- (a) Factors that could give rise to a substantial release of radioactive material, the measures available to prevent or to control such a release, and the maximum activity of radioactive material that, in the event of a major failure of the containment, could be released to the environment;
- (b) Factors that could give rise to a smaller but continuing release of radioactive material, and the measures available to detect and to prevent or to control such a release;

- (c) Factors that could give rise to unintended operation of any radiation generator or a loss of shielding, and the measures available to detect and to prevent or to control such occurrences;
- (d) The extent to which the use of redundant and diverse safety features that are independent of each other, so that failure of one does not result in failure of any other, is appropriate to restrict the likelihood and magnitude of potential exposures.

3.34. Registrants and licensees shall ensure that the safety assessment is documented and, where appropriate, that it is independently reviewed under the relevant management system.

3.35. Registrants and licensees shall perform additional reviews of the safety assessment as necessary to ensure that the technical specifications or conditions of use continue to be met when:

- (a) Significant modifications to the facility or to its operating procedures or maintenance procedures are envisaged;
- (b) Significant changes occur on the site that could affect the safety of the facility or of activities on the site;
- (c) Information on operating experience, or information about accidents and other incidents that could result in exposures, indicates that the current assessment might be invalid;
- (d) Any significant changes in activities are envisaged;
- (e) Any relevant changes in guidelines or standards have been made or are envisaged.

3.36. If as a result of a safety assessment, or for any other reason, opportunities to improve protection and safety appear to be available and improvement seems desirable, any consequential modifications shall be made cautiously and only after favourable assessment of all the implications for protection and safety. The implementation of all improvements shall be prioritized so as to optimize protection and safety.

Requirement 14: Monitoring for verification of compliance

Registrants and licensees and employers shall conduct monitoring to verify compliance with the requirements for protection and safety.

3.37. The regulatory body shall establish requirements that monitoring and measurements be performed to verify compliance with the requirements for

protection and safety. The regulatory body shall be responsible for review and approval of the monitoring and measurement programmes of registrants and licensees.

3.38. Registrants and licensees and employers shall ensure that:

- (a) Monitoring and measurements of parameters are performed as necessary for verification of compliance with the requirements of these Standards;
- (b) Suitable equipment is provided and procedures for verification are implemented;
- (c) Equipment is properly maintained, tested and calibrated at appropriate intervals with reference to standards traceable to national or international standards;
- (d) Records are maintained of the results of monitoring and verification of compliance, as required by the regulatory body, including records of the tests and calibrations carried out in accordance with these Standards;
- (e) The results of monitoring and verification of compliance are shared with the regulatory body as required.

Requirement 15: Prevention and mitigation of accidents

Registrants and licensees shall apply good engineering practice and shall take all practicable measures to prevent accidents and to mitigate the consequences of those accidents that do occur.

Good engineering practice

3.39. The registrant or licensee, in cooperation with other responsible parties, shall ensure that the siting, location, design, manufacture, construction, assembly, commissioning, operation, maintenance and decommissioning (or closure) of facilities or parts thereof are based on good engineering practice which shall, as appropriate:

- (a) Take account of international and national standards;
- (b) Be supported by managerial and organizational features, with the purpose of ensuring protection and safety throughout the lifetime of the facility;
- (c) Include adequate safety margins in the design and construction of the facility, and in operations involving the facility, so as to ensure reliable performance in normal operation, and take account of the necessary quality, redundancy and capability for inspection, with emphasis on preventing

accidents, mitigating the consequences of those accidents that do occur and restricting any possible future exposures;

- (d) Take account of relevant developments concerning technical criteria, as well as the results of any relevant research on protection and safety and feedback of information on lessons learned from experience.

Defence in depth

3.40. Registrants and licensees shall ensure that a multilevel (defence in depth) system of sequential, independent provisions for protection and safety that is commensurate with the likelihood and magnitude of potential exposures is applied to sources for which the registrants and licensees are authorized. Registrants and licensees shall ensure that if one level of protection were to fail, the subsequent independent level of protection would be available. Such defence in depth shall be applied for the purposes of:

- (a) Preventing accidents;
- (b) Mitigating the consequences of any accidents that do occur;
- (c) Restoring the sources to safe conditions after any such accidents.

Accident prevention

3.41. Registrants and licensees shall ensure that structures, systems and components, including software, that are related to protection and safety for facilities and activities are designed, constructed, commissioned, operated and maintained so as to prevent accidents as far as reasonably practicable.

3.42. The registrant or licensee for any facility or activity shall make suitable arrangements:

- (a) To prevent reasonably foreseeable accidents in the facility or the activity;
- (b) To mitigate the consequences of those accidents that do occur;
- (c) To provide workers with the information, instruction, training and equipment necessary to restrict potential exposures;
- (d) To ensure that there are adequate procedures for the control of the facility and for the management of any reasonably foreseeable accidents;
- (e) To ensure that safety significant structures, systems and components, including software, and other equipment can be inspected and tested regularly for any degradation that could lead to abnormal conditions or inadequate performance;

- (f) To ensure that maintenance, inspection and testing appropriate to the preservation of the provisions for protection and safety can be carried out without undue occupational exposure;
- (g) To provide, wherever appropriate, automatic systems for safely shutting off or reducing the release of radiation from facilities in the event that operating conditions are outside the stipulated ranges;
- (h) To ensure that abnormal operating conditions that could significantly affect protection and safety are detected by systems that respond quickly enough to allow for corrective action to be taken in a timely manner;
- (i) To ensure that all relevant safety documentation is available in the appropriate languages understandable to users.

Emergency preparedness and response

3.43. If the safety assessment indicates that there is a reasonable likelihood of an emergency affecting either workers or members of the public, the registrant or licensee shall prepare an emergency plan for the protection of people and the environment. As part of this emergency plan, the registrant or licensee shall include arrangements for the prompt identification of an emergency, and for determining the appropriate level of the emergency response [15]. In relation to the arrangements for the emergency response at the scene by the registrant or licensee, the emergency plan shall include, in particular:

- (a) Provision for individual monitoring and area monitoring, and arrangements for medical treatment;
- (b) Arrangements for assessing and mitigating any consequences of an emergency.

3.44. Registrants and licensees shall be responsible for the implementation of their emergency plans and shall be prepared to take any necessary action for effective response. To prevent the occurrence of conditions that could lead to a loss of control over a source or to the escalation of such conditions, registrants and licensees shall, as appropriate:

- (a) Develop, maintain and implement procedures to provide the means for preventing loss of control over the source and for regaining control over the source as necessary;
- (b) Make available equipment, instrumentation and diagnostic aids that may be needed;
- (c) Train and periodically retrain personnel in the procedures to be followed and exercise the procedures.

Requirement 16: Investigations and feedback of information on operating experience

Registrants and licensees shall conduct formal investigations of abnormal conditions arising in the operation of facilities or the conduct of activities, and shall disseminate information that is significant for protection and safety.

3.45. Registrants and licensees shall ensure that information on both normal operation and abnormal conditions that are significant for protection and safety is disseminated or made available, as appropriate, to the regulatory body and relevant parties, as specified by the regulatory body. This information would include, for example, details of doses associated with given activities, data on maintenance, descriptions of events and information on corrective actions, and information on operating experience from other relevant facilities and activities.

3.46. Registrants and licensees shall conduct an investigation as specified by the regulatory body in the event that:

- (a) A quantity or operating parameter relating to protection and safety exceeds an investigation level or is outside the stipulated range of operating conditions; or
- (b) Any equipment failure, accident, error, mishap or other unusual event or condition occurs that has the potential for causing a quantity to exceed any relevant limit or operating restriction.

3.47. The registrant or licensee shall conduct an investigation as soon as possible after an event and shall prepare a written record of its causes, or suspected causes, including a verification or determination of any doses received or committed and recommendations for preventing the recurrence of the event and the occurrence of similar events.

3.48. The registrant or licensee shall communicate to the regulatory body and to any other relevant parties, as appropriate, a written report of any formal investigation relating to events as prescribed by the regulatory body, including exposures giving rise to doses exceeding a dose limit. The registrant or licensee shall also immediately report to the regulatory body any event in which a dose limit is exceeded.

Requirement 17: Radiation generators and radioactive sources

Registrants and licensees shall ensure the safety of radiation generators and radioactive sources.

3.49. Registrants and licensees who are manufacturers or other suppliers of radiation generators and radioactive sources shall ensure that the following responsibilities are discharged, as applicable:

- (a) Supplying a well designed, well manufactured and well constructed radiation generator or radioactive source and device in which the radiation generator or radioactive source is used that:
 - (i) Provides for protection and safety in accordance with the requirements of these Standards;
 - (ii) Meets engineering, performance and functional specifications;
 - (iii) Meets quality standards commensurate with the significance for protection and safety of systems and components, including software;
 - (iv) Provides clear displays, gauges and instructions on operating consoles in the appropriate language understandable to users.
- (b) Ensuring that radiation generators and radioactive sources are tested to demonstrate compliance with the relevant specifications.
- (c) Making information available, in the appropriate language understandable to users, on the proper installation and use of the radiation generator or radioactive source and on its associated radiation risks, including performance specifications, instructions for operating and maintenance, and instructions for protection and safety.
- (d) Ensuring that the protection provided by shielding and by other protective devices is optimized.

3.50. Where applicable, registrants and licensees shall make suitable arrangements with suppliers of radiation generators and radioactive sources, the regulatory body and relevant parties for the purposes of:

- (a) Obtaining information on conditions of use and operating experience that may be important for protection and safety;
- (b) Providing feedback and information that may have implications for protection and safety for other users, or that may have implications for the possibility for improvements in protection and safety for radiation generators and radioactive sources.

3.51. When choosing a location to use or to store a radiation generator or radioactive source, registrants and licensees shall take into account:

- (a) Factors that could affect the safe management of and control over the radiation generator or radioactive source;
- (b) Factors that could affect occupational exposure and public exposure due to the radiation generator or radioactive source;
- (c) The feasibility of taking the foregoing factors into account in engineering design.

3.52. In selecting a site for a facility that will contain a large amount of radioactive material and that will have the potential for the release of significant amounts of radioactive material, registrants and licensees shall take into account features that might affect protection and safety, features that might affect the integrity or functioning of the facility, and the feasibility of carrying out off-site protective actions if they become necessary.

3.53. Registrants and licensees shall maintain an inventory that includes records of:

- (a) The location and description of each radiation generator or radioactive source for which they are responsible;
- (b) The activity and form of each radioactive source for which they are responsible.

3.54. Registrants and licensees shall provide the regulatory body as required with appropriate information from their inventory records of radiation generators and radioactive sources.

3.55. Registrants and licensees shall keep radiation generators and radioactive sources under control so as to prevent loss or damage and to prevent any unauthorized person from carrying out any of the activities specified in para. 3.5, by ensuring that:

- (a) Control over a radiation generator or radioactive source is relinquished only in compliance with all relevant requirements specified in the registration or licence;
- (b) The regulatory body is promptly notified of information regarding a radiation generator or radioactive source that is lost, missing or not under control;

- (c) A radiation generator or radioactive source is transferred only if the recipient possesses the necessary authorization;
- (d) An inventory, as required in para. 3.53, of radiation generators or radioactive sources is checked periodically to confirm that they are in their assigned locations and are under control.

3.56. Registrants and licensees shall ensure that sealed sources are categorized in accordance with the categorization scheme set out in Schedule II, and in accordance with the requirements of the regulatory body.

3.57. The manufacturer of a radioactive source or a device containing a radioactive source shall ensure that, where practicable, the source itself and its container are marked with the symbol recommended by the International Organization for Standardization [16]³⁰.

3.58. Registrants and licensees, in cooperation with manufacturers, shall ensure that, where practicable, sealed sources are identifiable and traceable.

3.59. Registrants and licensees shall ensure that when radioactive sources are not in use they are stored in an appropriate manner for protection and safety.

3.60. Registrants and licensees shall ensure that arrangements are made promptly for the safe management of and control over radiation generators and radioactive sources, including appropriate financial provision, once it has been decided to take them out of use.

Requirement 18: Human imaging using radiation for purposes other than medical diagnosis, medical treatment or biomedical research

The government shall ensure that the use of ionizing radiation for human imaging for purposes other than medical diagnosis, medical treatment or biomedical research is subject to the system of protection and safety.

3.61. The government, if so decided in accordance with paras 3.18, 3.20 and 3.21, shall ensure that the requirements of para. 3.16 for the justification of practices

³⁰ For categories 1, 2 and 3 sealed sources as defined in Schedule II, the manufacturer may consider placement near the source, preferably on the shield or near the point of access to the source, of the supplementary symbol specified in Ref. [17]. The supplementary symbol is not placed on the external surfaces of transport packages, freight containers or conveyances or on building access doors.

are applied to any type of human imaging procedure in which radiation is used for purposes other than for medical diagnosis or medical treatment or other than as part of a programme of biomedical research. The justification process shall include the consideration of:

- (a) The benefits and detriments of implementing the type of human imaging procedure;
- (b) The benefits and detriments of not implementing the type of human imaging procedure;
- (c) Any legal or ethical issues associated with the introduction of the type of human imaging procedure;
- (d) The effectiveness and suitability of the type of human imaging procedure, including the appropriateness of the radiation equipment for the intended use;
- (e) The availability of sufficient resources to conduct the human imaging procedure safely throughout the intended period of the practice.

3.62. If it has been determined by means of the process specified in para. 3.61 that a particular practice of human imaging using radiation is justified, then such a practice shall be subject to regulatory control.

3.63. The regulatory body, in cooperation with other relevant authorities, agencies and professional bodies, as appropriate, shall establish the requirements for regulatory control of the practice and for review of the justification.

3.64. For human imaging using radiation, performed by medical personnel using medical radiological equipment, that exposes people to radiation for employment related, legal or health insurance³¹ purposes without reference to clinical indications:

- (a) The government shall ensure, on the basis of consultation between relevant authorities, professional bodies and the regulatory body, that dose constraints are established for such human imaging;

³¹ Such purposes for performing human imaging using radiation include: assessment of fitness for employment (prior to employment or periodically during employment); assessment of physiological suitability for a career or a sport; assessment of athletes before a selection or transfer; determination of age for legal purposes; obtaining evidence for legal purposes; detection of drugs concealed within the body; immigration or emigration requirements; pre-insurance checks; and obtaining evidence for the purposes of a compensation claim.

- (b) The registrant or licensee shall ensure that the appropriate optimization requirements for medical exposure in paras 3.162–3.177 are applied, with dose constraints as required in (a) above used instead of diagnostic reference levels.

3.65. Procedures with inspection imaging devices in which radiation is used to expose persons for the purpose of detection of concealed weapons, contraband or other objects on or within the body shall be considered to give rise to public exposure. Registrants and licensees shall apply the requirements for public exposure in planned exposure situations. In particular, registrants and licensees shall ensure that optimization of protection and safety is subject to any dose constraints for public exposure set by the government or the regulatory body.

3.66. Registrants and licensees shall ensure that all persons who are to undergo procedures with inspection imaging devices in which ionizing radiation is used are informed of the possibility of requesting the use of an alternative inspection technique that does not use ionizing radiation, where available.

3.67. The registrant or licensee shall ensure that any inspection imaging device used for the detection of concealed objects on or within the body, whether it is manufactured in or imported into the State in which it is used, conforms to applicable standards of the International Electrotechnical Commission or the International Organization for Standardization or to equivalent national standards.

OCCUPATIONAL EXPOSURE

Scope

3.68. The requirements in respect of occupational exposure in planned exposure situations (paras 3.69–3.116) apply to occupational exposure due to a practice or a source within a practice, as stated in paras 3.1–3.3; and to occupational exposure as required in Section 4 for emergency exposure situations and as required in Section 5 for existing exposure situations. For exposure due to natural sources, these requirements for occupational exposure in planned exposure situations apply, as appropriate, only to the exposure situations specified in para. 3.4(a), (c) and (d).

Requirement 19: Responsibilities of the regulatory body specific to occupational exposure

The government or the regulatory body shall establish and enforce requirements to ensure that protection and safety is optimized, and the regulatory body shall enforce compliance with dose limits for occupational exposure.

3.69. The government or the regulatory body shall establish the responsibilities of employers, registrants and licensees with regard to application of the requirements for occupational exposure in planned exposure situations.

3.70. The government or the regulatory body shall establish and enforce requirements to ensure that protection and safety is optimized for occupational exposure.

3.71. The government or the regulatory body shall establish, and the regulatory body shall enforce compliance with, the dose limits specified in Schedule III for occupational exposure.

3.72. Before authorization of a new or modified practice, the regulatory body shall require, as appropriate, and review supporting documents from the responsible parties that state:

- (a) Design criteria and design features relating to the exposure and potential exposure of workers in all operational states and in accident conditions;
- (b) Design criteria and design features of the appropriate systems and programmes for monitoring of workers for occupational exposure in all operational states and in accident conditions.

Requirement 20: Requirements for monitoring and recording of occupational exposures

The regulatory body shall establish and enforce requirements for the monitoring and recording of occupational exposures in planned exposure situations.

3.73. The regulatory body shall be responsible, as appropriate, for:

- (a) Establishment and enforcement of requirements for the monitoring, recording and control of occupational exposures in planned exposure situations in accordance with the requirements of these Standards;
- (b) Review of monitoring programmes of registrants and licensees, which shall be adequate to ensure that the requirements with regard to occupational exposure in planned exposure situations are fulfilled;
- (c) Authorization or approval of service providers for individual monitoring and calibration services;
- (d) Review of periodic reports on occupational exposure (including results of monitoring programmes and dose assessments) submitted by employers, registrants and licensees;
- (e) Provision for maintaining exposure records and results of the assessment of doses from occupational exposure;
- (f) Verification of compliance of an authorized practice with the requirements on the control of occupational exposure.

Requirement 21: Responsibilities of employers, registrants and licensees for the protection of workers

Employers, registrants and licensees shall be responsible for the protection of workers against occupational exposure. Employers, registrants and licensees shall ensure that protection and safety is optimized and that the dose limits for occupational exposure are not exceeded.

3.74. For workers who are engaged in activities in which they are or could be subject to occupational exposure in planned exposure situations, employers, registrants and licensees shall be responsible for:

- (a) Protection of workers against occupational exposure;
- (b) Compliance with other relevant requirements of these Standards.

3.75. Employers who are also registrants or licensees shall have the responsibilities of both employers and registrants or licensees.

3.76. Employers, registrants and licensees shall ensure, for all workers engaged in activities in which they are or could be subject to occupational exposure, that:

- (a) Occupational exposure is controlled so that the relevant dose limits for occupational exposure specified in Schedule III are not exceeded;
- (b) Protection and safety is optimized in accordance with the requirements of these Standards;

- (c) Decisions with regard to measures for protection and safety are recorded and made available to relevant parties, through their representatives where appropriate, as specified by the regulatory body;
- (d) Policies, procedures and organizational arrangements for protection and safety are established for implementing the relevant requirements of these Standards, with priority given to design measures and technical measures for controlling occupational exposure;
- (e) Suitable and adequate facilities, equipment and services for protection and safety are provided, the type and extent of which are commensurate with the expected likelihood and magnitude of occupational exposure;
- (f) Necessary workers' health surveillance and health services for workers are provided;
- (g) Appropriate monitoring equipment and personal protective equipment is provided and arrangements are made for its proper use, calibration, testing and maintenance;
- (h) Suitable and adequate human resources and appropriate training in protection and safety are provided, as well as periodic retraining as required to ensure the necessary level of competence;
- (i) Adequate records are maintained in accordance with the requirements of these Standards;
- (j) Arrangements are made to facilitate consultation of and cooperation with workers, through their representatives where appropriate, with regard to protection and safety on all measures necessary to achieve the effective application of these Standards;
- (k) Necessary conditions for promoting safety culture are provided.

3.77. Employers, registrants and licensees:

- (a) Shall involve workers, through their representatives where appropriate, in optimization of protection and safety;
- (b) Shall establish and use, as appropriate, constraints as part of optimization of protection and safety.

3.78. Employers, registrants and licensees shall ensure that workers exposed to radiation from sources within a practice that are not required by or directly related to their work have the same level of protection against such exposure as members of the public.

3.79. Employers, registrants and licensees shall take such administrative actions as are necessary to ensure that workers are informed that ensuring protection and safety is an integral part of a general occupational health and safety programme in

which they have specific obligations and responsibilities for their own protection and the protection of others against radiation exposure and for the safety of sources.

3.80. Employers, registrants and licensees shall record any report received from a worker that identifies circumstances that could affect compliance with the requirements of these Standards, and shall take appropriate action.

3.81. Nothing in these Standards shall be construed as relieving employers from complying with applicable national and local laws and regulations governing hazards in the workplace.

3.82. Employers, registrants and licensees shall facilitate compliance by workers with the requirements of these Standards.

Requirement 22: Compliance by workers

Workers shall fulfil their obligations and carry out their duties for protection and safety.

3.83. Workers:

- (a) Shall follow any applicable rules and procedures for protection and safety as specified by the employer, registrant or licensee;
- (b) Shall use properly the monitoring equipment and personal protective equipment provided;
- (c) Shall cooperate with the employer, registrant or licensee with regard to protection and safety, and programmes for workers' health surveillance and programmes for dose assessment;
- (d) Shall provide to the employer, registrant or licensee such information on their past and present work that is relevant for ensuring effective and comprehensive protection and safety for themselves and others;
- (e) Shall abstain from any wilful action that could put themselves or others in situations that would not be in accordance with the requirements of these Standards;
- (f) Shall accept such information, instruction and training in protection and safety as will enable them to conduct their work in accordance with the requirements of these Standards.

3.84. A worker who identifies circumstances that could adversely affect protection and safety shall report such circumstances to the employer, registrant or licensee as soon as possible.

Requirement 23: Cooperation between employers and registrants and licensees

Employers and registrants and licensees shall cooperate to the extent necessary for compliance by all responsible parties with the requirements for protection and safety.

3.85. If workers are engaged in work that involves or that could involve a source that is not under the control of their employer, the registrant or licensee responsible for the source and the employer shall cooperate to the extent necessary for compliance by both parties with the requirements of these Standards.

3.86. Cooperation between the employer and the registrant or licensee shall include, where appropriate:

- (a) The development and use of specific restrictions on exposure and other means of ensuring that the measures for protection and safety for workers who are engaged in work that involves or could involve a source that is not under the control of their employer are at least as good as those for employees of the registrant or licensee;
- (b) Specific assessments of the doses received by workers as specified in (a) above;
- (c) A clear allocation and documentation of the responsibilities of the employer and those of the registrant or licensee for protection and safety.

3.87. As part of the cooperation between parties, the registrant or licensee responsible for the source or for the exposure as appropriate:

- (a) Shall obtain from the employers, including self-employed persons, the previous occupational exposure history of workers as specified in para. 3.103, and any other necessary information;
- (b) Shall provide appropriate information to the employer, including any available information relevant for compliance with the requirements of these Standards that the employer requests;
- (c) Shall provide both the worker and the employer with the relevant exposure records.

Requirement 24: Arrangements under the radiation protection programme

Employers, registrants and licensees shall establish and maintain organizational, procedural and technical arrangements for the designation of controlled areas and supervised areas, for local rules and for monitoring of the workplace, in a radiation protection programme for occupational exposure.

Classification of areas: Controlled areas

3.88. Registrants and licensees shall designate as a controlled area any area³² in which specific measures for protection and safety are or could be required for:

- (a) Controlling exposures or preventing the spread of contamination in normal operation;
- (b) Preventing or limiting the likelihood and magnitude of exposures in anticipated operational occurrences and accident conditions.

3.89. In defining the boundaries of any controlled area, registrants and licensees shall take account of the magnitude of the exposures expected in normal operation, the likelihood and magnitude of exposures in anticipated operational occurrences and in accident conditions, and the type and extent of the procedures required for protection and safety.

3.90. Registrants and licensees:

- (a) Shall delineate controlled areas by physical means or, where this is not reasonably practicable, by some other suitable means.
- (b) Shall, where a source is only intermittently brought into operation or energized, or is moved from place to place, delineate an appropriate controlled area by means that are appropriate under the prevailing circumstances and shall specify exposure times.
- (c) Shall display the symbol recommended by the International Organization for Standardization [16] and shall display instructions at access points to and at appropriate locations within controlled areas.

³² The transport of radioactive material is regulated in accordance with the IAEA Transport Regulations [12].

- (d) Shall establish measures for protection and safety, including, as appropriate, physical measures to control the spread of contamination and local rules and procedures for controlled areas.
- (e) Shall restrict access to controlled areas by means of administrative procedures such as the use of work permits, and by physical barriers, which could include locks or interlocks, the degree of restriction being commensurate with the likelihood and magnitude of exposures.
- (f) Shall provide, as appropriate, at entrances to controlled areas:
 - (i) Personal protective equipment;
 - (ii) Equipment for individual monitoring and workplace monitoring;
 - (iii) Suitable storage for personal clothing.
- (g) Shall provide, as appropriate, at exits from controlled areas:
 - (i) Equipment for monitoring for contamination of skin and clothing;
 - (ii) Equipment for monitoring for contamination of any objects or material being removed from the area;
 - (iii) Washing or showering facilities and other personal decontamination facilities;
 - (iv) Suitable storage for contaminated personal protective equipment.
- (h) Shall periodically review conditions to assess whether there is any need to modify the measures for protection and safety or the boundaries of controlled areas;
- (i) Shall provide appropriate information, instruction and training for persons working in controlled areas.

Classification of areas: Supervised areas

3.91. Registrants and licensees shall designate as a supervised area any area not already designated as a controlled area but for which occupational exposure conditions need to be kept under review, even though specific measures for protection and safety are not normally needed.

3.92. Registrants and licensees, taking into account the nature, likelihood and magnitude of exposures or contamination in the supervised areas:

- (a) Shall delineate the supervised areas by appropriate means;
- (b) Shall display approved signs, as appropriate, at access points to supervised areas;
- (c) Shall periodically review conditions to assess whether there is any need for further measures for protection and safety or any need for changes to the boundaries of supervised areas.

Local rules and procedures and personal protective equipment

3.93. Employers, registrants and licensees shall minimize the need to rely on administrative controls and personal protective equipment for protection and safety by providing well engineered controls and satisfactory working conditions, in accordance with the following hierarchy of preventive measures:

- (1) Engineered controls;
- (2) Administrative controls;
- (3) Personal protective equipment.

3.94. Employers, registrants and licensees, in consultation with workers, or through their representatives where appropriate:

- (a) Shall establish in writing local rules and procedures that are necessary for protection and safety for workers and other persons;
- (b) Shall include in the local rules and procedures any relevant investigation level or authorized level, and the procedures to be followed in the event that any such level is exceeded;
- (c) Shall make the local rules and procedures and the measures for protection and safety known to those workers to whom they apply and to other persons who may be affected by them;
- (d) Shall ensure that any work in which workers are or could be subject to occupational exposure is adequately supervised and shall take all reasonable steps to ensure that the rules, procedures, and measures for protection and safety are observed;
- (e) Shall designate, as appropriate, a radiation protection officer in accordance with criteria established by the regulatory body.

3.95. Employers, registrants and licensees shall ensure that:

- (a) Workers are provided with suitable and adequate personal protective equipment that meets relevant standards or specifications, including as appropriate:
 - (i) Protective clothing;
 - (ii) Respiratory protective equipment the characteristics of which are made known to the users;
 - (iii) Protective aprons, protective gloves and organ shields.
- (b) Where appropriate, workers receive adequate instruction in the proper use of respiratory protective equipment, including testing for good fit.

- (c) Tasks requiring the use of certain personal protective equipment are assigned only to workers who on the basis of medical advice are capable of safely sustaining the extra effort necessary.
- (d) All personal protective equipment, including equipment for use in an emergency, is maintained in proper condition and, if appropriate, is tested at regular intervals.
- (e) If the use of personal protective equipment is considered for any given task, account is taken of any additional exposure that could result owing to the additional time taken or the inconvenience, and of any non-radiological risks that might be associated with using personal protective equipment while performing the task.

Monitoring of the workplace

3.96. Registrants and licensees, in cooperation with employers where appropriate, shall establish, maintain and keep under review a programme for workplace monitoring under the supervision of a radiation protection officer or qualified expert.

3.97. The type and frequency of workplace monitoring:

- (a) Shall be sufficient to enable:
 - (i) Evaluation of the radiological conditions in all workplaces;
 - (ii) Assessment of exposures in controlled areas and supervised areas;
 - (iii) Review of the classification of controlled areas and supervised areas.
- (b) Shall be based on dose rate, activity concentration in air and surface contamination, and their expected fluctuations, and on the likelihood and magnitude of exposures in anticipated operational occurrences and accident conditions.

3.98. Registrants and licensees, in cooperation with employers where appropriate, shall maintain records of the findings of the workplace monitoring programme. The findings of the workplace monitoring programme shall be made available to workers, through their representatives where appropriate.

Requirement 25: Assessment of occupational exposure and workers' health surveillance

Employers, registrants and licensees shall be responsible for making arrangements for assessment and recording of occupational exposures and for workers' health surveillance.

Assessment of occupational exposure

3.99. Employers, as well as self-employed persons, and registrants and licensees shall be responsible for making arrangements for assessment of the occupational exposure of workers, on the basis of individual monitoring where appropriate, and shall ensure that arrangements are made with authorized or approved dosimetry service providers that operate under a quality management system.

3.100. For any worker who usually works in a controlled area, or who occasionally works in a controlled area and may receive a significant dose from occupational exposure, individual monitoring shall be undertaken where appropriate, adequate and feasible. In cases where individual monitoring of the worker is inappropriate, inadequate or not feasible, the occupational exposure shall be assessed on the basis of the results of workplace monitoring and information on the locations and durations of exposure of the worker³³.

3.101. For any worker who regularly works in a supervised area or who enters a controlled area only occasionally, the occupational exposure shall be assessed on the basis of the results of workplace monitoring or individual monitoring, as appropriate.

3.102. Employers shall ensure that workers who could be subject to exposure due to contamination are identified, including workers who use respiratory protective equipment. Employers shall arrange for appropriate monitoring to the extent necessary to demonstrate the effectiveness of the measures for protection and safety and to assess intakes of radionuclides and the committed effective doses.

Records of occupational exposure

3.103. Employers, registrants and licensees shall maintain records of occupational exposure³⁴ for every worker for whom assessment of occupational exposure is required in paras 3.99–3.102.

³³ The distinction between types of worker in paras 3.100 and 3.101 for the purposes of monitoring has similarities to the distinction between category A and category B workers in European Union legislation [18].

³⁴ Records of occupational exposure are also referred to as ‘exposure records’ or ‘dose records’.

3.104. Records of occupational exposure for each worker shall be maintained during and after the worker's working life, at least until the former worker attains or would have attained the age of 75 years, and for not less than 30 years after cessation of the work in which the worker was subject to occupational exposure.

3.105. Records of occupational exposure shall include:

- (a) Information on the general nature of the work in which the worker was subject to occupational exposure;
- (b) Information on dose assessments, exposures and intakes at or above the relevant recording levels specified by the regulatory body and the data upon which the dose assessments were based;
- (c) When a worker is or has been exposed while in the employ of more than one employer, information on the dates of employment with each employer and on the doses, exposures and intakes in each such employment;
- (d) Records of any assessments made of doses, exposures and intakes due to actions taken in an emergency or due to accidents or other incidents, which shall be distinguished from assessments of doses, exposures and intakes due to normal conditions of work and which shall include references to reports of any relevant investigations.

3.106. Employers, registrants and licensees:

- (a) Shall provide workers with access to records of their own occupational exposure;
- (b) Shall provide the supervisor of the programme for workers' health surveillance, the regulatory body and the relevant employer with access to workers' records of occupational exposure;
- (c) Shall facilitate the provision of copies of workers' exposure records to new employers when workers change employment;
- (d) Shall make arrangements for the retention of exposure records for former workers by the employer, registrant or licensee, as appropriate;
- (e) Shall, in complying with (a)–(d) above, give due care and attention to maintaining the confidentiality of records.

3.107. If employers, registrants and licensees cease to conduct activities in which workers are subject to occupational exposure, they shall make arrangements for the retention of workers' records of occupational exposure by the regulatory body or a State registry, or by a relevant employer, registrant or licensee, as appropriate.

Workers' health surveillance

3.108. Programmes for workers' health surveillance as required in para. 3.76(f):

- (a) Shall be based on the general principles of occupational health [19];
- (b) Shall be designed to assess the initial fitness and continuing fitness of workers for their intended tasks.

3.109. If one or more workers are to be engaged in work in which they are or could be exposed to radiation from a source that is not under the control of their employer, the registrant or licensee responsible for the source shall, as a precondition for the engagement of such workers, make with the employer any special arrangements for workers' health surveillance that are needed to comply with the rules established by the regulatory body or other relevant authority.

Requirement 26: Information, instruction and training

Employers, registrants and licensees shall provide workers with adequate information, instruction and training for protection and safety.

3.110. Employers, in cooperation with registrants and licensees:

- (a) Shall provide all workers with adequate information on health risks due to their occupational exposure in normal operation, anticipated operational occurrences and accident conditions, adequate instruction and training and periodic retraining in protection and safety, and adequate information on the significance of their actions for protection and safety;
- (b) Shall provide those workers who could be involved in or affected by the response to an emergency with appropriate information, and adequate instruction and training and periodic retraining, for protection and safety;
- (c) Shall maintain records of the training provided to individual workers.

Requirement 27: Conditions of service

Employers, registrants and licensees shall not offer benefits as substitutes for measures for protection and safety.

3.111. The conditions of service of workers shall be independent of whether they are or could be subject to occupational exposure. Special compensatory arrangements, or preferential consideration with respect to salary, special insurance coverage, working hours, length of vacation, additional holidays

or retirement benefits, shall neither be granted nor be used as substitutes for measures for protection and safety in accordance with the requirements of these Standards.

3.112. Employers shall make all reasonable efforts to provide workers with suitable alternative employment in circumstances for which it has been determined, either by the regulatory body or in the framework of the programme for workers' health surveillance in accordance with the requirements of these Standards, that workers, for health reasons, may no longer continue in employment in which they are or could be subject to occupational exposure.

Requirement 28: Special arrangements for protection and safety for female workers and for persons under 18 years of age undergoing training

Employers, registrants and licensees shall make special arrangements for female workers, as necessary, for protection of the embryo or fetus and breastfed infants. Employers, registrants and licensees shall make special arrangements for protection and safety for persons under 18 years of age who are undergoing training.

3.113. Employers, in cooperation with registrants and licensees, shall provide female workers who are liable to enter controlled areas or supervised areas or who may undertake emergency duties with appropriate information on:

- (a) The risk to the embryo or fetus due to exposure of a pregnant woman;
- (b) The importance for a female worker of notifying her employer as soon as possible if she suspects that she is pregnant³⁵ or if she is breast-feeding;
- (c) The risk of health effects for a breastfed infant due to ingestion of radioactive substances.

3.114. Notification of the employer by a female worker if she suspects that she is pregnant or if she is breast-feeding shall not be considered a reason to exclude the female worker from work. The employer of a female worker, who has been notified of her suspected pregnancy or that she is breast-feeding, shall adapt the working conditions in respect of occupational exposure so as to ensure

³⁵ Notification of an employer of a suspected pregnancy or of breast-feeding cannot be made a requirement on a female worker in these Standards. However, it is necessary that all female workers understand the importance of making such notifications so that their working conditions may be modified accordingly.

that the embryo or fetus or the breastfed infant is afforded the same broad level of protection as is required for members of the public.

3.115. Employers, registrants and licensees shall ensure that no person under the age of 16 years is or could be subject to occupational exposure.

3.116. Employers, registrants and licensees shall ensure that persons under the age of 18 years are allowed access to a controlled area only under supervision and only for the purpose of training for employment in which they are or could be subject to occupational exposure or for the purpose of studies in which sources are used.

PUBLIC EXPOSURE

Scope

3.117. The requirements in respect of public exposure in planned exposure situations (paras 3.118–3.144) apply to public exposure due to a practice or a source within a practice, as referred to in paras 3.1–3.3. For exposure due to natural sources, such requirements apply only to the types of public exposure specified in para. 3.4(a) and (b).

Requirement 29: Responsibilities of the government and the regulatory body specific to public exposure

The government or the regulatory body shall establish the responsibilities of relevant parties that are specific to public exposure, shall establish and enforce requirements for optimization, and shall establish, and the regulatory body shall enforce compliance with, dose limits for public exposure.

3.118. The government or the regulatory body shall establish the responsibilities of registrants and licensees, of suppliers, and of providers of consumer products³⁶ in relation to the application of requirements for public exposure in planned exposure situations.

³⁶ The term ‘providers of consumer products’ includes the designers, manufacturers, producers, constructors, installers, distributors, sellers, and importers and exporters of consumer products.

3.119. The government or the regulatory body shall establish and enforce requirements for the optimization of protection and safety for situations in which individuals are or could be subject to public exposure.

3.120. The government or the regulatory body shall establish or approve constraints on dose and constraints on risk to be used in the optimization of protection and safety for members of the public. When establishing or approving constraints in respect of a source within a practice, the government or the regulatory body shall take into account, as appropriate:

- (a) The characteristics of the source and of the practice that are of relevance for public exposure;
- (b) Good practice in the operation of similar sources;
- (c) Dose contributions from other authorized practices or from possible future authorized practices³⁷, estimated at the design and planning stage, so that the total dose to members of the public is not expected to exceed the dose limit at any time after the start of operation of the source;
- (d) The views of interested parties.

3.121. The government or the regulatory body shall establish, and the regulatory body shall enforce compliance with, the dose limits specified in Schedule III for public exposure.

3.122. Before authorization of a new or modified practice, the regulatory body shall require the submission of, and shall review, the safety assessments (paras 3.29–3.36) and other design related documents from the responsible parties that address the optimization of protection and safety, the design criteria and the design features relating to the assessment of exposure and potential exposure of members of the public.

3.123. The regulatory body shall establish or approve operational limits and conditions relating to public exposure, including authorized limits for discharges. These operational limits and conditions:

- (a) Shall be used by registrants and licensees as the criteria for demonstration of compliance after the commencement of operation of a source;

³⁷ Dose contributions from possible future authorized practices have to be anticipated in an assessment made on the basis of realistic assumptions.

- (b) Shall correspond to doses below the dose limits with account taken of the results of optimization of protection and safety;
- (c) Shall reflect good practice in the operation of similar facilities or activities;
- (d) Shall allow for operational flexibility;
- (e) Shall take into account the results of the prospective assessment for radiological environmental impacts that is undertaken in accordance with requirements of the regulatory body (see paras 3.9(e) and 3.15(d)).

3.124. When a source within a practice could cause public exposure outside the territory or other area under the jurisdiction or control of the State in which the source is located, the government or the regulatory body:

- (a) Shall ensure that the assessment for radiological impacts includes those impacts outside the territory or other area under the jurisdiction or control of the State;
- (b) Shall, to the extent possible, establish requirements for the control of discharges;
- (c) Shall arrange with the affected State the means for the exchange of information and consultations, as appropriate.

Requirement 30: Responsibilities of relevant parties specific to public exposure

Relevant parties shall apply the system of protection and safety to protect members of the public against exposure.

General considerations

3.125. Registrants and licensees, in cooperation with suppliers and with providers of consumer products, shall apply the requirements of these Standards and shall verify and demonstrate compliance with them, as specified by the regulatory body, in relation to any public exposure delivered by a source for which they have responsibility.

3.126. Registrants and licensees, in cooperation with suppliers, in applying the principle of optimization of protection and safety in the design, planning, operation and decommissioning of a source (or for closure and the post-closure period for waste disposal facilities), shall take into account:

- (a) Possible changes in any conditions that could affect exposure of members of the public, such as changes in the characteristics and use of the source,

changes in environmental dispersion conditions, changes in exposure pathways or changes in values of parameters used for the determination of the representative person;

- (b) Good practice in the operation of similar sources or the conduct of similar practices;
- (c) Possible buildup and accumulation in the environment of radioactive substances from discharges during the lifetime of the source;
- (d) Uncertainties in the assessment of doses, especially uncertainties in contributions to doses if the source and the representative person are separated in space or in time.

3.127. Registrants and licensees, for sources under their responsibility, shall establish, implement and maintain:

- (a) Policies, procedures and organizational arrangements for protection and safety in relation to public exposure, in accordance with the requirements of these Standards.
- (b) Measures for ensuring:
 - (i) Optimization of protection and safety;
 - (ii) Limitation of exposure of members of the public from such sources, in accordance with the authorization.
- (c) Measures for ensuring the safety of such sources.
- (d) Provision for suitable and adequate resources (including facilities, equipment and services) for the protection and safety of members of the public, commensurate with the likelihood and magnitude of exposures.
- (e) Programmes for appropriate training of personnel having functions relevant to protection and safety of members of the public, as well as periodic retraining as required, to ensure the necessary level of competence.
- (f) Provision for appropriate monitoring equipment, monitoring programmes and methods for assessing public exposure.
- (g) Adequate records of monitoring programmes.
- (h) Emergency plans, emergency procedures and emergency arrangements, in accordance with the nature and magnitude of the radiation risks associated with the sources.

Visitors

3.128. Registrants and licensees, in cooperation with employers where appropriate:

- (a) Shall apply the relevant requirements of these Standards in respect of public exposure for visitors to a controlled area or a supervised area;
- (b) Shall ensure that visitors are accompanied in any controlled area by a person who knows the measures for protection and safety for the controlled area;
- (c) Shall provide adequate information and instructions to visitors before they enter a controlled area or a supervised area, so as to provide for protection and safety for visitors and for other individuals who could be affected by their actions;
- (d) Shall ensure that adequate control is maintained over the entry of visitors to a controlled area or a supervised area, including the use of signs for such areas.

External exposure and contamination in areas accessible to members of the public

3.129. Registrants and licensees shall ensure that if a source can give rise to external exposure of members of the public:

- (a) The floor plans and arrangements of equipment for all new installations utilizing such sources, as well as all significant modifications to existing installations, are subject, as appropriate, to review and approval by the regulatory body prior to commissioning;
- (b) Shielding and other measures for protection and safety, including access control, are provided as appropriate for restricting public exposure, in particular at open sites such as for some applications of industrial radiography.

3.130. Registrants and licensees shall ensure, as appropriate, that:

- (a) Specific provisions for confinement are established for the design and operation of a source that could cause the spread of contamination in areas that are accessible to members of the public;
- (b) Measures for protection and safety are implemented for restricting public exposure due to contamination in areas within a facility that are accessible to members of the public.

Requirement 31: Radioactive waste and discharges

Relevant parties shall ensure that radioactive waste and discharges of radioactive material to the environment are managed in accordance with the authorization.

Radioactive waste

3.131. Registrants and licensees, in cooperation with suppliers, as appropriate:

- (a) Shall ensure that any radioactive waste generated is kept to the minimum practicable in terms of both activity and volume;
- (b) Shall ensure that radioactive waste is managed in accordance with the requirements of these Standards and the requirements of other applicable IAEA standards, and in accordance with the relevant authorization;
- (c) Shall ensure that there is separate processing of radioactive waste of different types, where warranted by differences in factors such as radionuclide content, half-life, activity concentration, volume, and physical and chemical properties, taking into account the available options for storage and disposal of radioactive waste, without precluding the mixing of radioactive waste for purposes of protection and safety;
- (d) Shall ensure that activities for the predisposal management of and for the disposal of radioactive waste are conducted in accordance with the requirements of applicable IAEA standards³⁸, and in accordance with the authorization;
- (e) Shall maintain an inventory of all radioactive waste that is generated, stored, transferred or disposed of;
- (f) Shall develop and implement a strategy for radioactive waste management and shall include appropriate evidence that protection and safety is optimized.

Discharges

3.132. Registrants and licensees, in cooperation with suppliers, in applying for an authorization for discharges, as appropriate:

³⁸ Requirements for the predisposal management of radioactive waste are established in Ref. [10] and requirements for the disposal of radioactive waste are established in Ref. [11].

- (a) Shall determine the characteristics and activity of the material to be discharged, and the possible points and methods of discharge;
- (b) Shall determine by an appropriate pre-operational study all significant exposure pathways by which discharged radionuclides could give rise to exposure of members of the public;
- (c) Shall assess the doses to the representative person due to the planned discharges;
- (d) Shall consider the radiological environmental impacts in an integrated manner with features of the system of protection and safety, as required by the regulatory body;
- (e) Shall submit to the regulatory body the findings of (a)–(d) above as an input to the establishment by the regulatory body, in accordance with para. 3.123, of authorized limits on discharges and conditions for their implementation.

3.133. Registrants and licensees shall ensure that operational limits and conditions relating to public exposure are met in accordance with paras 3.123 and 3.124.

3.134. Registrants and licensees shall review and modify their discharge control measures, as appropriate and in agreement with the regulatory body, taking into account:

- (a) Operating experience;
- (b) Any changes in exposure pathways or in the characteristics of the representative person that could affect the assessment of doses due to the discharges.

Requirement 32: Monitoring and reporting

The regulatory body and relevant parties shall ensure that programmes for source monitoring and environmental monitoring are in place and that the results from the monitoring are recorded and are made available.

3.135. The regulatory body shall be responsible, as appropriate, for:

- (a) Review and approval of monitoring programmes of registrants and licensees, which shall be sufficient for:
 - (i) Verifying compliance with the requirements of these Standards in respect of public exposure in planned exposure situations;
 - (ii) Assessing doses from public exposure.

- (b) Review of periodic reports on public exposure (including results of monitoring programmes and dose assessments) submitted by registrants and licensees.
- (c) Making provision for an independent monitoring programme.
- (d) Assessment of the total public exposure due to authorized sources and practices in the State on the basis of monitoring data provided by registrants and licensees and with the use of data from independent monitoring and assessments.
- (e) Making provision for maintaining records of discharges, results of monitoring programmes and results of assessments of public exposure.
- (f) Verification of compliance of an authorized practice with the requirements of these Standards for the control of public exposure.

3.136. The regulatory body shall publish or shall make available on request, as appropriate, results from source monitoring and environmental monitoring programmes and assessments of doses from public exposure.

3.137. Registrants and licensees shall, as appropriate:

- (a) Establish and implement monitoring programmes to ensure that public exposure due to sources under their responsibility is adequately assessed and that the assessment is sufficient to verify and demonstrate compliance with the authorization. These programmes shall include monitoring of the following, as appropriate:
 - (i) External exposure due to such sources;
 - (ii) Discharges;
 - (iii) Radioactivity in the environment;
 - (iv) Other parameters important for the assessment of public exposure.
- (b) Maintain appropriate records of the results of the monitoring programmes and estimated doses to members of the public.
- (c) Report or make available to the regulatory body the results of the monitoring programme at approved intervals, including, as applicable, the levels and composition of discharges, dose rates at the site boundary and in premises open to members of the public, results of environmental monitoring and retrospective assessments of doses to the representative person.
- (d) Report promptly to the regulatory body any levels exceeding the operational limits and conditions relating to public exposure, including authorized limits on discharges, in accordance with reporting criteria established by the regulatory body.
- (e) Report promptly to the regulatory body any significant increase in dose rate or concentrations of radionuclides in the environment that could be

attributed to the authorized practice, in accordance with reporting criteria established by the regulatory body.

- (f) Establish and maintain a capability to conduct monitoring in an emergency in the event of unexpected increases in radiation levels or in concentrations of radionuclides in the environment due to an accident or other unusual event attributed to the authorized source or facility.
- (g) Verify the adequacy of the assumptions made for the assessment of public exposure and the assessment for radiological environmental impacts.
- (h) Publish or make available on request, as appropriate, results from source monitoring and environmental monitoring programmes and assessments of doses from public exposure.

Requirement 33: Consumer products

Providers of consumer products shall ensure that consumer products are not made available to the public unless their use by members of the public has been justified, and either their use has been exempted or their provision to the public has been authorized.

3.138. Providers of consumer products shall ensure that consumer products are not made available to the public unless the justification of their use by members of the public has been approved by the government or the regulatory body, and either their use has been exempted on the basis of the criteria specified in Schedule I or their provision to the public has been authorized.

3.139. Upon receipt of a request for authorization to provide consumer products to the public, the regulatory body:

- (a) Shall require the provider of the consumer product to provide documents to demonstrate compliance with the requirements in paras 3.138–3.144;
- (b) Shall verify the assessments and the selection of parameters presented in the request for authorization;
- (c) Shall determine whether the end use of the consumer product can be exempted;
- (d) Shall authorize the provision to the public of the consumer product, where appropriate, subject to specific conditions of authorization.

3.140. Providers of consumer products:

- (a) Shall comply with the conditions of the authorization to provide consumer products to the public;

- (b) Shall ensure that consumer products comply with the requirements of these Standards;
- (c) Shall plan for appropriate arrangements for the servicing, maintenance, recycling or disposal of consumer products.

3.141. The design and manufacture of consumer products, with regard to features that could affect exposure during normal handling, transport and use, as well as in the event of mishandling, misuse, accident or disposal, shall be subject to the optimization of protection and safety. In this regard, designers, manufacturers and other providers of consumer products shall take into account the following:

- (a) The various radionuclides that could be used in consumer products and their radiation types, energies, activities and half-lives;
- (b) The chemical and physical forms of the radionuclides that could be used in consumer products and their significance for protection and safety in normal conditions and abnormal conditions;
- (c) The containment and shielding of radioactive substances in consumer products and access to these radioactive substances in normal conditions and abnormal conditions;
- (d) The need for servicing or repair of consumer products and ways in which this could be done;
- (e) Relevant experience with similar consumer products.

3.142. Providers of consumer products shall ensure that:

- (a) Where practicable, a legible label is firmly affixed to a visible surface of each consumer product that:
 - (i) States that the consumer product contains radioactive substances and identifies the radionuclides and their activities;
 - (ii) States that the provision of the consumer product to the public has been authorized by the regulatory body;
 - (iii) Provides information on required or recommended options for recycling or disposal.
- (b) The information specified in (a) above is also printed legibly on the retail packaging of the consumer product.

3.143. Providers of consumer products shall provide clear and appropriate information and instructions with each consumer product on:

- (a) Correct installation, use and maintenance of the consumer product;
- (b) Servicing and repair;
- (c) The radionuclides and their activities at a specified date;
- (d) Dose rates in normal operation and during servicing and repair;
- (e) Required or recommended options for recycling or disposal.

3.144. Providers of consumer products shall provide the consumer product retailers with appropriate information on safety and instructions on their transport and storage.

MEDICAL EXPOSURE

Scope

3.145. The requirements in respect of medical exposure in planned exposure situations (paras 3.146–3.185) apply to all medical exposures³⁹, including intended, unintended and accidental exposures.

3.146. Dose limits do not apply to medical exposures.

Requirement 34: Responsibilities of the government specific to medical exposure

The government shall ensure that relevant parties are authorized to assume their roles and responsibilities, and that diagnostic reference levels, dose constraints, and criteria and guidelines for the release of patients are established.

3.147. The government, in accordance with paras 2.13–2.28, shall ensure with regard to medical exposures that, as a result of consultation between the health authority, relevant professional bodies and the regulatory body, the relevant parties identified in paras 2.40 and 2.41 are authorized to assume their roles and responsibilities, and shall ensure that they are notified of their duties in relation to protection and safety for individuals undergoing medical exposures.

³⁹ Requirements on human imaging using radiation for purposes other than medical diagnosis, medical treatment or biomedical research (and, hence, not within the scope of medical exposure) are stated in paras 3.61–3.67.

3.148. The government shall ensure, as part of the responsibilities specified in para. 2.15, that as a result of consultation between the health authority, relevant professional bodies and the regulatory body, a set of diagnostic reference levels is established for medical exposures incurred in medical imaging, including image guided interventional procedures. In setting such diagnostic reference levels, account shall be taken of the need for adequate image quality, to enable the requirements of para. 3.169 to be fulfilled. Such diagnostic reference levels shall be based, as far as possible, on wide scale surveys or on published values that are appropriate for the local circumstances.

3.149. The government shall ensure that, as a result of consultation between the health authority, relevant professional bodies and the regulatory body, the following are established:

- (a) Dose constraints, to enable the requirements of paras 3.173 and 3.174, respectively, to be fulfilled for:
 - (i) Exposures of carers and comforters⁴⁰;
 - (ii) Exposures due to diagnostic investigations of volunteers participating in a programme of biomedical research.
- (b) Criteria and guidelines for the release of patients who have undergone therapeutic radiological procedures using unsealed sources or patients who still retain implanted sealed sources.

Requirement 35: Responsibilities of the regulatory body specific to medical exposure

The regulatory body shall require that health professionals with responsibilities for medical exposure are specialized in the appropriate area and that they fulfil the requirements for education, training and competence in the relevant specialty.

3.150. The regulatory body shall ensure that the authorization for medical exposures to be performed at a particular medical radiation facility allows personnel (radiological medical practitioners, medical physicists, medical radiation technologists and any other health professionals with specific duties in relation to the radiation protection of patients) to assume the responsibilities specified in these Standards only if they:

⁴⁰ The selection of constraints for carers and comforters is a complex process in which a number of factors have to be taken into account, such as the age of the individual and for a woman the possibility of her being pregnant.

- (a) Are specialized⁴¹ in the appropriate area⁴²;
- (b) Meet the respective requirements for education, training and competence in radiation protection, in accordance with para. 2.32;
- (c) Are named in a list maintained up to date by the registrant or licensee.

Requirement 36: Responsibilities of registrants and licensees specific to medical exposure

Registrants and licensees shall ensure that no person incurs a medical exposure unless there has been an appropriate referral, responsibility has been assumed for ensuring protection and safety, and the person subject to exposure has been informed as appropriate of the expected benefits and risks.

3.151. Registrants and licensees shall ensure that no patient, whether symptomatic or asymptomatic, undergoes a medical exposure unless:

- (a) It is a radiological procedure that has been requested by a referring medical practitioner and information on the clinical context has been provided, or it is part of an approved health screening programme;
- (b) The medical exposure has been justified by means of consultation between the radiological medical practitioner and the referring medical practitioner, as appropriate, or it is part of an approved health screening programme;
- (c) A radiological medical practitioner has assumed responsibility for protection and safety in the planning and delivery of the medical exposure as specified in para. 3.154(a);
- (d) The patient or the patient's legal authorized representative has been informed as appropriate of the expected diagnostic or therapeutic benefits of the radiological procedure as well as the radiation risks.

3.152. Registrants and licensees shall ensure that no individual incurs a medical exposure as part of a programme of biomedical research unless the exposure

⁴¹ 'Specialized' means specialized as acknowledged by the relevant professional body, health authority or appropriate organization.

⁴² 'The appropriate area' means, in the first instance, diagnostic radiology, image guided interventional procedures, or radiation therapy or nuclear medicine (diagnostic radiological procedures, therapeutic radiological procedures or both). The area of specialization is often likely to be narrower, however, in particular with regard to the radiological medical practitioner. Examples are dental, chiropractic or podiatric specialists in the case of diagnostic radiology, and cardiologists, urologists or neurologists in the case of image guided interventional procedures.

has been approved by an ethics committee (or other institutional body that has been assigned functions similar to those of an ethics committee by the relevant authority) as required in para. 3.161 and a radiological medical practitioner has assumed responsibility as specified in para. 3.154(a). Registrants and licensees shall ensure that the requirements specified in para. 3.174 are fulfilled for the optimization of protection and safety for persons subject to exposure as part of a programme of biomedical research.

3.153. Registrants and licensees shall ensure that no individual incurs a medical exposure as a carer or comforter unless he or she has received, and has indicated an understanding of, relevant information on radiation protection and information on the radiation risks prior to providing care and comfort to an individual undergoing a radiological procedure. Registrants and licensees shall ensure that the requirements specified in para. 3.173 are fulfilled for the optimization of protection and safety for any radiological procedure in which an individual acts as a carer or comforter.

3.154. Registrants and licensees shall ensure that:

- (a) The radiological medical practitioner performing or overseeing the radiological procedure has assumed responsibility for ensuring overall protection and safety for patients in the planning and delivery of the medical exposure, including the justification of the radiological procedure as required in paras 3.155–3.161 and the optimization of protection and safety, in cooperation with the medical physicist and the medical radiation technologist as required in paras 3.162–3.177;
- (b) Radiological medical practitioners, medical physicists, medical radiation technologists and other health professionals with specific duties in relation to protection and safety for patients in a given radiological procedure are specialized in the appropriate area;
- (c) Sufficient medical personnel and paramedical personnel are available as specified by the health authority;
- (d) For therapeutic radiological procedures, the requirements of these Standards for calibration, dosimetry and quality assurance, including the acceptance and commissioning of medical radiological equipment, as specified in paras 3.167, 3.168(c), 3.170 and 3.171, are fulfilled by or under the supervision of a medical physicist;
- (e) For diagnostic radiological procedures and image guided interventional procedures, the requirements of these Standards for medical imaging, calibration, dosimetry and quality assurance, including the acceptance and commissioning of medical radiological equipment, as specified in

paras 3.167, 3.168(a) and (b), 3.169, 3.170 and 3.171, are fulfilled by or under the oversight of or with the documented advice of a medical physicist, whose degree of involvement is determined by the complexity of the radiological procedures and the associated radiation risks;

- (f) Any delegation of responsibilities by a principal party is documented.

Requirement 37: Justification of medical exposures

Relevant parties shall ensure that medical exposures are justified.

3.155. Medical exposures shall be justified by weighing the diagnostic or therapeutic benefits⁴³ that they are expected to yield against the radiation detriment that they might cause, with account taken of the benefits and the risks of available alternative techniques that do not involve medical exposure.

3.156. Generic justification of a radiological procedure shall be carried out by the health authority in conjunction with appropriate professional bodies, and shall be reviewed from time to time, with account taken of advances in knowledge and technological developments.

3.157. The justification of medical exposure for an individual patient shall be carried out by means of consultation between the radiological medical practitioner and the referring medical practitioner, as appropriate, with account taken, in particular for patients who are pregnant or breast-feeding or are paediatric, of:

- (a) The appropriateness of the request;
- (b) The urgency of the radiological procedure;
- (c) The characteristics of the medical exposure;
- (d) The characteristics of the individual patient;
- (e) Relevant information from the patient's previous radiological procedures.

3.158. Relevant national or international referral guidelines shall be taken into account for the justification of the medical exposure of an individual patient in a radiological procedure.

⁴³ The diagnostic or therapeutic benefit that medical exposures are expected to yield may not necessarily be to the person exposed. For patients, this is clearly the case, but for exposures in biomedical research the benefit is expected to be for biomedical sciences and for future health care. Similarly, the benefit for carers and comforters might be, for example, the successful performance of a diagnostic procedure on a child.

3.159. Justification for radiological procedures to be performed as part of a health screening programme for asymptomatic populations shall be carried out by the health authority in conjunction with appropriate professional bodies.

3.160. Any radiological procedure on an asymptomatic individual that is intended to be performed for the early detection of disease, but not as part of an approved health screening programme, shall require specific justification for that individual by the radiological medical practitioner and the referring medical practitioner, in accordance with the guidelines of relevant professional bodies or the health authority. As part of this process, the individual shall be informed in advance of the expected benefits, risks and limitations of the radiological procedure.

3.161. The medical exposure of volunteers as part of a programme of biomedical research is deemed to be not justified unless:

- (a) It is in accordance with the provisions of the Helsinki Declaration [20] and takes into account the guidelines published by the Council for International Organizations of Medical Sciences [21], together with the recommendations of the ICRP [22];
- (b) It is subject to approval by an ethics committee (or other institutional body that has been assigned functions similar to those of an ethics committee by the relevant authority), subject to any dose constraints that may be specified (as required in paras 3.149(a)(ii) and 3.174), and subject to applicable national regulations and local regulations.

Requirement 38: Optimization of protection and safety

Registrants and licensees and radiological medical practitioners shall ensure that protection and safety is optimized for each medical exposure.

Design considerations

3.162. In addition to ensuring that the responsibilities stated in para. 3.49 are discharged, as applicable, registrants and licensees, in cooperation with suppliers, shall ensure that medical radiological equipment and software that could influence the delivery of medical exposure are used only if they conform to the applicable standards of the International Electrotechnical Commission and the International Organization for Standardization or to national standards adopted by the regulatory body.

Operational considerations

3.163. For diagnostic radiological procedures and image guided interventional procedures, the radiological medical practitioner, in cooperation with the medical radiation technologist and the medical physicist, and if appropriate with the radiopharmacist or radiochemist, shall ensure that the following are used:

- (a) Appropriate medical radiological equipment and software, and, for nuclear medicine, appropriate radiopharmaceuticals;
- (b) Appropriate techniques and parameters to deliver a medical exposure of the patient that is the minimum necessary to fulfil the clinical purpose of the radiological procedure, with account taken of relevant norms of acceptable image quality established by relevant professional bodies and of relevant diagnostic reference levels established in accordance with paras 3.148 and 3.169.

3.164. For therapeutic radiological procedures, the radiological medical practitioner, in cooperation with the medical physicist and the medical radiation technologist, shall ensure that for each patient the exposure of volumes other than the planning target volume is kept as low as reasonably achievable consistent with delivery of the prescribed dose to the planning target volume within the required tolerances.

3.165. For therapeutic radiological procedures in which radiopharmaceuticals are administered, the radiological medical practitioner, in cooperation with the medical physicist and the medical radiation technologist, and if appropriate with the radiopharmacist or radiochemist, shall ensure that for each patient the appropriate radiopharmaceutical with the appropriate activity is selected and administered, so that the radioactivity is primarily localized in the organ(s) of interest, while the radioactivity in the rest of the body is kept as low as reasonably achievable.

3.166. Registrants and licensees shall ensure that the particular aspects of medical exposures are considered in the optimization process for:

- (a) Paediatric patients subject to medical exposure;
- (b) Individuals subject to medical exposure as part of an approved health screening programme;
- (c) Volunteers subject to medical exposure as part of a programme of biomedical research;

- (d) Relatively high doses⁴⁴ to the patient;
- (e) Exposure of the embryo or fetus, in particular for radiological procedures in which the abdomen or pelvis of the pregnant female patient is exposed to the useful radiation beam or could otherwise receive a significant dose;
- (f) Exposure of a breastfed infant as a result of a female patient having undergone a radiological procedure with radiopharmaceuticals.

Calibration

3.167. In accordance with para. 3.154(d) and (e), the medical physicist shall ensure that:

- (a) All sources giving rise to medical exposure are calibrated in terms of appropriate quantities using internationally accepted or nationally accepted protocols;
- (b) Calibrations are carried out at the time of commissioning a unit prior to clinical use, after any maintenance procedure that could affect the dosimetry and at intervals approved by the regulatory body;
- (c) Calibrations of radiation therapy units are subject to independent verification⁴⁵ prior to clinical use;
- (d) Calibration of all dosimeters used for dosimetry of patients and for the calibration of sources is traceable to a standards dosimetry laboratory.

Dosimetry of patients

3.168. Registrants and licensees shall ensure that dosimetry of patients is performed and documented by or under the supervision of a medical physicist, using calibrated dosimeters and following internationally accepted or nationally accepted protocols, including dosimetry to determine the following:

⁴⁴ The term 'relatively high dose' is intended to apply in a given context. Clearly, doses from therapeutic radiological procedures are included in 'relatively high doses', as are image guided interventional procedures. In medical imaging, 'relatively high doses' would include doses from exposures in computed tomography and in radiological procedures in nuclear medicine with higher doses.

⁴⁵ 'Independent verification' ideally means verification by a different, independent medical physicist using different dosimetry equipment. However, other options, such as verification by a second medical physicist or verification using a second set of equipment, or even using a form of verification by postal thermoluminescence dosimetry, could be acceptable. In checking for compliance, the regulatory body needs to be aware of the limitations on local resources.

- (a) For diagnostic radiological procedures, typical doses to patients for common procedures;
- (b) For image guided interventional procedures, typical doses to patients;
- (c) For therapeutic radiological procedures, absorbed doses to the planning target volume for each patient treated with external beam therapy and/or brachytherapy and absorbed doses to relevant tissues or organs as determined by the radiological medical practitioner;
- (d) For therapeutic radiological procedures with unsealed sources, typical absorbed doses to patients.

Diagnostic reference levels

3.169. Registrants and licensees shall ensure that:

- (a) Local assessments, on the basis of the measurements required in para. 3.168, are made at approved intervals for those radiological procedures for which diagnostic reference levels have been established (para. 3.148).
- (b) A review is conducted to determine whether the optimization of protection and safety for patients is adequate, or whether corrective action is required if, for a given radiological procedure:
 - (i) Typical doses or activities exceed the relevant diagnostic reference level; or
 - (ii) Typical doses or activities fall substantially below the relevant diagnostic reference level and the exposures do not provide useful diagnostic information or do not yield the expected medical benefit to the patient.

Quality assurance for medical exposures

3.170. Registrants and licensees, in applying the requirements of these Standards in respect of management systems, shall establish a comprehensive programme of quality assurance for medical exposures with the active participation of medical physicists, radiological medical practitioners, medical radiation technologists and, for complex nuclear medicine facilities, radiopharmacists and radiochemists, and in conjunction with other health professionals as appropriate. Principles established by the World Health Organization, the Pan American Health Organization and relevant professional bodies shall be taken into account.

3.171. Registrants and licensees shall ensure that programmes of quality assurance for medical exposure include, as appropriate to the medical radiation facility:

- (a) Measurements of the physical parameters of medical radiological equipment made by, or under the supervision of, a medical physicist:
 - (i) At the time of acceptance and commissioning of the equipment prior to its clinical use on patients;
 - (ii) Periodically thereafter;
 - (iii) After any major maintenance procedure that could affect protection and safety of patients;
 - (iv) After any installation of new software or modification of existing software that could affect protection and safety of patients.
- (b) Implementation of corrective actions if measured values of the physical parameters mentioned in (a) above are outside established tolerance limits.
- (c) Verification of the appropriate physical and clinical factors used in radiological procedures.
- (d) Maintaining records of relevant procedures and results.
- (e) Periodic checks of the calibration and conditions of operation of dosimetry equipment and monitoring equipment.

3.172. Registrants and licensees shall ensure that regular and independent audits are made of the programme of quality assurance for medical exposures, and that their frequency is in accordance with the complexity of the radiological procedures being performed and the associated risks.

Dose constraints

3.173. Registrants and licensees shall ensure that relevant dose constraints (para. 3.149(a)(i)) are used in the optimization of protection and safety in any radiological procedure in which an individual acts as a carer or comforter.

3.174. Registrants and licensees shall ensure that dose constraints specified or approved by the ethics committee (or other institutional body that has been assigned functions similar to those of an ethics committee by the relevant authority) on a case by case basis as part of a proposal for biomedical research (para. 3.161) are used in the optimization of protection and safety for persons subject to exposure as part of a programme of biomedical research.

Requirement 39: Pregnant or breast-feeding female patients

Registrants and licensees shall ensure that there are arrangements in place for appropriate radiation protection in cases where a female patient is or might be pregnant or is breast-feeding.

3.175. Registrants and licensees shall ensure that signs in appropriate languages are placed in public places, waiting rooms for patients, cubicles and other appropriate places, and that other means of communication are also used as appropriate⁴⁶, to request female patients who are to undergo a radiological procedure to notify the radiological medical practitioner, medical radiation technologist or other personnel in the event that:

- (a) She is or might be pregnant;
- (b) She is breast-feeding and the scheduled radiological procedure includes the administration of a radiopharmaceutical.

3.176. Registrants and licensees shall ensure that there are procedures in place for ascertaining the pregnancy status of a female patient of reproductive capacity before the performance of any radiological procedure that could result in a significant dose to the embryo or fetus, so that this information can be considered in the justification for the radiological procedure (paras 3.155 and 3.156) and in the optimization of protection and safety (para. 3.166).

3.177. Registrants and licensees shall ensure that there are arrangements in place for establishing that a female patient is not currently breast-feeding before the performance of any radiological procedure involving the administration of a radiopharmaceutical that could result in a significant dose to a breastfed infant, so that this information can be considered in the justification for the radiological procedure (paras 3.155 and 3.157) and in the optimization of protection and safety (para. 3.166).

Requirement 40: Release of patients after radionuclide therapy

Registrants and licensees shall ensure that there are arrangements in place to ensure appropriate radiation protection for members of the public and for family members before a patient is released following radionuclide therapy.

3.178. The radiological medical practitioner shall ensure that no patient who has undergone a therapeutic radiological procedure with a sealed source or an unsealed source is discharged from a medical radiation facility until it has been established by either a medical physicist or the facility's radiation protection officer that:

⁴⁶ 'Other means of communication' include explicitly asking female patients whether they are or might be pregnant or whether they are breast-feeding.

- (a) The activity of radionuclides in the patient is such that doses that could be received by members of the public and family members would be in compliance with the requirements set by the relevant authorities (para. 3.149(b)); and
- (b) The patient or the legal guardian of the patient is provided with:
 - (i) Written instructions for keeping doses to persons in contact with or in the vicinity of the patient as low as reasonably achievable and for avoiding the spread of contamination;
 - (ii) Information on the radiation risks.

Requirement 41: Unintended and accidental medical exposures

Registrants and licensees shall ensure that all practicable measures are taken to minimize the likelihood of unintended or accidental medical exposures. Registrants and licensees shall promptly investigate unintended or accidental medical exposures and, if appropriate, shall implement corrective actions.

3.179. Registrants and licensees, in accordance with the relevant requirements of paras 2.51, 3.41–3.42 and 3.49–3.50, shall ensure that all practicable measures are taken to minimize the likelihood of unintended or accidental medical exposures arising from flaws in design and operational failures of medical radiological equipment, from failures of and errors in software, or as a result of human error.

Investigation of unintended and accidental medical exposures

3.180. Registrants and licensees shall promptly investigate any of the following unintended or accidental medical exposures:

- (a) Any medical treatment delivered to the wrong individual or to the wrong tissue or organ of the patient, or using the wrong radiopharmaceutical, or with an activity, a dose or dose fractionation differing substantially from (over or under) the values prescribed by the radiological medical practitioner, or that could lead to unduly severe secondary effects;
- (b) Any diagnostic radiological procedure or image guided interventional procedure in which the wrong individual or the wrong tissue or organ of the patient is subject to exposure;
- (c) Any exposure for diagnostic purposes that is substantially greater than was intended;
- (d) Any exposure arising from an image guided interventional procedure that is substantially greater than was intended;

- (e) Any inadvertent exposure of the embryo or fetus in the course of performing a radiological procedure;
- (f) Any failure of medical radiological equipment, failure of software or system failure, or accident, error, mishap or other unusual occurrence with the potential for subjecting the patient to a medical exposure that is substantially different from what was intended.

3.181. Registrants and licensees shall, with regard to any unintended or accidental medical exposures investigated as required in para. 3.180:

- (a) Calculate or estimate the doses received and the dose distribution within the patient;
- (b) Indicate the corrective actions required to prevent the recurrence of such an unintended or accidental medical exposure;
- (c) Implement all the corrective actions that are under their own responsibility;
- (d) Produce and keep, as soon as possible after the investigation or as otherwise required by the regulatory body, a written record that states the cause of the unintended or accidental medical exposure and includes the information specified in (a)–(c) above, as relevant, and any other information as required by the regulatory body; and for significant unintended or accidental medical exposures or as otherwise required by the regulatory body, submit this written record, as soon as possible, to the regulatory body, and to the relevant health authority if appropriate;
- (e) Ensure that the appropriate radiological medical practitioner informs the referring medical practitioner and the patient or the patient's legal authorized representative of the unintended or accidental medical exposure.

Requirement 42: Reviews and records

Registrants and licensees shall ensure that radiological reviews are performed periodically at medical radiation facilities and that records are maintained.

Radiological reviews

3.182. Registrants and licensees shall ensure that radiological reviews are performed periodically by the radiological medical practitioners at the medical radiation facility, in cooperation with the medical radiation technologists and the medical physicists. The radiological review shall include an investigation and critical review of the current practical application of the radiation protection

principles of justification and optimization for the radiological procedures that are performed in the medical radiation facility.

Records

3.183. Registrants and licensees shall maintain for a period as specified by the regulatory body and shall make available, as required, the following personnel records:

- (a) Records of any delegation of responsibilities by a principal party (as required in para. 3.154(f));
- (b) Records of training of personnel in radiation protection (as required in para. 3.150(b)).

3.184. Registrants and licensees shall maintain for a period as specified by the regulatory body and shall make available, as required, the following records of calibration, dosimetry and quality assurance:

- (a) Records of the results of the calibrations and periodic checks of the relevant physical and clinical parameters selected during treatment of patients;
- (b) Records of dosimetry of patients, as required in para. 3.168;
- (c) Records of local assessments and reviews made with regard to diagnostic reference levels, as required in para. 3.169;
- (d) Records associated with the quality assurance programme, as required in para. 3.171(d).

3.185. Registrants and licensees shall maintain for a period as specified by the regulatory body and shall make available, as required, the following records for medical exposure:

- (a) For diagnostic radiology, information necessary for retrospective assessment of doses, including the number of exposures and the duration of fluoroscopic radiological procedures;
- (b) For image guided interventional procedures, information necessary for retrospective assessment of doses, including the duration of the fluoroscopic component and the number of images acquired;
- (c) For nuclear medicine, the types of radiopharmaceutical administered and their activity;
- (d) For external beam radiation therapy or brachytherapy, a description of the planning target volume, the absorbed dose to the centre of the planning target volume, and the maximum and minimum absorbed doses delivered

to the planning target volume, or equivalent alternative information on absorbed doses to the planning target volume, and the absorbed doses to relevant tissues or organs as determined by the radiological medical practitioner; and in addition, for external beam radiation therapy, the dose fractionation and the overall treatment time;

- (e) Exposure records for volunteers subject to medical exposure as part of a programme of biomedical research;
- (f) Reports on investigations of unintended and accidental medical exposures (as required in para. 3.181(d)).

4. EMERGENCY EXPOSURE SITUATIONS

SCOPE

4.1. The requirements for emergency exposure situations established in Section 4 apply to activities undertaken in preparedness for and in response to a nuclear or radiological emergency.

GENERIC REQUIREMENTS

Requirement 43: Emergency management system

The government shall ensure that an integrated and coordinated emergency management system is established and maintained.

4.2. The government shall ensure that an emergency management system is established and maintained on the territories and within the jurisdiction of the State for the purposes of emergency response to protect human life, health and the environment in the event of a nuclear or radiological emergency.

4.3. The emergency management system shall be designed to be commensurate with the results of a hazard assessment [15] and to enable an effective emergency response to reasonably foreseeable events (including very low probability events) in connection with facilities or activities.

4.4. The emergency management system shall be integrated, to the extent practicable, into an all-hazards emergency management system.

4.5. The emergency management system shall provide for essential elements at the scene, and at the local, national and international level, as appropriate, including the following [15]:

- (a) Hazard assessment;
- (b) Development and exercising of emergency plans and emergency procedures;
- (c) Clear allocation of responsibilities to persons and organizations having roles in the arrangements for emergency preparedness and response;
- (d) Arrangements for efficient and effective cooperation and coordination among organizations;
- (e) Reliable communication, including public information;
- (f) Optimized protection strategies for the implementation and the termination of measures for the protection of members of the public who could be subject to exposure in an emergency, including relevant considerations for protection of the environment;
- (g) Arrangements for the protection of emergency workers;
- (h) Education and training, including training in radiation protection, of all persons involved in emergency response and exercising of emergency plans and emergency procedures;
- (i) Preparations for the transition from emergency exposure situation to existing exposure situation;
- (j) Arrangements for the medical response and the public health response in an emergency;
- (k) Provision for individual monitoring and environmental monitoring and for dose assessment;
- (l) Involvement of relevant parties and interested parties.

4.6. The government shall ensure the coordination of its emergency arrangements and capabilities with the relevant international emergency arrangements.

PUBLIC EXPOSURE

Requirement 44: Preparedness and response for an emergency

The government shall ensure that protection strategies are developed, justified and optimized at the planning stage, and that emergency response is undertaken by their timely implementation.

4.7. The government shall ensure that protection strategies are developed, justified and optimized at the planning stage, by using scenarios based on the hazard assessment, for avoiding deterministic effects and reducing the likelihood of stochastic effects due to public exposure.

4.8. Development of a protection strategy shall include, but shall not be limited to, the following three successive steps:

- (1) A reference level expressed in terms of residual dose shall be set, typically an effective dose in the range of 20–100 mSv, that includes dose contributions via all exposure pathways. The protection strategy shall include planning for residual doses to be as low as reasonably achievable below the reference level, and the strategy shall be optimized.
- (2) On the basis of the outcome of the optimization of the protection strategy, using the reference level, generic criteria for particular protective actions and other response actions, expressed in terms of projected dose or of dose that has been received, shall be developed. If the numerical values of the generic criteria⁴⁷ are exceeded, those protective actions and other response actions, either individually or in combination, shall be implemented.
- (3) Once the protection strategy has been optimized and a set of generic criteria has been developed, pre-established operational criteria for initiating the different parts of an emergency plan, primarily for the initial phase, shall be derived from the generic criteria. Operational criteria, such as on-scene conditions, operational intervention levels and emergency action levels, shall be expressed in terms of parameters or observable conditions. Arrangements shall be established in advance to revise these operational criteria, as appropriate, in an emergency, with account taken of the prevailing conditions as they evolve.

4.9. Each protective action shall be justified in the context of the protection strategy.

4.10. The government shall ensure that in making arrangements for emergency preparedness and response it is taken into consideration that emergencies are dynamic, that decisions taken early in the emergency response may influence subsequent actions, and that different geographical areas may have different prevailing conditions and there may be different requirements for the response.

⁴⁷ Table A–1 in the Annex (p. 380) provides a set of generic criteria for use in the protection strategy that are compatible with reference levels within a range of 20–100 mSv, and provides further details for specific actions in different time frames.

4.11. The government shall ensure that the response in an emergency exposure situation is undertaken by the timely implementation of arrangements for emergency response, including but not limited to:

- (a) Promptly taking protective actions and other response actions to avoid severe deterministic effects on the basis of observed conditions and, if possible, before any exposure occurs. Dose levels required to be used as generic criteria for preventing severe deterministic effects are given in Table IV.1 of Schedule IV (p. 372);
- (b) Assessing the effectiveness of the protective actions and other response actions taken and modifying them as appropriate;
- (c) Comparing residual doses with the applicable reference level, giving priority to those groups for whom residual doses exceed the reference level;
- (d) Implementing further protection strategies as necessary, on the basis of prevailing conditions and available information.

EXPOSURE OF EMERGENCY WORKERS

Requirement 45: Arrangements for controlling the exposure of emergency workers

The government shall establish a programme for managing, controlling and recording the doses received in an emergency by emergency workers.

4.12. The government shall establish a programme for managing, controlling and recording the doses received in an emergency by emergency workers, which shall be implemented by response organizations and employers.

4.13. The response organization and employers responsible for ensuring compliance with the requirements in paras 4.14–4.19 shall be specified in the emergency plan.

4.14. In an emergency exposure situation, the relevant requirements for occupational exposure in planned exposure situations (paras 3.69–3.116) shall be applied for emergency workers, in accordance with a graded approach, except as required in para. 4.15.

4.15. Response organizations and employers shall ensure that no emergency worker is subject to an exposure in an emergency in excess of 50 mSv other than:

- (a) For the purposes of saving life or preventing serious injury;
- (b) When undertaking actions to prevent severe deterministic effects and actions to prevent the development of catastrophic conditions that could significantly affect people and the environment; or
- (c) When undertaking actions to avert a large collective dose.

4.16. In the exceptional circumstances specified in para. 4.15, response organizations and employers shall make all reasonable efforts to keep doses to emergency workers below the values set out in Table IV.2 of Schedule IV (p. 373). In addition, emergency workers undertaking actions as a result of which their doses could approach or exceed the values set out in Table IV.2 of Schedule IV shall do so only when the expected benefits to others would clearly outweigh the risks to the emergency workers.

4.17. Response organizations and employers shall ensure that emergency workers who undertake actions in which the doses received might exceed 50 mSv do so voluntarily⁴⁸; that they have been clearly and comprehensively informed in advance of the associated health risks, as well as of available measures for protection and safety; and that they are, to the extent possible, trained in the actions that they may be required to take.

4.18. Response organizations and employers shall take all reasonable steps to assess and record the doses received in an emergency by emergency workers. Information on the doses received and information concerning the associated health risks shall be communicated to the workers involved.

4.19. Workers who receive doses in an emergency exposure situation shall not normally be precluded from incurring further occupational exposure. However, qualified medical advice shall be obtained before any further occupational exposure if such a worker has received a dose exceeding 200 mSv or at the request of the worker.

⁴⁸ The voluntary basis for actions to be taken by emergency workers is usually covered in emergency arrangements.

TRANSITION FROM AN EMERGENCY EXPOSURE SITUATION TO AN EXISTING EXPOSURE SITUATION

Requirement 46: Arrangements for the transition from an emergency exposure situation to an existing exposure situation

The government shall ensure that arrangements are in place and are implemented as appropriate for the transition from an emergency exposure situation to an existing exposure situation.

4.20. The government shall ensure that, as part of its overall emergency preparedness, arrangements are in place for the transition from an emergency exposure situation to an existing exposure situation. The arrangements shall take into account that different geographical areas may undergo the transition at different times. The responsible authority shall take the decision to make the transition to an existing exposure situation. The transition shall be made in a coordinated and orderly manner, by making any necessary transfer of responsibilities between organizations, with the involvement of relevant authorities and interested parties.

4.21. Workers undertaking work such as repairs to plant and buildings or activities for radioactive waste management, or undertaking remedial actions for the decontamination of the site and surrounding areas, shall be subject to the relevant requirements for occupational exposure in planned exposure situations stated in Section 3.

5. EXISTING EXPOSURE SITUATIONS

SCOPE

5.1. The requirements for existing exposure situations in Section 5 apply to:

- (a) Exposure due to contamination of areas by residual radioactive material deriving from:
 - (i) Past activities that were never subject to regulatory control or that were subject to regulatory control but not in accordance with the requirements of these Standards;

- (ii) A nuclear or radiological emergency, after an emergency has been declared to be ended (as required in para. 4.20).
- (b) Exposure due to commodities, including food, feed, drinking water and construction materials, that incorporate radionuclides deriving from residual radioactive material as stated in para. 5.1(a).
- (c) Exposure due to natural sources, including:
 - (i) ^{222}Rn and its progeny and ^{220}Rn and its progeny, in workplaces other than those workplaces for which exposure due to other radionuclides in the uranium decay chain or the thorium decay chain is controlled as a planned exposure situation, in dwellings and in other buildings with high occupancy factors for members of the public;
 - (ii) Radionuclides of natural origin, regardless of activity concentration, in commodities, including food, feed, drinking water, agricultural fertilizer and soil amendments, and construction materials, and residual radioactive material in the environment;
 - (iii) Materials, other than those stated in (c)(ii) above, in which the activity concentration of no radionuclide in either the uranium decay chain or the thorium decay chain exceeds 1 Bq/g and the activity concentration of ^{40}K does not exceed 10 Bq/g;
 - (iv) Exposure of aircrew and space crew to cosmic radiation.

GENERIC REQUIREMENTS

Requirement 47: Responsibilities of the government specific to existing exposure situations

The government shall ensure that existing exposure situations that have been identified are evaluated to determine which occupational exposures and public exposures are of concern from the point of view of radiation protection.

5.2. The government shall ensure that, when an existing exposure situation is identified, responsibilities for protection and safety are assigned and appropriate reference levels are established.

5.3. The government shall include in the legal and regulatory framework for protection and safety (see Section 2) provision for the management of existing exposure situations. The government, in the legal and regulatory framework, as appropriate:

- (a) Shall specify the exposure situations that are included in the scope of existing exposure situations;⁴⁹
- (b) Shall specify the general principles underlying the protection strategies developed to reduce exposure when remedial actions and protective actions have been determined to be justified;⁵⁰
- (c) Shall assign responsibilities for the establishment and implementation of protection strategies to the regulatory body and to other relevant authorities⁵¹ and, as appropriate, to registrants, licensees and other parties involved in the implementation of remedial actions and protective actions;
- (d) Shall provide for the involvement of interested parties in decisions regarding the development and implementation of protection strategies, as appropriate.

5.4. The regulatory body or other relevant authority assigned to establish a protection strategy for an existing exposure situation shall ensure that it specifies:

- (a) The objectives to be achieved by means of the protection strategy;
- (b) Appropriate reference levels.

5.5. The regulatory body or other relevant authority shall implement the protection strategy, including:

- (a) Arranging for evaluation of the available remedial actions and protective actions for achieving the objectives, and for evaluation of the efficiency of the actions planned and implemented;
- (b) Ensuring that information is available to individuals subject to exposure on potential health risks and on the means available for reducing their exposures and the associated risks.

⁴⁹ In the case of exposure due to radon, the types of situation that are included in the scope of existing exposure situations will include exposure in workplaces for which the exposure due to radon is not required by or directly related to the work and for which annual average activity concentrations due to ²²²Rn might be expected not to exceed the reference level established in accordance with para. 5.27.

⁵⁰ Such actions include remedial actions such as the removal or reduction of the source giving rise to the exposure, as well as other longer term protective actions such as restriction of the use of construction materials, restriction of the consumption of foodstuffs and restriction of land use or of access to land or buildings.

⁵¹ In existing exposure situations that do not fall under the jurisdiction of the regulatory body, another relevant authority such as a health authority may have authority for implementing measures for protection and safety.

PUBLIC EXPOSURE

Scope

5.6. The requirements in respect of public exposure in existing exposure situations (paras 5.7–5.23) apply to any public exposure arising from the situations specified in para. 5.1.

Requirement 48: Justification for protective actions and optimization of protection and safety

The government and the regulatory body or other relevant authority shall ensure that remedial actions and protective actions are justified and that protection and safety is optimized.

5.7. The government and the regulatory body or other relevant authority shall ensure that the protection strategy for the management of existing exposure situations, established in accordance with paras 5.2 and 5.4, is commensurate with the radiation risks associated with the existing exposure situation; and that remedial actions or protective actions are expected to yield sufficient benefits to outweigh the detriments associated with taking them, including detriments in the form of radiation risks.⁵²

5.8. The regulatory body or other relevant authority and other parties responsible for remedial actions or protective actions shall ensure that the form, scale and duration of such actions are optimized. While this optimization process is intended to provide optimized protection for all individuals subject to exposure, priority shall be given to those groups for whom the dose exceeds the reference level. All reasonable steps shall be taken to prevent doses from remaining above the reference levels. Reference levels shall typically be expressed as an annual effective dose to the representative person in the range of 1–20 mSv or other corresponding quantity, the actual value depending on the feasibility of controlling the situation and on experience in managing similar situations in the past.

⁵² The implementation of remedial actions (remediation) does not imply the elimination of all radioactivity or all traces of radioactive substances. The optimization process may lead to extensive remediation but not necessarily to the restoration of previous conditions.

5.9. The regulatory body or other relevant authority shall periodically review the reference levels to ensure that they remain appropriate in the light of the prevailing circumstances.

Requirement 49: Responsibilities for remediation of areas with residual radioactive material

The government shall ensure that provision is made for identifying those persons or organizations responsible for areas with residual radioactive material; for establishing and implementing remediation programmes and post-remediation control measures, if appropriate; and for putting in place an appropriate strategy for radioactive waste management.

5.10. For the remediation of areas with residual radioactive material deriving from past activities or from a nuclear or radiological emergency (para. 5.1(a)), the government shall ensure that provision is made in the framework for protection and safety for:

- (a) The identification of those persons or organizations responsible for the contamination of areas and those responsible for financing the remediation programme, and the determination of appropriate arrangements for alternative sources of funding if such persons or organizations are no longer present or are unable to meet their liabilities;
- (b) The designation of persons or organizations responsible for planning, implementing and verifying the results of remedial actions;
- (c) The establishment of any restrictions on the use of or access to the areas concerned before, during and, if necessary, after remediation;
- (d) An appropriate system for maintaining, retrieval and amendment of records that cover the nature and the extent of contamination; the decisions made before, during and after remediation; and information on verification of the results of remedial actions, including the results of all monitoring programmes after completion of the remedial actions.

5.11. The government shall ensure that a strategy for radioactive waste management is put in place to deal with any waste arising from the remedial actions and that provision for such a strategy is made in the framework for protection and safety.

5.12. The persons or organizations responsible for the planning, implementation and verification of remedial actions shall, as appropriate, ensure that:

- (a) A remedial action plan, supported by a safety assessment, is prepared and is submitted to the regulatory body or other relevant authority for approval.
- (b) The remedial action plan is aimed at the timely and progressive reduction of the radiation risks and eventually, if possible, at the removal of restrictions on the use of or access to the area.
- (c) Any additional doses received by members of the public as a result of the remedial actions are justified on the basis of the resulting net benefit, including consideration of the consequent reduction of the annual dose.
- (d) In the choice of the optimized remediation option:
 - (i) Radiological impacts on people and the environment are considered together with non-radiological impacts on people and the environment, and with technical, societal and economic factors;
 - (ii) The costs of the transport and management of radioactive waste, the radiation exposure of and health risks to the workers managing the radioactive waste, and any subsequent public exposure associated with its disposal are all taken into account.
- (e) A mechanism for public information is in place and interested parties are involved in the planning, implementation and verification of the remedial actions, including any monitoring following remediation.
- (f) A monitoring programme is established and implemented.
- (g) A system for maintaining adequate records relating to the existing exposure situation and to actions taken for protection and safety is in place.
- (h) Procedures are in place for reporting to the regulatory body or other relevant authority on any abnormal conditions relevant to protection and safety.

5.13. The regulatory body, in accordance with para. 2.29, or other relevant authority shall take responsibility, in particular for:

- (a) Review of the safety assessment submitted by the responsible person or organization, approval of the remedial action plan and of any subsequent changes to the remedial action plan, and granting of any necessary authorization;
- (b) Establishment of criteria and methods for assessing safety;
- (c) Review of work procedures, monitoring programmes and records;
- (d) Review and approval of significant changes to procedures or equipment that may have radiological environmental impacts or that may alter the exposure conditions for workers taking remedial actions or for members of the public;
- (e) Where necessary, establishment of regulatory requirements for control measures following remediation.

5.14. The person or organization responsible for carrying out the remedial actions:

- (a) Shall ensure that the work, including management of the radioactive waste arising, is conducted in accordance with the remedial action plan;
- (b) Shall take responsibility for all aspects of protection and safety, including the conduct of a safety assessment;
- (c) Shall monitor the area regularly during the remediation so as to verify levels of contamination, to verify compliance with the requirements for radioactive waste management, and to enable any unexpected levels of radiation to be detected and the remedial action plan to be modified accordingly, subject to approval by the regulatory body or other relevant authority;
- (d) Shall perform a radiological survey after completion of remedial actions to demonstrate that the end point conditions, as established in the remedial action plan, have been met;
- (e) Shall prepare and retain a final remediation report and shall submit a copy to the regulatory body or other relevant authority.

5.15. After the remedial actions have been completed, the regulatory body or other relevant authority:

- (a) Shall review, amend as necessary and formalize the type, extent and duration of any post-remediation control measures already identified in the remedial action plan, with due consideration of the residual radiation risks.
- (b) Shall identify the person or organization responsible for any post-remediation control measures.
- (c) Shall, where necessary, impose specific restrictions for the remediated area to control:
 - (i) Access by unauthorized persons;
 - (ii) Removal of radioactive material or use of such material, including its use in commodities;
 - (iii) Future use of the area, including the use of water resources and its use for the production of food or feed, and the consumption of food from the area.
- (d) Shall periodically review conditions in the remediated area and, if appropriate, shall amend or remove any restrictions.

5.16. The person or organization responsible for post-remediation control measures shall establish and maintain, for as long as required by the regulatory body or other relevant authority, an appropriate programme, including any

necessary provision for monitoring, to verify the long term effectiveness of the completed remedial actions for areas in which controls are required after remediation.

5.17. For those areas with long lasting residual radioactive material, in which the government has decided to allow habitation and the resumption of social and economic activities, the government, in consultation with interested parties, shall ensure that arrangements are in place, as necessary, for the continuing control of exposure with the aim of establishing conditions for sustainable living, including:

- (a) Establishment of reference levels for protection and safety that are consistent with day to day life;
- (b) Establishment of an infrastructure to support continuing ‘self-help protective actions’ in the affected areas, such as by the provision of information and advice, and by monitoring.

5.18. The conditions prevailing after the completion of remedial actions, if the regulatory body or other relevant authority has imposed no restrictions or controls, shall be considered to constitute the background conditions for any new facilities and activities or for habitation on the land.

Requirement 50: Public exposure due to radon indoors

The government shall provide information on levels of radon indoors and the associated health risks and, if appropriate, shall establish and implement an action plan for controlling public exposure due to radon indoors.

5.19. As part of its responsibilities, as required in para. 5.3, the government shall ensure that:

- (a) Information is gathered on activity concentrations of radon in dwellings and other buildings with high occupancy factors for members of the public⁵³ through appropriate means, such as representative radon surveys;
- (b) Relevant information on exposure due to radon and the associated health risks, including the increased risks relating to smoking, is provided to the public and other interested parties.

⁵³ Buildings with high occupancy factors for members of the public include kindergartens, schools and hospitals.

5.20. Where activity concentrations of radon that are of concern for public health are identified on the basis of the information gathered as required in para. 5.19(a), the government shall ensure that an action plan is established comprising coordinated actions to reduce activity concentrations of radon in existing buildings and in future buildings, which includes⁵⁴:

- (a) Establishing an appropriate reference level for ^{222}Rn for dwellings and other buildings with high occupancy factors for members of the public, with account taken of the prevailing social and economic circumstances, that in general will not exceed an annual average activity concentration due to ^{222}Rn of 300 Bq/m^3 ⁵⁵;
- (b) Reducing activity concentrations of ^{222}Rn and consequent exposures to levels at which protection is optimized;
- (c) Giving priority to actions to reduce activity concentrations of ^{222}Rn in those situations for which such action is likely to be most effective⁵⁶;
- (d) Including in building codes appropriate preventive measures and corrective actions to prevent the ingress of ^{222}Rn and to facilitate further actions wherever necessary.

5.21. The government shall assign responsibility for:

- (a) Establishing and implementing the action plan for controlling public exposure due to ^{222}Rn indoors;
- (b) Determining the circumstances under which actions are to be mandatory or are to be voluntary, with account taken of legal requirements and of the prevailing social and economic circumstances.

⁵⁴ Guidance on the preparation of an action plan for radon is provided in Ref. [6], for example.

⁵⁵ On the assumption of an equilibrium factor for ^{222}Rn of 0.4 and an annual occupancy of 7000 h, the value of activity concentration due to ^{222}Rn of 300 Bq/m^3 corresponds to an annual effective dose of the order of 10 mSv.

⁵⁶ Examples of giving priority to reducing activity concentrations of ^{222}Rn in those situations for which such action is likely to be most effective include (i) specifying the levels of activity concentrations of ^{222}Rn in dwellings and other buildings with high occupancy factors at which protection can be considered optimized; (ii) identifying radon prone areas; (iii) identifying characteristics of buildings that are likely to give rise to elevated activity concentrations of ^{222}Rn ; and (iv) identifying and requiring preventive measures for radon in future buildings that can be introduced at relatively low cost.

Requirement 51: Exposure due to radionuclides in commodities

The regulatory body or other relevant authority shall establish reference levels for exposure due to radionuclides in commodities.

5.22. The regulatory body or other relevant authority shall establish specific reference levels for exposure due to radionuclides in commodities such as construction materials, food and feed, and in drinking water, each of which shall typically be expressed as, or be based on, an annual effective dose to the representative person that generally does not exceed a value of about 1 mSv.

5.23. The regulatory body or other relevant authority shall consider the guideline levels for radionuclides in food traded internationally that could contain radioactive substances as a result of a nuclear or radiological emergency, which have been published by the Joint Food and Agriculture Organization of the United Nations/World Health Organization Codex Alimentarius Commission [23]. The regulatory body or other relevant authority shall consider the guideline levels for radionuclides contained in drinking water that have been published by the World Health Organization [24].

OCCUPATIONAL EXPOSURE

Scope

5.24. The requirements in respect of occupational exposure in existing exposure situations (paras 5.25–5.33) apply to any occupational exposure arising from the situations specified in para. 5.1.

Requirement 52: Exposure in workplaces

The regulatory body shall establish and enforce requirements for the protection of workers in existing exposure situations.

5.25. The requirements in respect of public exposure stated in paras 5.7–5.9 shall be applied for protection and safety for workers in existing exposure situations, other than in those specific situations identified in paras 5.26–5.33.

Remediation of areas with residual radioactive material

5.26. Employers shall ensure that the exposure of workers undertaking remedial actions is controlled in accordance with the relevant requirements on occupational exposure in planned exposure situations as established in Section 3.

Exposure due to radon in workplaces

5.27. The regulatory body or other relevant authority shall establish a strategy for protection against exposure due to ^{222}Rn in workplaces, including the establishment of an appropriate reference level for ^{222}Rn . The reference level for ^{222}Rn shall be set at a value that does not exceed an annual average activity concentration of ^{222}Rn of 1000 Bq/m^3 , with account taken of the prevailing social and economic circumstances.⁵⁷

5.28. Employers shall ensure that activity concentrations of ^{222}Rn in workplaces are as low as reasonably achievable below the reference level established in accordance with para. 5.27, and shall ensure that protection is optimized.

5.29. If, despite all reasonable efforts by the employer to reduce activity concentrations of radon, the activity concentration of ^{222}Rn in workplaces remains above the reference level established in accordance with para. 5.27, the relevant requirements for occupational exposure in planned exposure situations as stated in Section 3 shall apply.

Exposure of aircrew and space crew due to cosmic radiation

5.30. The regulatory body or other relevant authority shall determine whether assessment of the exposure of aircrew due to cosmic radiation is warranted.

5.31. Where such assessment is deemed to be warranted, the regulatory body or other relevant authority shall establish a framework which shall include a reference level of dose and a methodology for the assessment and recording of doses received by aircrew from occupational exposure to cosmic radiation.

⁵⁷ On the assumption of an equilibrium factor for ^{222}Rn of 0.4 and an annual occupancy of 2000 h, the value of activity concentration due to ^{222}Rn of 1000 Bq/m^3 corresponds to an annual effective dose of the order of 10 mSv.

5.32. In accordance with para. 5.31:

- (a) Where the doses of aircrew are likely to exceed the reference level, employers of aircrew:
 - (i) Shall assess and keep records of doses;
 - (ii) Shall make records of doses available to aircrew.
- (b) Employers:
 - (i) Shall inform female aircrew of the risk to the embryo or fetus due to exposure to cosmic radiation and of the need for early notification of pregnancy;
 - (ii) Shall apply the requirements of para. 3.114 in respect of notification of pregnancy.

5.33. The regulatory body or other relevant authority shall establish, where appropriate, a framework for radiation protection that applies to individuals in space based activities that is appropriate for the exceptional conditions of space. While the requirements of these Standards in respect of dose limits do not apply to individuals in space based activities, all reasonable efforts shall be made to optimize protection for individuals in space based activities by restricting the doses received by such individuals while not unduly limiting the extent of such activities.

Schedule I

EXEMPTION AND CLEARANCE

CRITERIA FOR EXEMPTION

I.1. The general criteria for exemption of a practice or a source within a practice from some or all of the requirements of these Standards are that:

- (a) Radiation risks arising from the practice or from a source within the practice are sufficiently low as not to warrant regulatory control, with no appreciable likelihood of situations arising that could lead to a failure to meet the general criterion for exemption; or
- (b) Regulatory control of the practice or the source would yield no net benefit, in that no reasonable measures for regulatory control would achieve a worthwhile return in terms of reduction of individual doses or of health risks.

I.2. A practice or a source within a practice may be exempted without further consideration from some or all of the requirements of these Standards under the terms of para. I.1(a) provided that under all reasonably foreseeable circumstances the effective dose expected to be incurred by any individual (normally evaluated on the basis of a safety assessment) owing to the exempt practice or the exempt source within the practice is of the order of 10 μ Sv or less in a year. To take into account low probability scenarios, a different criterion could be used, namely that the effective dose expected to be incurred by any individual for such low probability scenarios does not exceed 1 mSv in a year.

I.3. Under the criteria set out in paras I.1 and I.2, the following sources within justified practices are automatically exempted without further consideration from the requirements of these Standards, including requirements for notification, registration or licensing:

- (a) Material in a moderate amount⁵⁸ for which either the total activity of an individual radionuclide present on the premises at any one time or the activity concentration as used in the practice does not exceed the applicable exemption level given in Table I.1 (p. 111)⁵⁹.
- (b) Material in bulk amount⁵⁸ for which the activity concentration of a given radionuclide of artificial origin used in the practice does not exceed the relevant value given in Table I.2 (p. 124)⁵⁹.
- (c) Radiation generators of a type approved by the regulatory body, or in the form of an electronic tube, such as a cathode ray tube for the display of visual images, provided that:
 - (i) They do not in normal operating conditions cause an ambient dose equivalent rate or a directional dose equivalent rate, as appropriate, exceeding 1 µSv/h at a distance of 0.1 m from any accessible surface of the equipment; or
 - (ii) The maximum energy of the radiation generated is no greater than 5 keV.

I.4. For radionuclides of natural origin, exemption of bulk amounts of material is necessarily considered on a case by case basis⁶⁰ by using a dose criterion of the order of 1 mSv in a year, commensurate with typical doses due to natural background levels of radiation.

⁵⁸ The exemption values (activity concentrations) presented in Table I.1 have been calculated on the basis of scenarios involving a moderate amount of material: “The calculated values apply to practices involving small scale usage of activity where the quantities involved are at the most of the order of a tonne” (see Ref. [25]). The regulatory body will need to establish the amounts for which the concentration values in Table I.1 may be applied, bearing in mind that for many radionuclides, in particular those for which there is no corresponding value given in Table I.2 (p. 124), a restriction on the amount is not meaningful.

⁵⁹ The exemption levels set out in Table I.1 (p. 111) and the exemption and clearance levels set out in Table I.2 are subject to the following considerations: (a) they were derived using a conservative model based on (i) the criteria of paras I.2 and I.11, respectively, and (ii) a series of limiting (bounding) scenarios for use and disposal (see Refs [25, 26] in the case of Table I.1 and Ref. [27] in the case of Table I.2); (b) if there is more than one radionuclide, the derived exemption level or derived clearance level for the mixture is determined as specified in paras I.7 and I.14.

⁶⁰ Material containing radionuclides of natural origin at an activity concentration of less than 1 Bq/g for any radionuclide in the uranium decay chain or the thorium decay chain and of less than 10 Bq/g for ⁴⁰K is not subject to the requirements in Section 3 for planned exposure situations (para. 3.4(a)); hence, the concept of exemption from the requirements of these Standards does not apply for such material.

I.5. The IAEA Regulations for the Safe Transport of Radioactive Material [12] (the IAEA Transport Regulations) do not apply to exempt material or exempt consignments; that is, they do not apply to material in transport for which the activity concentration of the material (for exempt material) or the total activity of radionuclides in the consignment (for an exempt consignment) does not exceed the relevant 'basic radionuclide value' given in the IAEA Transport Regulations for exemption from the requirements of the IAEA Transport Regulations⁶¹. Usually, such basic radionuclide values are numerically equal to the corresponding exempt activity concentrations or exempt activities given in Table I.1 (p. 111).

I.6. Exemptions may be granted subject to conditions specified by the regulatory body, such as conditions relating to the physical or chemical form of the radioactive material, and to its use or the means of its disposal. In particular, such an exemption may be granted for equipment containing radioactive material that is not otherwise automatically exempted without further consideration from some or all of the requirements of these Standards under para. I.3(a) provided that:

- (a) The equipment containing radioactive material is of a type approved by the regulatory body.
- (b) The radioactive material:
 - (i) Is in the form of a sealed source that effectively prevents any contact with the radioactive material and prevents its leakage; or
 - (ii) Is in the form of an unsealed source in a small amount such as sources used for radioimmunoassay.
- (c) In normal operating conditions, the equipment does not cause an ambient dose equivalent rate or a directional dose equivalent rate, as appropriate, exceeding 1 $\mu\text{Sv/h}$ at a distance of 0.1 m from any accessible surface of the equipment.
- (d) Necessary conditions for disposal of the equipment have been specified by the regulatory body.

I.7. For exemption of radioactive material containing more than one radionuclide, on the basis of the levels given in Tables I.1 (p. 111) and I.2 (p. 124), the condition for exemption from some or all of the requirements of

⁶¹ For the purposes of material in transport, 'exemption' means exemption from the requirements of the IAEA Transport Regulations [12].

these Standards is that the sum of the individual radionuclide activities or activity concentrations, as appropriate, is less than the derived exemption level for the mixture (X_m), determined as follows:

$$X_m = \frac{1}{\sum_{i=1}^n \frac{f(i)}{X(i)}} \quad (I.1)$$

where

$f(i)$ is the fraction of activity or activity concentration, as appropriate, of radionuclide i in the mixture;

$X(i)$ is the applicable level for radionuclide i as given in Table I.1 (p. 111) or Table I.2 (p. 124);

and n is the number of radionuclides present.

I.8. Radioactive material arising from authorized discharges is exempt from any requirements for notification, registration or licensing unless otherwise specified by the regulatory body.

I.9. The values provided in Tables I.1 and I.2 are not intended to be applied to the control of discharges or to the control of residual radioactive material in the environment.

CRITERIA FOR CLEARANCE

I.10. The general criteria for clearance are that:

- (a) Radiation risks arising from the cleared material are sufficiently low as not to warrant regulatory control, and there is no appreciable likelihood of occurrence for scenarios that could lead to a failure to meet the general criterion for clearance; or
- (b) Continued regulatory control of the material would yield no net benefit, in that no reasonable control measures would achieve a worthwhile return in terms of reduction of individual doses or reduction of health risks.

I.11. Material may be cleared without further consideration under the terms of para. I.10(a) provided that in reasonably foreseeable circumstances the effective

dose expected to be incurred by any individual owing to the cleared material is of the order of 10 μ Sv or less in a year. To take into account low probability scenarios, a different criterion can be used, namely that the effective dose expected to be incurred by any individual for such low probability scenarios does not exceed 1 mSv in a year.

I.12. Radioactive material within a notified practice or an authorized practice may be cleared without further consideration provided that:

- (a) The activity concentration of an individual radionuclide of artificial origin in solid form does not exceed the relevant level given in Table I.2 (p. 124)⁶²; or
- (b) The activity concentrations of radionuclides of natural origin do not exceed the relevant level given in Table I.3 (p. 128)⁶³; or
- (c) For radionuclides of natural origin in residues that might be recycled into construction materials⁶⁴, or the disposal of which is liable to cause the contamination of drinking water supplies, the activity concentration in the residues does not exceed specific values derived so as to meet a dose criterion of the order of 1 mSv in a year, which is commensurate with typical doses due to natural background levels of radiation.

I.13. Clearance may be granted by the regulatory body for specific situations, on the basis of the criteria of paras I.10 and I.11, with account taken of the physical or chemical form of the radioactive material, and its use or the means of its disposal⁶⁵. Such clearance levels may be specified in terms of activity concentration per unit mass or activity concentration per unit surface area.

⁶² The exemption levels set out in Table I.1 (p. 111) and the exemption and clearance levels set out in Table I.2 are subject to the following considerations: (a) they were derived using a conservative model based on (i) the criteria of paras I.2 and I.11, respectively, and (ii) a series of limiting (bounding) scenarios for use and disposal (see Refs [25, 26] in the case of Table I.1 and Ref. [27] in the case of Table I.2); (b) if there is more than one radionuclide, the derived exemption level or derived clearance level for the mixture is determined as specified in paras I.7 and I.14.

⁶³ These values of activity concentration may also be applied for the clearance of materials arising from practices subject to the clearance criteria given in para. I.11, pending establishment of radionuclide specific values for the radionuclides of natural origin given in Table I.3.

⁶⁴ Regulatory control of construction materials incorporating radionuclides is addressed in Section 5 as an existing exposure situation.

⁶⁵ For example, specific clearance levels may be developed for metals, for rubble from buildings and waste for disposal in landfill sites.

I.14. For clearance of radioactive material containing more than one radionuclide of artificial origin, on the basis of the levels given in Table I.2 (p. 124), the condition for clearance is that the sum of the activity concentrations for individual radionuclides is less than the derived clearance level for the mixture (X_m), determined as follows:

$$X_m = \frac{1}{\sum_{i=1}^n \frac{f(i)}{X(i)}} \quad (I.2)$$

where

$f(i)$ is the fraction of activity concentration of radionuclide i in the mixture;
 $X(i)$ is the applicable level for radionuclide i as given in Table I.2;

and n is the number of radionuclides present.

I.15. For clearance of bulk quantities of material containing a mixture of radionuclides of natural origin and radionuclides of artificial origin, the conditions given in paras I.12(b) and I.14 both have to be satisfied.

TABLE I.1. LEVELS FOR EXEMPTION OF MODERATE AMOUNTS OF MATERIAL WITHOUT FURTHER CONSIDERATION: EXEMPT ACTIVITY CONCENTRATIONS AND EXEMPT ACTIVITIES OF RADIONUCLIDES

Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)	Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)
H-3	1×10^6	1×10^9	Sc-45	1×10^2	1×10^7
Be-7	1×10^3	1×10^7	Sc-46	1×10^1	1×10^6
Be-10	1×10^4	1×10^6	Sc-47	1×10^2	1×10^6
C-11	1×10^1	1×10^6	Sc-48	1×10^1	1×10^5
C-14	1×10^4	1×10^7	Sc-49	1×10^3	1×10^5
N-13	1×10^2	1×10^9	Ti-44	1×10^1	1×10^5
Ne-19	1×10^2	1×10^9	Ti-45	1×10^1	1×10^6
O-15	1×10^2	1×10^9	V-47	1×10^1	1×10^5
F-18	1×10^1	1×10^6	V-48	1×10^1	1×10^5
Na-22	1×10^1	1×10^6	V-49	1×10^4	1×10^7
Na-24	1×10^1	1×10^5	Cr-48	1×10^2	1×10^6
Mg-28	1×10^1	1×10^5	Cr-49	1×10^1	1×10^6
Al-26	1×10^1	1×10^5	Cr-51	1×10^3	1×10^7
Si-31	1×10^3	1×10^6	Mn-51	1×10^1	1×10^5
Si-32	1×10^3	1×10^6	Mn-52	1×10^1	1×10^5
P-32	1×10^3	1×10^5	Mn-52m	1×10^1	1×10^5
P-33	1×10^5	1×10^8	Mn-53	1×10^4	1×10^9
S-35	1×10^5	1×10^8	Mn-54	1×10^1	1×10^6
Cl-36	1×10^4	1×10^6	Mn-56	1×10^1	1×10^5
Cl-38	1×10^1	1×10^5	Fe-52	1×10^1	1×10^6
Cl-39	1×10^1	1×10^5	Fe-55	1×10^4	1×10^6
Ar-37	1×10^6	1×10^8	Fe-59	1×10^1	1×10^6
Ar-39	1×10^7	1×10^4	Fe-60	1×10^2	1×10^5
Ar-41	1×10^2	1×10^9	Co-55	1×10^1	1×10^6
K-40	1×10^2	1×10^6	Co-56	1×10^1	1×10^5
K-42	1×10^2	1×10^6	Co-57	1×10^2	1×10^6
K-43	1×10^1	1×10^6	Co-58	1×10^1	1×10^6
K-44	1×10^1	1×10^5	Co-58m	1×10^4	1×10^7
K-45	1×10^1	1×10^5	Co-60	1×10^1	1×10^5
Ca-41	1×10^5	1×10^7	Co-60m	1×10^3	1×10^6
Ca-45	1×10^4	1×10^7	Co-61	1×10^2	1×10^6
Ca-47	1×10^1	1×10^6	Co-62m	1×10^1	1×10^5
Sc-43	1×10^1	1×10^6	Ni-56	1×10^1	1×10^6
Sc-44	1×10^1	1×10^5	Ni-57	1×10^1	1×10^6

TABLE I.1. LEVELS FOR EXEMPTION OF MODERATE AMOUNTS OF MATERIAL WITHOUT FURTHER CONSIDERATION: EXEMPT ACTIVITY CONCENTRATIONS AND EXEMPT ACTIVITIES OF RADIONUCLIDES (cont.)

Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)	Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)
Ni-59	1×10^4	1×10^8	As-72	1×10^1	1×10^5
Ni-63	1×10^5	1×10^8	As-73	1×10^3	1×10^7
Ni-65	1×10^1	1×10^6	As-74	1×10^1	1×10^6
Ni-66	1×10^4	1×10^7	As-76	1×10^2	1×10^5
Cu-60	1×10^1	1×10^5	As-77	1×10^3	1×10^6
Cu-61	1×10^1	1×10^6	As-78	1×10^1	1×10^5
Cu-64	1×10^2	1×10^6	Se-70	1×10^1	1×10^6
Cu-67	1×10^2	1×10^6	Se-73	1×10^1	1×10^6
Zn-62	1×10^2	1×10^6	Se-73m	1×10^2	1×10^6
Zn-63	1×10^1	1×10^5	Se-75	1×10^2	1×10^6
Zn-65	1×10^1	1×10^6	Se-79	1×10^4	1×10^7
Zn-69	1×10^4	1×10^6	Se-81	1×10^3	1×10^6
Zn-69m	1×10^2	1×10^6	Se-81m	1×10^3	1×10^7
Zn-71m	1×10^1	1×10^6	Se-83	1×10^1	1×10^5
Zn-72	1×10^2	1×10^6	Br-74	1×10^1	1×10^5
Ga-65	1×10^1	1×10^5	Br-74m	1×10^1	1×10^5
Ga-66	1×10^1	1×10^5	Br-75	1×10^1	1×10^6
Ga-67	1×10^2	1×10^6	Br-76	1×10^1	1×10^5
Ga-68	1×10^1	1×10^5	Br-77	1×10^2	1×10^6
Ga-70	1×10^2	1×10^6	Br-80	1×10^2	1×10^5
Ga-72	1×10^1	1×10^5	Br-80m	1×10^3	1×10^7
Ga-73	1×10^2	1×10^6	Br-82	1×10^1	1×10^6
Ge-66	1×10^1	1×10^6	Br-83	1×10^3	1×10^6
Ge-67	1×10^1	1×10^5	Br-84	1×10^1	1×10^5
Ge-68 ^b	1×10^1	1×10^5	Kr-74	1×10^2	1×10^9
Ge-69	1×10^1	1×10^6	Kr-76	1×10^2	1×10^9
Ge-71	1×10^4	1×10^8	Kr-77	1×10^2	1×10^9
Ge-75	1×10^3	1×10^6	Kr-79	1×10^3	1×10^5
Ge-77	1×10^1	1×10^5	Kr-81	1×10^4	1×10^7
Ge-78	1×10^2	1×10^6	Kr-81m	1×10^3	1×10^{10}
As-69	1×10^1	1×10^5	Kr-83m	1×10^5	1×10^{12}
As-70	1×10^1	1×10^5	Kr-85	1×10^5	1×10^4
As-71	1×10^1	1×10^6	Kr-85m	1×10^3	1×10^{10}

TABLE I.1. LEVELS FOR EXEMPTION OF MODERATE AMOUNTS OF MATERIAL WITHOUT FURTHER CONSIDERATION: EXEMPT ACTIVITY CONCENTRATIONS AND EXEMPT ACTIVITIES OF RADIONUCLIDES (cont.)

Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)	Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)
Kr-87	1×10^2	1×10^9	Y-94	1×10^1	1×10^5
Kr-88	1×10^2	1×10^9	Y-95	1×10^1	1×10^5
Rb-79	1×10^1	1×10^5	Zr-86	1×10^2	1×10^7
Rb-81	1×10^1	1×10^6	Zr-88	1×10^2	1×10^6
Rb-81m	1×10^3	1×10^7	Zr-89	1×10^1	1×10^6
Rb-82m	1×10^1	1×10^6	Zr-93 ^b	1×10^3	1×10^7
Rb-83 ^b	1×10^2	1×10^6	Zr-95	1×10^1	1×10^6
Rb-84	1×10^1	1×10^6	Zr-97 ^b	1×10^1	1×10^5
Rb-86	1×10^2	1×10^5	Nb-88	1×10^1	1×10^5
Rb-87	1×10^3	1×10^7	Nb-89	1×10^1	1×10^5
Rb-88	1×10^2	1×10^5	Nb-89m	1×10^1	1×10^5
Rb-89	1×10^2	1×10^5	Nb-90	1×10^1	1×10^5
Sr-80	1×10^3	1×10^7	Nb-93m	1×10^4	1×10^7
Sr-81	1×10^1	1×10^5	Nb-94	1×10^1	1×10^6
Sr-82 ^b	1×10^1	1×10^5	Nb-95	1×10^1	1×10^6
Sr-83	1×10^1	1×10^6	Nb-95m	1×10^2	1×10^7
Sr-85	1×10^2	1×10^6	Nb-96	1×10^1	1×10^5
Sr-85m	1×10^2	1×10^7	Nb-97	1×10^1	1×10^6
Sr-87m	1×10^2	1×10^6	Nb-98	1×10^1	1×10^5
Sr-89	1×10^3	1×10^6	Mo-90	1×10^1	1×10^6
Sr-90 ^b	1×10^2	1×10^4	Mo-93	1×10^3	1×10^8
Sr-91	1×10^1	1×10^5	Mo-93m	1×10^1	1×10^6
Sr-92	1×10^1	1×10^6	Mo-99	1×10^2	1×10^6
Y-86	1×10^1	1×10^5	Mo-101	1×10^1	1×10^6
Y-86m	1×10^2	1×10^7	Tc-93	1×10^1	1×10^6
Y-87 ^b	1×10^1	1×10^6	Tc-93m	1×10^1	1×10^6
Y-88	1×10^1	1×10^6	Tc-94	1×10^1	1×10^6
Y-90	1×10^3	1×10^5	Tc-94m	1×10^1	1×10^5
Y-90m	1×10^1	1×10^6	Tc-95	1×10^1	1×10^6
Y-91	1×10^3	1×10^6	Tc-95m	1×10^1	1×10^6
Y-91m	1×10^2	1×10^6	Tc-96	1×10^1	1×10^6
Y-92	1×10^2	1×10^5	Tc-96m	1×10^3	1×10^7
Y-93	1×10^2	1×10^5	Tc-97	1×10^3	1×10^8

TABLE I.1. LEVELS FOR EXEMPTION OF MODERATE AMOUNTS OF MATERIAL WITHOUT FURTHER CONSIDERATION: EXEMPT ACTIVITY CONCENTRATIONS AND EXEMPT ACTIVITIES OF RADIONUCLIDES (cont.)

Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)	Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)
Tc-97m	1×10^3	1×10^7	Ag-106m	1×10^1	1×10^6
Tc-98	1×10^1	1×10^6	Ag-108m	1×10^1	1×10^6
Tc-99	1×10^4	1×10^7	Ag-110m	1×10^1	1×10^6
Tc-99m	1×10^2	1×10^7	Ag-111	1×10^3	1×10^6
Tc-101	1×10^2	1×10^6	Ag-112	1×10^1	1×10^5
Tc-104	1×10^1	1×10^5	Ag-115	1×10^1	1×10^5
Ru-94	1×10^2	1×10^6	Cd-104	1×10^2	1×10^7
Ru-97	1×10^2	1×10^7	Cd-107	1×10^3	1×10^7
Ru-103	1×10^2	1×10^6	Cd-109	1×10^4	1×10^6
Ru-105	1×10^1	1×10^6	Cd-113	1×10^3	1×10^6
Ru-106 ^b	1×10^2	1×10^5	Cd-113m	1×10^3	1×10^6
Rh-99	1×10^1	1×10^6	Cd-115	1×10^2	1×10^6
Rh-99m	1×10^1	1×10^6	Cd-115m	1×10^3	1×10^6
Rh-100	1×10^1	1×10^6	Cd-117	1×10^1	1×10^6
Rh-101	1×10^2	1×10^7	Cd-117m	1×10^1	1×10^6
Rh-101m	1×10^2	1×10^7	In-109	1×10^1	1×10^6
Rh-102	1×10^1	1×10^6	In-110	1×10^1	1×10^6
Rh-102m	1×10^2	1×10^6	In-110m	1×10^1	1×10^5
Rh-103m	1×10^4	1×10^8	In-111	1×10^2	1×10^6
Rh-105	1×10^2	1×10^7	In-112	1×10^2	1×10^6
Rh-106m	1×10^1	1×10^5	In-113m	1×10^2	1×10^6
Rh-107	1×10^2	1×10^6	In-114	1×10^3	1×10^5
Pd-100	1×10^2	1×10^7	In-114m	1×10^2	1×10^6
Pd-101	1×10^2	1×10^6	In-115	1×10^3	1×10^5
Pd-103	1×10^3	1×10^8	In-115m	1×10^2	1×10^6
Pd-107	1×10^5	1×10^8	In-116m	1×10^1	1×10^5
Pd-109	1×10^3	1×10^6	In-117	1×10^1	1×10^6
Ag-102	1×10^1	1×10^5	In-117m	1×10^2	1×10^6
Ag-103	1×10^1	1×10^6	In-119m	1×10^2	1×10^5
Ag-104	1×10^1	1×10^6	Sn-110	1×10^2	1×10^7
Ag-104m	1×10^1	1×10^6	Sn-111	1×10^2	1×10^6
Ag-105	1×10^2	1×10^6	Sn-113	1×10^3	1×10^7
Ag-106	1×10^1	1×10^6	Sn-117m	1×10^2	1×10^6

TABLE I.1. LEVELS FOR EXEMPTION OF MODERATE AMOUNTS OF MATERIAL WITHOUT FURTHER CONSIDERATION: EXEMPT ACTIVITY CONCENTRATIONS AND EXEMPT ACTIVITIES OF RADIONUCLIDES (cont.)

Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)	Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)
Sn-119m	1×10^3	1×10^7	Te-123m	1×10^2	1×10^7
Sn-121	1×10^5	1×10^7	Te-125m	1×10^3	1×10^7
Sn-121m ^b	1×10^3	1×10^7	Te-127	1×10^3	1×10^6
Sn-123	1×10^3	1×10^6	Te-127m	1×10^3	1×10^7
Sn-123m	1×10^2	1×10^6	Te-129	1×10^2	1×10^6
Sn-125	1×10^2	1×10^5	Te-129m	1×10^3	1×10^6
Sn-126 ^b	1×10^1	1×10^5	Te-131	1×10^2	1×10^5
Sn-127	1×10^1	1×10^6	Te-131m	1×10^1	1×10^6
Sn-128	1×10^1	1×10^6	Te-132	1×10^2	1×10^7
Sb-115	1×10^1	1×10^6	Te-133	1×10^1	1×10^5
Sb-116	1×10^1	1×10^6	Te-133m	1×10^1	1×10^5
Sb-116m	1×10^1	1×10^5	Te-134	1×10^1	1×10^6
Sb-117	1×10^2	1×10^7	I-120	1×10^1	1×10^5
Sb-118m	1×10^1	1×10^6	I-120m	1×10^1	1×10^5
Sb-119	1×10^3	1×10^7	I-121	1×10^2	1×10^6
Sb-120	1×10^2	1×10^6	I-123	1×10^2	1×10^7
Sb-120m	1×10^1	1×10^6	I-124	1×10^1	1×10^6
Sb-122	1×10^2	1×10^4	I-125	1×10^3	1×10^6
Sb-124	1×10^1	1×10^6	I-126	1×10^2	1×10^6
Sb-124m	1×10^2	1×10^6	I-128	1×10^2	1×10^5
Sb-125	1×10^2	1×10^6	I-129	1×10^2	1×10^5
Sb-126	1×10^1	1×10^5	I-130	1×10^1	1×10^6
Sb-126m	1×10^1	1×10^5	I-131	1×10^2	1×10^6
Sb-127	1×10^1	1×10^6	I-132	1×10^1	1×10^5
Sb-128	1×10^1	1×10^5	I-132m	1×10^2	1×10^6
Sb-128m	1×10^1	1×10^5	I-133	1×10^1	1×10^6
Sb-129	1×10^1	1×10^6	I-134	1×10^1	1×10^5
Sb-130	1×10^1	1×10^5	I-135	1×10^1	1×10^6
Sb-131	1×10^1	1×10^6	Xe-120	1×10^2	1×10^9
Te-116	1×10^2	1×10^7	Xe-121	1×10^2	1×10^9
Te-121	1×10^1	1×10^6	Xe-122 ^b	1×10^2	1×10^9
Te-121m	1×10^2	1×10^6	Xe-123	1×10^2	1×10^9
Te-123	1×10^3	1×10^6	Xe-125	1×10^3	1×10^9

TABLE I.1. LEVELS FOR EXEMPTION OF MODERATE AMOUNTS OF MATERIAL WITHOUT FURTHER CONSIDERATION: EXEMPT ACTIVITY CONCENTRATIONS AND EXEMPT ACTIVITIES OF RADIONUCLIDES (cont.)

Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)	Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)
Xe-127	1×10^3	1×10^5	La-131	1×10^1	1×10^6
Xe-129m	1×10^3	1×10^4	La-132	1×10^1	1×10^6
Xe-131m	1×10^4	1×10^4	La-135	1×10^3	1×10^7
Xe-133m	1×10^3	1×10^4	La-137	1×10^3	1×10^7
Xe-133	1×10^3	1×10^4	La-138	1×10^1	1×10^6
Xe-135	1×10^3	1×10^{10}	La-140	1×10^1	1×10^5
Xe-135m	1×10^2	1×10^9	La-141	1×10^2	1×10^5
Xe-138	1×10^2	1×10^9	La-142	1×10^1	1×10^5
Cs-125	1×10^1	1×10^4	La-143	1×10^2	1×10^5
Cs-127	1×10^2	1×10^5	Ce-134	1×10^3	1×10^7
Cs-129	1×10^2	1×10^5	Ce-135	1×10^1	1×10^6
Cs-130	1×10^2	1×10^6	Ce-137	1×10^3	1×10^7
Cs-131	1×10^3	1×10^6	Ce-137m	1×10^3	1×10^6
Cs-132	1×10^1	1×10^5	Ce-139	1×10^2	1×10^6
Cs-134m	1×10^3	1×10^5	Ce-141	1×10^2	1×10^7
Cs-134	1×10^1	1×10^4	Ce-143	1×10^2	1×10^6
Cs-135	1×10^4	1×10^7	Ce-144 ^b	1×10^2	1×10^5
Cs-135m	1×10^1	1×10^6	Pr-136	1×10^1	1×10^5
Cs-136	1×10^1	1×10^5	Pr-137	1×10^2	1×10^6
Cs-137 ^b	1×10^1	1×10^4	Pr-138m	1×10^1	1×10^6
Cs-138	1×10^1	1×10^4	Pr-139	1×10^2	1×10^7
Ba-126	1×10^2	1×10^7	Pr-142	1×10^2	1×10^5
Ba-128	1×10^2	1×10^7	Pr-142m	1×10^7	1×10^9
Ba-131	1×10^2	1×10^6	Pr-143	1×10^4	1×10^6
Ba-131m	1×10^2	1×10^7	Pr-144	1×10^2	1×10^5
Ba-133	1×10^2	1×10^6	Pr-145	1×10^3	1×10^5
Ba-133m	1×10^2	1×10^6	Pr-147	1×10^1	1×10^5
Ba-135m	1×10^2	1×10^6	Nd-136	1×10^2	1×10^6
Ba-137m	1×10^1	1×10^6	Nd-138	1×10^3	1×10^7
Ba-139	1×10^2	1×10^5	Nd-139	1×10^2	1×10^6
Ba-140 ^b	1×10^1	1×10^5	Nd-139m	1×10^1	1×10^6
Ba-141	1×10^2	1×10^5	Nd-141	1×10^2	1×10^7
Ba-142	1×10^2	1×10^6	Nd-147	1×10^2	1×10^6

TABLE I.1. LEVELS FOR EXEMPTION OF MODERATE AMOUNTS OF MATERIAL WITHOUT FURTHER CONSIDERATION: EXEMPT ACTIVITY CONCENTRATIONS AND EXEMPT ACTIVITIES OF RADIONUCLIDES (cont.)

Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)	Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)
Nd-149	1×10^2	1×10^6	Eu-155	1×10^2	1×10^7
Nd-151	1×10^1	1×10^5	Eu-156	1×10^1	1×10^6
Pm-141	1×10^1	1×10^5	Eu-157	1×10^2	1×10^6
Pm-143	1×10^2	1×10^6	Eu-158	1×10^1	1×10^5
Pm-144	1×10^1	1×10^6	Gd-145	1×10^1	1×10^5
Pm-145	1×10^3	1×10^7	Gd-146 ^b	1×10^1	1×10^6
Pm-146	1×10^1	1×10^6	Gd-147	1×10^1	1×10^6
Pm-147	1×10^4	1×10^7	Gd-148	1×10^1	1×10^4
Pm-148	1×10^1	1×10^5	Gd-149	1×10^2	1×10^6
Pm-148m	1×10^1	1×10^6	Gd-151	1×10^2	1×10^7
Pm-149	1×10^3	1×10^6	Gd-152	1×10^1	1×10^4
Pm-150	1×10^1	1×10^5	Gd-153	1×10^2	1×10^7
Pm-151	1×10^2	1×10^6	Gd-159	1×10^3	1×10^6
Sm-141	1×10^1	1×10^5	Tb-147	1×10^1	1×10^6
Sm-141m	1×10^1	1×10^6	Tb-149	1×10^1	1×10^6
Sm-142	1×10^2	1×10^7	Tb-150	1×10^1	1×10^6
Sm-145	1×10^2	1×10^7	Tb-151	1×10^1	1×10^6
Sm-146	1×10^1	1×10^5	Tb-153	1×10^2	1×10^7
Sm-147	1×10^1	1×10^4	Tb-154	1×10^1	1×10^6
Sm-151	1×10^4	1×10^8	Tb-155	1×10^2	1×10^7
Sm-153	1×10^2	1×10^6	Tb-156	1×10^1	1×10^6
Sm-155	1×10^2	1×10^6	Tb-156m (24.4 h)	1×10^3	1×10^7
Sm-156	1×10^2	1×10^6	Tb-156m' (5 h)	1×10^4	1×10^7
Eu-145	1×10^1	1×10^6	Tb-157	1×10^4	1×10^7
Eu-146	1×10^1	1×10^6	Tb-158	1×10^1	1×10^6
Eu-147	1×10^2	1×10^6	Tb-160	1×10^1	1×10^6
Eu-148	1×10^1	1×10^6	Tb-161	1×10^3	1×10^6
Eu-149	1×10^2	1×10^7	Dy-155	1×10^1	1×10^6
Eu-150	1×10^1	1×10^6	Dy-157	1×10^2	1×10^6
Eu-150m	1×10^3	1×10^6	Dy-159	1×10^3	1×10^7
Eu-152	1×10^1	1×10^6	Dy-165	1×10^3	1×10^6
Eu-152m	1×10^2	1×10^6	Dy-166	1×10^3	1×10^6
Eu-154	1×10^1	1×10^6	Ho-155	1×10^2	1×10^6

TABLE I.1. LEVELS FOR EXEMPTION OF MODERATE AMOUNTS OF MATERIAL WITHOUT FURTHER CONSIDERATION: EXEMPT ACTIVITY CONCENTRATIONS AND EXEMPT ACTIVITIES OF RADIONUCLIDES (cont.)

Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)	Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)
Ho-157	1×10^2	1×10^6	Lu-172	1×10^1	1×10^6
Ho-159	1×10^2	1×10^6	Lu-173	1×10^2	1×10^7
Ho-161	1×10^2	1×10^7	Lu-174	1×10^2	1×10^7
Ho-162	1×10^2	1×10^7	Lu-174m	1×10^2	1×10^7
Ho-162m	1×10^1	1×10^6	Lu-176	1×10^2	1×10^6
Ho-164	1×10^3	1×10^6	Lu-176m	1×10^3	1×10^6
Ho-164m	1×10^3	1×10^7	Lu-177	1×10^3	1×10^7
Ho-166	1×10^3	1×10^5	Lu-177m	1×10^1	1×10^6
Ho-166m	1×10^1	1×10^6	Lu-178	1×10^2	1×10^5
Ho-167	1×10^2	1×10^6	Lu-178m	1×10^1	1×10^5
Er-161	1×10^1	1×10^6	Lu-179	1×10^3	1×10^6
Er-165	1×10^3	1×10^7	Hf-170	1×10^2	1×10^6
Er-169	1×10^4	1×10^7	Hf-172 ^b	1×10^1	1×10^6
Er-171	1×10^2	1×10^6	Hf-173	1×10^2	1×10^6
Er-172	1×10^2	1×10^6	Hf-175	1×10^2	1×10^6
Tm-162	1×10^1	1×10^6	Hf-177m	1×10^1	1×10^5
Tm-166	1×10^1	1×10^6	Hf-178m	1×10^1	1×10^6
Tm-167	1×10^2	1×10^6	Hf-179m	1×10^1	1×10^6
Tm-170	1×10^3	1×10^6	Hf-180m	1×10^1	1×10^6
Tm-171	1×10^4	1×10^8	Hf-181	1×10^1	1×10^6
Tm-172	1×10^2	1×10^6	Hf-182	1×10^2	1×10^6
Tm-173	1×10^2	1×10^6	Hf-182m	1×10^1	1×10^6
Tm-175	1×10^1	1×10^6	Hf-183	1×10^1	1×10^6
Yb-162	1×10^2	1×10^7	Hf-184	1×10^2	1×10^6
Yb-166	1×10^2	1×10^7	Ta-172	1×10^1	1×10^6
Yb-167	1×10^2	1×10^6	Ta-173	1×10^1	1×10^6
Yb-169	1×10^2	1×10^7	Ta-174	1×10^1	1×10^6
Yb-175	1×10^3	1×10^7	Ta-175	1×10^1	1×10^6
Yb-177	1×10^2	1×10^6	Ta-176	1×10^1	1×10^6
Yb-178	1×10^3	1×10^6	Ta-177	1×10^2	1×10^7
Lu-169	1×10^1	1×10^6	Ta-178	1×10^1	1×10^6
Lu-170	1×10^1	1×10^6	Ta-179	1×10^3	1×10^7
Lu-171	1×10^1	1×10^6	Ta-180	1×10^1	1×10^6

TABLE I.1. LEVELS FOR EXEMPTION OF MODERATE AMOUNTS OF MATERIAL WITHOUT FURTHER CONSIDERATION: EXEMPT ACTIVITY CONCENTRATIONS AND EXEMPT ACTIVITIES OF RADIONUCLIDES (cont.)

Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)	Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)
Ta-180m	1×10^3	1×10^7	Os-191	1×10^2	1×10^7
Ta-182	1×10^1	1×10^4	Os-191m	1×10^3	1×10^7
Ta-182m	1×10^2	1×10^6	Os-193	1×10^2	1×10^6
Ta-183	1×10^2	1×10^6	Os-194 ^b	1×10^2	1×10^5
Ta-184	1×10^1	1×10^6	Ir-182	1×10^1	1×10^5
Ta-185	1×10^2	1×10^5	Ir-184	1×10^1	1×10^6
Ta-186	1×10^1	1×10^5	Ir-185	1×10^1	1×10^6
W-176	1×10^2	1×10^6	Ir-186	1×10^1	1×10^6
W-177	1×10^1	1×10^6	Ir-186m	1×10^1	1×10^6
W-178 ^b	1×10^1	1×10^6	Ir-187	1×10^2	1×10^6
W-179	1×10^2	1×10^7	Ir-188	1×10^1	1×10^6
W-181	1×10^3	1×10^7	Ir-189 ^b	1×10^2	1×10^7
W-185	1×10^4	1×10^7	Ir-190	1×10^1	1×10^6
W-187	1×10^2	1×10^6	Ir-190m (3.1 h)	1×10^1	1×10^6
W-188 ^b	1×10^2	1×10^5	Ir-190m' (1.2 h)	1×10^4	1×10^7
Re-177	1×10^1	1×10^6	Ir-192	1×10^1	1×10^4
Re-178	1×10^1	1×10^6	Ir-192m	1×10^2	1×10^7
Re-181	1×10^1	1×10^6	Ir-193m	1×10^4	1×10^7
Re-182	1×10^1	1×10^6	Ir-194	1×10^2	1×10^5
Re-182m	1×10^1	1×10^6	Ir-194m	1×10^1	1×10^6
Re-184	1×10^1	1×10^6	Ir-195	1×10^2	1×10^6
Re-184m	1×10^2	1×10^6	Ir-195m	1×10^2	1×10^6
Re-186	1×10^3	1×10^6	Pt-186	1×10^1	1×10^6
Re-186m	1×10^3	1×10^7	Pt-188 ^b	1×10^1	1×10^6
Re-187	1×10^6	1×10^9	Pt-189	1×10^2	1×10^6
Re-188	1×10^2	1×10^5	Pt-191	1×10^2	1×10^6
Re-188m	1×10^2	1×10^7	Pt-193	1×10^4	1×10^7
Re-189 ^b	1×10^2	1×10^6	Pt-193m	1×10^3	1×10^7
Os-180	1×10^2	1×10^7	Pt-195m	1×10^2	1×10^6
Os-181	1×10^1	1×10^6	Pt-197	1×10^3	1×10^6
Os-182	1×10^2	1×10^6	Pt-197m	1×10^2	1×10^6
Os-185	1×10^1	1×10^6	Pt-199	1×10^2	1×10^6
Os-189m	1×10^4	1×10^7	Pt-200	1×10^2	1×10^6

TABLE I.1. LEVELS FOR EXEMPTION OF MODERATE AMOUNTS OF MATERIAL WITHOUT FURTHER CONSIDERATION: EXEMPT ACTIVITY CONCENTRATIONS AND EXEMPT ACTIVITIES OF RADIONUCLIDES (cont.)

Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)	Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)
Au-193	1×10^2	1×10^7	Pb-201	1×10^1	1×10^6
Au-194	1×10^1	1×10^6	Pb-202	1×10^3	1×10^6
Au-195	1×10^2	1×10^7	Pb-202m	1×10^1	1×10^6
Au-198	1×10^2	1×10^6	Pb-203	1×10^2	1×10^6
Au-198m	1×10^1	1×10^6	Pb-205	1×10^4	1×10^7
Au-199	1×10^2	1×10^6	Pb-209	1×10^5	1×10^6
Au-200	1×10^2	1×10^5	Pb-210 ^b	1×10^1	1×10^4
Au-200m	1×10^1	1×10^6	Pb-211	1×10^2	1×10^6
Au-201	1×10^2	1×10^6	Pb-212 ^b	1×10^1	1×10^5
Hg-193	1×10^2	1×10^6	Pb-214	1×10^2	1×10^6
Hg-193m	1×10^1	1×10^6	Bi-200	1×10^1	1×10^6
Hg-194 ^b	1×10^1	1×10^6	Bi-201	1×10^1	1×10^6
Hg-195	1×10^2	1×10^6	Bi-202	1×10^1	1×10^6
Hg-195m ^b	1×10^2	1×10^6	Bi-203	1×10^1	1×10^6
Hg-197	1×10^2	1×10^7	Bi-205	1×10^1	1×10^6
Hg-197m	1×10^2	1×10^6	Bi-206	1×10^1	1×10^5
Hg-199m	1×10^2	1×10^6	Bi-207	1×10^1	1×10^6
Hg-203	1×10^2	1×10^5	Bi-210	1×10^3	1×10^6
Tl-194	1×10^1	1×10^6	Bi-210m ^b	1×10^1	1×10^5
Tl-194m	1×10^1	1×10^6	Bi-212 ^b	1×10^1	1×10^5
Tl-195	1×10^1	1×10^6	Bi-213	1×10^2	1×10^6
Tl-197	1×10^2	1×10^6	Bi-214	1×10^1	1×10^5
Tl-198	1×10^1	1×10^6	Po-203	1×10^1	1×10^6
Tl-198m	1×10^1	1×10^6	Po-205	1×10^1	1×10^6
Tl-199	1×10^2	1×10^6	Po-206	1×10^1	1×10^6
Tl-200	1×10^1	1×10^6	Po-207	1×10^1	1×10^6
Tl-201	1×10^2	1×10^6	Po-208	1×10^1	1×10^4
Tl-202	1×10^2	1×10^6	Po-209	1×10^1	1×10^4
Tl-204	1×10^4	1×10^4	Po-210	1×10^1	1×10^4
Pb-195m	1×10^1	1×10^6	At-207	1×10^1	1×10^6
Pb-198	1×10^2	1×10^6	At-211	1×10^3	1×10^7
Pb-199	1×10^1	1×10^6	Fr-222	1×10^3	1×10^5
Pb-200	1×10^2	1×10^6	Fr-223	1×10^2	1×10^6

TABLE I.1. LEVELS FOR EXEMPTION OF MODERATE AMOUNTS OF MATERIAL WITHOUT FURTHER CONSIDERATION: EXEMPT ACTIVITY CONCENTRATIONS AND EXEMPT ACTIVITIES OF RADIONUCLIDES (cont.)

Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)	Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)
Rn-220 ^b	1×10^4	1×10^7	U-235 ^b	1×10^1	1×10^4
Rn-222 ^b	1×10^1	1×10^8	U-236	1×10^1	1×10^4
Ra-223 ^b	1×10^2	1×10^5	U-237	1×10^2	1×10^6
Ra-224 ^b	1×10^1	1×10^5	U-238 ^b	1×10^1	1×10^4
Ra-225	1×10^2	1×10^5	U-239	1×10^2	1×10^6
Ra-226 ^b	1×10^1	1×10^4	U-240	1×10^3	1×10^7
Ra-227	1×10^2	1×10^6	U-240 ^b	1×10^1	1×10^6
Ra-228 ^b	1×10^1	1×10^5	Np-232	1×10^1	1×10^6
Ac-224	1×10^2	1×10^6	Np-233	1×10^2	1×10^7
Ac-225 ^b	1×10^1	1×10^4	Np-234	1×10^1	1×10^6
Ac-226	1×10^2	1×10^5	Np-235	1×10^3	1×10^7
Ac-227 ^b	1×10^{-1}	1×10^3	Np-236	1×10^2	1×10^5
Ac-228	1×10^1	1×10^6	Np-236m	1×10^3	1×10^7
Th-226 ^b	1×10^3	1×10^7	Np-237 ^b	1×10^0	1×10^3
Th-227	1×10^1	1×10^4	Np-238	1×10^2	1×10^6
Th-228 ^b	1×10^0	1×10^4	Np-239	1×10^2	1×10^7
Th-229 ^b	1×10^0	1×10^3	Np-240	1×10^1	1×10^6
Th-230	1×10^0	1×10^4	Pu-234	1×10^2	1×10^7
Th-231	1×10^3	1×10^7	Pu-235	1×10^2	1×10^7
Th-232	1×10^1	1×10^4	Pu-236	1×10^1	1×10^4
Th-234 ^b	1×10^3	1×10^5	Pu-237	1×10^3	1×10^7
Pa-227	1×10^1	1×10^6	Pu-238	1×10^0	1×10^4
Pa-228	1×10^1	1×10^6	Pu-239	1×10^0	1×10^4
Pa-230	1×10^1	1×10^6	Pu-240	1×10^0	1×10^3
Pa-231	1×10^0	1×10^3	Pu-241	1×10^2	1×10^5
Pa-232	1×10^1	1×10^6	Pu-242	1×10^0	1×10^4
Pa-233	1×10^2	1×10^7	Pu-243	1×10^3	1×10^7
Pa-234	1×10^1	1×10^6	Pu-244	1×10^0	1×10^4
U-230 ^b	1×10^1	1×10^5	Pu-245	1×10^2	1×10^6
U-231	1×10^2	1×10^7	Pu-246	1×10^2	1×10^6
U-232 ^b	1×10^0	1×10^3	Am-237	1×10^2	1×10^6
U-233	1×10^1	1×10^4	Am-238	1×10^1	1×10^6
U-234	1×10^1	1×10^4	Am-239	1×10^2	1×10^6

TABLE I.1. LEVELS FOR EXEMPTION OF MODERATE AMOUNTS OF MATERIAL WITHOUT FURTHER CONSIDERATION: EXEMPT ACTIVITY CONCENTRATIONS AND EXEMPT ACTIVITIES OF RADIONUCLIDES (cont.)

Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)	Radionuclide ^a	Activity concentration (Bq/g)	Activity (Bq)
Am-240	1×10^1	1×10^6	Bk-247	1×10^0	1×10^4
Am-241	1×10^0	1×10^4	Bk-249	1×10^3	1×10^6
Am-242	1×10^3	1×10^6	Bk-250	1×10^1	1×10^6
Am-242m ^b	1×10^0	1×10^4	Cf-244	1×10^4	1×10^7
Am-243 ^b	1×10^0	1×10^3	Cf-246	1×10^3	1×10^6
Am-244	1×10^1	1×10^6	Cf-248	1×10^1	1×10^4
Am-244m	1×10^4	1×10^7	Cf-249	1×10^0	1×10^3
Am-245	1×10^3	1×10^6	Cf-250	1×10^1	1×10^4
Am-246	1×10^1	1×10^5	Cf-251	1×10^0	1×10^3
Am-246m	1×10^1	1×10^6	Cf-252	1×10^1	1×10^4
Cm-238	1×10^2	1×10^7	Cf-253	1×10^2	1×10^5
Cm-240	1×10^2	1×10^5	Cf-254	1×10^0	1×10^3
Cm-241	1×10^2	1×10^6	Es-250	1×10^2	1×10^6
Cm-242	1×10^2	1×10^5	Es-251	1×10^2	1×10^7
Cm-243	1×10^0	1×10^4	Es-253	1×10^2	1×10^5
Cm-244	1×10^1	1×10^4	Es-254	1×10^1	1×10^4
Cm-245	1×10^0	1×10^3	Es-254m	1×10^2	1×10^6
Cm-246	1×10^0	1×10^3	Fm-252	1×10^3	1×10^6
Cm-247	1×10^0	1×10^4	Fm-253	1×10^2	1×10^6
Cm-248	1×10^0	1×10^3	Fm-254	1×10^4	1×10^7
Cm-249	1×10^3	1×10^6	Fm-255	1×10^3	1×10^6
Cm-250	1×10^{-1}	1×10^3	Fm-257	1×10^1	1×10^5
Bk-245	1×10^2	1×10^6	Md-257	1×10^2	1×10^7
Bk-246	1×10^1	1×10^6	Md-258	1×10^2	1×10^5

^a m and m' denote metastable states of the radionuclide. The metastable state m' is of higher energy than the metastable state m.

^b Parent radionuclides and their progeny whose dose contributions are taken into account in the dose calculations (thus requiring only the exemption level of the parent radionuclide to be considered) are listed here:

Ge-68	Ga-68	Y-87	Sr-87m
Rb-83	Kr-83m	Zr-93	Nb-93m
Sr-82	Rb-82	Zr-97	Nb-97
Sr-90	Y-90	Ru-106	Rh-106

Ag-108m	Ag-108	Ra-226	Rn-222, Po-218, Pb-214,
Sn-121m	Sn-121 (0.776)		Bi-214, Po-214, Pb-210,
Sn-126	Sb-126m		Bi-210, Po-210
Xe-122	I-122	Ra-228	Ac-228
Cs-137	Ba-137m	Ac-225	Fr-221, At-217, Bi-213,
Ba-140	La-140		Po-213 (0.978),
Ce-134	La-134		Tl-209 (0.0216),
Ce-144	Pr-144		Pb-209 (0.978)
Gd-146	Eu-146	Ac-227	Fr-223 (0.0138)
Hf-172	Lu-172	Th-226	Ra-222, Rn-218, Po-214
W-178	Ta-178	Th-228	Ra-224, Rn-220, Po-216,
W-188	Re-188		Pb-212, Bi-212, Tl-208 (0.36),
Re-189	Os-189m (0.241)		Po-212 (0.64)
Ir-189	Os-189m	Th-229	Ra-225, Ac-225, Fr-221,
Pt-188	Ir-188		At-217, Bi-213, Po-213,
Hg-194	Au-194	Th-234	Pb-209
Hg-195m	Hg-195 (0.542)	U-230	Pa-234m
Pb-210	Bi-210, Po-210		Th-226, Ra-222, Rn-218,
Pb-212	Bi-212, Tl-208 (0.36), Po-212 (0.64)	U-232	Po-214
Bi-210m	Tl-206		Th-228, Ra-224, Rn-220,
Bi-212	Tl-208 (0.36), Po-212 (0.64)	U-235	Po-216, Pb-212, Bi-212,
Rn-220	Po-216	U-238	Tl-208 (0.36), Po-212 (0.64)
Rn-222	Po-218, Pb-214, Bi-214, Po-214	U-240	Th-234, Pa-234m
Ra-223	Rn-219, Po-215, Pb-211, Bi-211, Tl-207	Np-237	Np-240m
Ra-224	Rn-220, Po-216, Pb-212, Bi-212, Tl-208 (0.36), Po-212 (0.64)	Am-242m	Pa-233
		Am-243	Am-242
			Np-239

Note: The exemption values (activity concentrations) presented in this table have been calculated on the basis of scenarios involving a moderate amount of material: “The calculated values apply to practices involving small scale usage of activity where the quantities involved are at the most of the order of a tonne” (see Ref. [25]). The regulatory body will need to establish the amounts for which the concentration values in this table may be applied, bearing in mind that for many radionuclides, in particular those for which there is no corresponding value given in Table I.2, a restriction on the amount is not meaningful. The exemption levels set out in this table and the exemption and clearance levels set out in Table I.2 are subject to the following considerations: (a) they were derived using a conservative model based on (i) the criteria of paras I.2 and I.11, respectively, and (ii) a series of limiting (bounding) scenarios for use and disposal (see Refs [25, 26] in the case of this table and Ref. [27] in the case of Table I.2); (b) if there is more than one radionuclide, the derived exemption level or derived clearance level for the mixture is determined as specified in paras I.7 and I.14.

TABLE I.2. LEVELS FOR EXEMPTION OF BULK AMOUNTS OF SOLID MATERIAL WITHOUT FURTHER CONSIDERATION AND FOR CLEARANCE OF SOLID MATERIAL WITHOUT FURTHER CONSIDERATION: ACTIVITY CONCENTRATIONS OF RADIONUCLIDES OF ARTIFICIAL ORIGIN

Radionuclide	Activity concentration (Bq/g)	Radionuclide	Activity concentration (Bq/g)
H-3	100	Co-58	1
Be-7	10	Co-58m	10 000
C-14	1	Co-60	0.1
F-18	10	Co-60m	1 000
Na-22	0.1	Co-61	100
Na-24	1	Co-62m	10
Si-31	1 000	Ni-59	100
P-32	1 000	Ni-63	100
P-33	1 000	Ni-65	10
S-35	100	Cu-64	100
Cl-36	1	Zn-65	0.1
Cl-38	10	Zn-69	1 000
K-42	100	Zn-69m ^a	10
K-43	10	Ga-72	10
Ca-45	100	Ge-71	10 000
Ca-47	10	As-73	1 000
Sc-46	0.1	As-74	10
Sc-47	100	As-76	10
Sc-48	1	As-77	1 000
V-48	1	Se-75	1
Cr-51	100	Br-82	1
Mn-51	10	Rb-86	100
Mn-52	1	Sr-85	1
Mn-52m	10	Sr-85m	100
Mn-53	100	Sr-87m	100
Mn-54	0.1	Sr-89	1 000
Mn-56	10	Sr-90 ^a	1
Fe-52 ^a	10	Sr-91 ^a	10
Fe-55	1 000	Sr-92	10
Fe-59	1	Y-90	1 000
Co-55	10	Y-91	100
Co-56	0.1	Y-91m	100
Co-57	1	Y-92	100

TABLE I.2. LEVELS FOR EXEMPTION OF BULK AMOUNTS OF SOLID MATERIAL WITHOUT FURTHER CONSIDERATION AND FOR CLEARANCE OF SOLID MATERIAL WITHOUT FURTHER CONSIDERATION: ACTIVITY CONCENTRATIONS OF RADIONUCLIDES OF ARTIFICIAL ORIGIN (cont.)

Radionuclide	Activity concentration (Bq/g)	Radionuclide	Activity concentration (Bq/g)
Y-93	100	In-111	10
Zr-93	10	In-113m	100
Zr-95 ^a	1	In-114m ^a	10
Zr-97 ^a	10	In-115m	100
Nb-93m	10	Sn-113 ^a	1
Nb-94	0.1	Sn-125	10
Nb-95	1	Sb-122	10
Nb-97 ^a	10	Sb-124	1
Nb-98	10	Sb-125 ^a	0.1
Mo-90	10	Te-123m	1
Mo-93	10	Te-125m	1 000
Mo-99 ^a	10	Te-127	1 000
Mo-101 ^a	10	Te-127m ^a	10
Tc-96	1	Te-129	100
Tc-96m	1 000	Te-129m ^a	10
Tc-97	10	Te-131	100
Tc-97m	100	Te-131m ^a	10
Tc-99	1	Te-132 ^a	1
Tc-99m	100	Te-133	10
Ru-97	10	Te-133m	10
Ru-103 ^a	1	Te-134	10
Ru-105 ^a	10	I-123	100
Ru-106 ^a	0.1	I-125	100
Rh-103m	10 000	I-126	10
Rh-105	100	I-129	0.01
Pd-103 ^a	1 000	I-130	10
Pd-109 ^a	100	I-131	10
Ag-105	1	I-132	10
Ag-110m ^a	0.1	I-133	10
Ag-111	100	I-134	10
Cd-109 ^a	1	I-135	10
Cd-115 ^a	10	Cs-129	10
Cd-115m ^a	100	Cs-131	1 000

TABLE I.2. LEVELS FOR EXEMPTION OF BULK AMOUNTS OF SOLID MATERIAL WITHOUT FURTHER CONSIDERATION AND FOR CLEARANCE OF SOLID MATERIAL WITHOUT FURTHER CONSIDERATION: ACTIVITY CONCENTRATIONS OF RADIONUCLIDES OF ARTIFICIAL ORIGIN (cont.)

Radionuclide	Activity concentration (Bq/g)	Radionuclide	Activity concentration (Bq/g)
Cs-132	10	Er-171	100
Cs-134	0.1	Tm-170	100
Cs-134m	1 000	Tm-171	1 000
Cs-135	100	Yb-175	100
Cs-136	1	Lu-177	100
Cs-137 ^a	0.1	Hf-181	1
Cs-138	10	Ta-182	0.1
Ba-131	10	W-181	10
Ba-140	1	W-185	1 000
La-140	1	W-187	10
Ce-139	1	Re-186	1 000
Ce-141	100	Re-188	100
Ce-143	10	Os-185	1
Ce-144 ^a	10	Os-191	100
Pr-142	100	Os-191m	1 000
Pr-143	1 000	Os-193	100
Nd-147	100	Ir-190	1
Nd-149	100	Ir-192	1
Pm-147	1 000	Ir-194	100
Pm-149	1 000	Pt-191	10
Sm-151	1 000	Pt-193m	1 000
Sm-153	100	Pt-197	1 000
Eu-152	0.1	Pt-197m	100
Eu-152m	100	Au-198	10
Eu-154	0.1	Au-199	100
Eu-155	1	Hg-197	100
Gd-153	10	Hg-197m	100
Gd-159	100	Hg-203	10
Tb-160	1	Tl-200	10
Dy-165	1 000	Tl-201	100
Dy-166	100	Tl-202	10
Ho-166	100	Tl-204	1
Er-169	1 000	Pb-203	10

TABLE I.2. LEVELS FOR EXEMPTION OF BULK AMOUNTS OF SOLID MATERIAL WITHOUT FURTHER CONSIDERATION AND FOR CLEARANCE OF SOLID MATERIAL WITHOUT FURTHER CONSIDERATION: ACTIVITY CONCENTRATIONS OF RADIONUCLIDES OF ARTIFICIAL ORIGIN (cont.)

Radionuclide	Activity concentration (Bq/g)	Radionuclide	Activity concentration (Bq/g)
Bi-206	1	Pu-241	10
Bi-207	0.1	Pu-242	0.1
Po-203	10	Pu-243	1 000
Po-205	10	Pu-244 ^a	0.1
Po-207	10	Am-241	0.1
At-211	1 000	Am-242	1 000
Ra-225	10	Am-242m ^a	0.1
Ra-227	100	Am-243 ^a	0.1
Th-226	1 000	Cm-242	10
Th-229	0.1	Cm-243	1
Pa-230	10	Cm-244	1
Pa-233	10	Cm-245	0.1
U-230	10	Cm-246	0.1
U-231	100	Cm-247 ^a	0.1
U-232 ^a	0.1	Cm-248	0.1
U-233	1	Bk-249	100
U-236	10	Cf-246	1 000
U-237	100	Cf-248	1
U-239	100	Cf-249	0.1
U-240 ^a	100	Cf-250	1
Np-237 ^a	1	Cf-251	0.1
Np-239	100	Cf-252	1
Np-240	10	Cf-253	100
Pu-234	100	Cf-254	1
Pu-235	100	Es-253	100
Pu-236	1	Es-254 ^a	0.1
Pu-237	100	Es-254m ^a	10
Pu-238	0.1	Fm-254	10 000
Pu-239	0.1	Fm-255	100
Pu-240	0.1		

^a Parent radionuclides, and their progeny whose dose contributions are taken into account in the dose calculations (thus requiring only the exemption level of the parent radionuclide to be considered), are listed here:

Fe-52	Mn-52m	Sn-113	In-113m
Zn-69m	Zn-69	Sb-125	Te-125m
Sr-90	Y-90	Te-127m	Te-127
Sr-91	Y-91m	Te-129m	Te-129
Zr-95	Nb-95	Te-131m	Te-131
Zr-97	Nb-97m, Nb-97	Te-132	I-132
Nb-97	Nb-97m	Cs-137	Ba-137m
Mo-99	Tc-99m	Ce-144	Pr-144, Pr-144m
Mo-101	Tc-101	U-232	Th-228, Ra-224, Rn-220, Po-216, Pb-212, Bi-212, Tl-208
Ru-103	Rh-103m	U-240	Np-240m, Np-240
Ru-105	Rh-105m	Np-237	Pa-233
Ru-106	Rh-106	Pu-244	U-240, Np-240m, Np-240
Pd-103	Rh-103m	Am-242m	Np-238
Pd-109	Ag-109m	Am-243	Np-239
Ag-110m	Ag-110	Cm-247	Pu-243
Cd-109	Ag-109m	Es-254	Bk-250
Cd-115	In-115m	Es-254m	Fm-254
Cd-115m	In-115m		
In-114m	In-114		

Note: The exemption levels set out in Table I.1 (p. 111) and the exemption and clearance levels set out in this table are subject to the following considerations: (a) they were derived using a conservative model based on (i) the criteria of paras I.2 and I.11, respectively, and (ii) a series of limiting (bounding) scenarios for use and disposal (see Refs [25, 26] in the case of Table I.1 and Ref. [27] in the case of this table); (b) if there is more than one radionuclide, the derived exemption level or derived clearance level for the mixture is determined as specified in paras I.7 and I.14.

TABLE I.3. LEVELS FOR CLEARANCE OF MATERIAL: ACTIVITY CONCENTRATIONS OF RADIONUCLIDES OF NATURAL ORIGIN

Radionuclide	Activity concentration (Bq/g)
K-40	10
Each radionuclide in the uranium decay chain or the thorium decay chain	1

Schedule II

CATEGORIES FOR SEALED SOURCES USED IN COMMON PRACTICES

II.1. Table II.1 shows categories for sealed sources used in common practices, and Table II.2 shows the activity corresponding to a dangerous source (D value) for selected radionuclides.

TABLE II.1. CATEGORIES FOR SEALED SOURCES USED IN COMMON PRACTICES

Category	Ratio of activity in the source to activity that is considered dangerous ^a (A/D)	Example of sources ^b and practices
1	$A/D \geq 1000$	Radioisotope thermoelectric generators; Irradiators; Teletherapy sources; Fixed, multibeam teletherapy (‘gamma knife’) sources
2	$1000 > A/D \geq 10$	Industrial gamma radiography sources; High/medium dose rate brachytherapy sources
3	$10 > A/D \geq 1$	Fixed industrial gauges incorporating high activity sources; Well logging gauges
4	$1 > A/D \geq 0.01$	Low dose rate brachytherapy sources (except eye plaques and permanent implants); Industrial gauges not incorporating high activity sources; Bone densitometers; Static eliminators

TABLE II.1. CATEGORIES FOR SEALED SOURCES USED IN COMMON PRACTICES (cont.)

Category	Ratio of activity in the source to activity that is considered dangerous ^a (<i>A/D</i>)	Example of sources ^b and practices
5	$0.01 > A/D$ and $A > \text{level for exemption}^c$	Low dose rate brachytherapy eye plaques and permanent implant sources; X ray fluorescence devices; Electron capture devices; Mossbauer spectrometry sources; Positron emission tomography check sources

^a *A* is the activity of the radionuclide in a source and *D* is the activity of that radionuclide that is regarded as dangerous. A dangerous source is defined as one that could, if not under control, give rise to exposure sufficient to cause severe deterministic effects. Values of *D* for selected radionuclides are given in Table II.2 on the basis of the quantity of radioactive material that could give rise to severe deterministic effects for given exposure scenarios and for given dose criteria. This column of the table can, thus, be used to determine the category of a source, purely on the basis of the value of *A/D*. This may be appropriate if, for example: the practice is not known or is not listed; if sources have a short half-life and/or are unsealed; or if sources are aggregated.

^b Factors other than *A/D* have been taken into consideration in assigning these sources to a particular category [28].

^c Levels for exemption are given in Schedule I.

TABLE II.2. ACTIVITY^a CORRESPONDING TO A DANGEROUS SOURCE (D VALUE^b) FOR SELECTED RADIONUCLIDES

Radionuclide	D value (TBq)	Radionuclide	D value (TBq)
Am-241	6×10^{-2}	Mo-99	3×10^{-1}
Am-241/Be	6×10^{-2}	Ni-63	6×10^1
Au-198	2×10^{-1}	P-32	1×10^1
Cd-109	2×10^1	Pd-103	9×10^1
Cf-252	2×10^{-2}	Pm-147	4×10^1
Cm-244	5×10^{-2}	Po-210	6×10^{-2}
Co-57	7×10^{-1}	Pu-238	6×10^{-2}
Co-60	3×10^{-2}	Pu-239/Be	6×10^{-2}
Cs-137	1×10^{-1}	Ra-226	4×10^{-2}
Fe-55	8×10^2	Ru-106 (Rh-106)	3×10^{-1}
Gd-153	1×10^0	Se-75	2×10^{-1}
Ge-68	7×10^{-2}	Sr-90 (Y-90)	1×10^0
H-3	2×10^3	Tc-99m	7×10^{-1}
I-125	2×10^{-1}	Tl-204	2×10^1
I-131	2×10^{-1}	Tm-170	2×10^1
Ir-192	8×10^{-2}	Yb-169	3×10^{-1}
Kr-85	3×10^1		

^a Since this table does not state which dose criteria were used, these D values cannot be used ‘in reverse’ to derive possible doses from exposure due to sources of known activity.

^b Full details of the derivation of the D values and D values for additional radionuclides are provided in Ref. [29].

Schedule III

DOSE LIMITS FOR PLANNED EXPOSURE SITUATIONS

OCCUPATIONAL EXPOSURE

III.1. For occupational exposure of workers over the age of 18 years, the dose limits are:

- (a) An effective dose of 20 mSv per year averaged over five consecutive years⁶⁶ (100 mSv in 5 years) and of 50 mSv in any single year;
- (b) An equivalent dose to the lens of the eye of 20 mSv per year averaged over five consecutive years (100 mSv in 5 years) and of 50 mSv in any single year;
- (c) An equivalent dose to the extremities (hands and feet) or to the skin⁶⁷ of 500 mSv in a year.

Additional restrictions apply to occupational exposure for a female worker who has notified pregnancy or is breast-feeding (para. 3.114).

III.2. For occupational exposure of apprentices of 16 to 18 years of age who are being trained for employment involving radiation and for exposure of students of age 16 to 18 who use sources in the course of their studies, the dose limits are:

- (a) An effective dose of 6 mSv in a year;
- (b) An equivalent dose to the lens of the eye of 20 mSv in a year;
- (c) An equivalent dose to the extremities (hands and feet) or to the skin⁶⁷ of 150 mSv in a year.

⁶⁶ The start of the averaging period shall be coincident with the first day of the relevant annual period after the date of entry into force of these Standards, with no retrospective averaging.

⁶⁷ The equivalent dose limits for the skin apply to the average dose over 1 cm² of the most highly irradiated area of the skin. The dose to the skin also contributes to the effective dose, this contribution being the average dose to the entire skin multiplied by the tissue weighting factor for the skin.

PUBLIC EXPOSURE

III.3. For public exposure, the dose limits are:

- (a) An effective dose of 1 mSv in a year;
- (b) In special circumstances⁶⁸, a higher value of effective dose in a single year could apply, provided that the average effective dose over five consecutive years does not exceed 1 mSv per year;
- (c) An equivalent dose to the lens of the eye of 15 mSv in a year;
- (d) An equivalent dose to the skin of 50 mSv in a year.

VERIFICATION OF COMPLIANCE WITH DOSE LIMITS

III.4. The effective dose limits specified in this schedule apply to the sum of the relevant doses from external exposure in the specified period and the relevant committed doses from intakes in the same period; the period for calculating the committed dose shall normally be 50 years for intakes by adults and shall be up to age 70 years for intakes by children.

III.5. For occupational exposure, the personal dose equivalent $H_p(10)$ ⁶⁹ may be used as an approximation of the effective dose from external exposure to penetrating radiation.

III.6. Values of the effective dose per unit air kerma free-in-air and per unit particle fluence are given in Tables III.1A–III.1D [30].

III.7. Doses per unit intake (dose coefficients) for the estimation of the committed effective dose for ingestion and inhalation of radionuclides are given in Tables III.2A–III.2H (p. 137) [37, 38].

⁶⁸ For example, in authorized, justified and planned operational conditions that lead to transitory increases in exposures.

⁶⁹ $H_p(10)$ is the personal dose equivalent $H_p(d)$ where $d = 10$ mm.

TABLE III.1A. CONVERSION COEFFICIENTS FROM AIR KERMA FREE-IN-AIR TO $H_p(10,0^\circ)$ IN AN INTERNATIONAL COMMISSION ON RADIATION UNITS AND MEASUREMENTS ICRU SLAB (PHOTONS) [30]

Photon energy (MeV)	$H_p(10,0^\circ)/K$ (Sv/Gy)	Photon energy (MeV)	$H_p(10,0^\circ)/K$ (Sv/Gy)
0.010	0.009	0.150	1.607
0.0125	0.098	0.200	1.492
0.015	0.264	0.300	1.369
0.0175	0.445	0.400	1.300
0.020	0.611	0.500	1.256
0.025	0.883	0.600	1.226
0.030	1.112	0.800	1.190
0.040	1.490	1.0	1.167
0.050	1.766	1.5	1.139
0.060	1.892	3.0	1.117
0.080	1.903	6.0	1.109
0.100	1.811	10.0	1.111
0.125	1.696		

TABLE III.1B. CONVERSION COEFFICIENTS FROM AIR KERMA FREE-IN-AIR TO $H_p(0.07,0^\circ)$ IN AN INTERNATIONAL COMMISSION ON RADIATION UNITS AND MEASUREMENTS ICRU SLAB (PHOTONS) [30]

Photon energy (MeV)	$H_p(0.07,0^\circ)/K$ (Sv/Gy)	Photon energy (MeV)	$H_p(0.07,0^\circ)/K$ (Sv/Gy)
0.005	0.750	0.100	1.669
0.010	0.947	0.150	1.518
0.015	0.981	0.200	1.432
0.020	1.045	0.300	1.336
0.030	1.230	0.400	1.280
0.040	1.444	0.500	1.244
0.050	1.632	0.600	1.220
0.060	1.716	0.800	1.189
0.080	1.732	1.000	1.173

TABLE III.1C. EFFECTIVE DOSE PER UNIT NEUTRON FLUENCE E/Φ FOR MONOENERGETIC NEUTRONS INCIDENT IN INTERNATIONAL ORGANIZATION FOR STANDARDIZATION GEOMETRY ON AN ADULT ANTHROPOMORPHIC COMPUTATIONAL PHANTOM [30]

Neutron energy (MeV)	E/Φ (pSv·cm ²)	Neutron energy (MeV)	E/Φ (pSv·cm ²)
1.00×10^{-9}	2.40	1.50×10^{-1}	35.2
1.00×10^{-8}	2.89	2.00×10^{-1}	42.4
2.53×10^{-8}	3.30	3.00×10^{-1}	54.7
1.00×10^{-7}	4.13	5.00×10^{-1}	75.0
2.00×10^{-7}	4.59	7.00×10^{-1}	92.8
5.00×10^{-7}	5.20	9.00×10^{-1}	108
1.00×10^{-6}	5.63	1.00×10^0	116
2.00×10^{-6}	5.96	1.20×10^0	130
5.00×10^{-6}	6.28	2.00×10^0	178
1.00×10^{-5}	6.44	3.00×10^0	220
2.00×10^{-5}	6.51	4.00×10^0	250
5.00×10^{-5}	6.51	5.00×10^0	272
1.00×10^{-4}	6.45	6.00×10^0	282
2.00×10^{-4}	6.32	7.00×10^0	290
5.00×10^{-4}	6.14	8.00×10^0	297
1.00×10^{-3}	6.04	9.00×10^0	303
2.00×10^{-3}	6.05	1.00×10^1	309
5.00×10^{-3}	6.52	1.20×10^1	322
1.00×10^{-2}	7.70	1.40×10^1	333
2.00×10^{-2}	10.2	1.50×10^1	338
3.00×10^{-2}	12.7	1.60×10^1	342
5.00×10^{-2}	17.3	1.80×10^1	345
7.00×10^{-2}	21.5	2.00×10^1	343
1.00×10^{-1}	25.2		

TABLE III.1D. REFERENCE CONVERSION COEFFICIENTS FROM FLUENCE TO DIRECTIONAL DOSE EQUIVALENT FOR MONOENERGETIC ELECTRONS AND NORMAL INCIDENCE [30]

Electron energy (MeV)	$H'(0.07,0^\circ)/\Phi$ (nSv·cm ²)	$H'(3,0^\circ)/\Phi$ (nSv·cm ²)	$H'(10,0^\circ)/\Phi$ (nSv·cm ²)
0.07	0.221		
0.08	1.056		
0.09	1.527		
0.10	1.661		
0.1125	1.627		
0.125	1.513		
0.15	1.229		
0.20	0.834		
0.30	0.542		
0.40	0.455		
0.50	0.403		
0.60	0.366		
0.70	0.344	0.000	
0.80	0.329	0.045	
1.00	0.312	0.301	
1.25	0.296	0.486	
1.50	0.287	0.524	
1.75	0.282	0.512	0.000
2.00	0.279	0.481	0.005
2.50	0.278	0.417	0.156
3.00	0.276	0.373	0.336
3.50	0.274	0.351	0.421
4.00	0.272	0.334	0.447
5.00	0.271	0.317	0.430
6.00	0.271	0.309	0.389
7.00	0.271	0.306	0.360
8.00	0.271	0.305	0.341
10.00	0.275	0.303	0.330

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_1	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_1	$e(g)$
Hydrogen							
Tritiated water	12.3 a					1.000	1.8×10^{-11}
Organically bound tritium	12.3 a					1.000	4.2×10^{-11}
Beryllium							
Be-7	53.3 d	M	0.005	4.8×10^{-11}	4.3×10^{-11}	0.005	2.8×10^{-11}
		S	0.005	5.2×10^{-11}	4.6×10^{-11}		
Be-10	1.60×10^6 a	M	0.005	9.1×10^{-9}	6.7×10^{-9}	0.005	1.1×10^{-9}
		S	0.005	3.2×10^{-8}	1.9×10^{-8}		
Carbon							
C-11	0.340 h					1.000	2.4×10^{-11}
C-14	5.73×10^3 a					1.000	5.8×10^{-10}
Fluorine							
F-18	1.83 h	F	1.000	3.0×10^{-11}	5.4×10^{-11}	1.000	4.9×10^{-11}
		M	1.000	5.7×10^{-11}	8.9×10^{-11}		
		S	1.000	6.0×10^{-11}	9.3×10^{-11}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Sodium							
Na-22	2.60 a	F	1.000	1.3×10^{-9}	2.0×10^{-9}	1.000	3.2×10^{-9}
Na-24	15.0 h	F	1.000	2.9×10^{-10}	5.3×10^{-10}	1.000	4.3×10^{-10}
Magnesium							
Mg-28	20.9 h	F	0.500	6.4×10^{-10}	1.1×10^{-9}	0.500	2.2×10^{-9}
		M	0.500	1.2×10^{-9}	1.7×10^{-9}		
Aluminium							
Al-26	7.16×10^5 a	F	0.010	1.1×10^{-8}	1.4×10^{-8}	0.010	3.5×10^{-9}
		M	0.010	1.8×10^{-8}	1.2×10^{-8}		
Silicon							
Si-31	2.62 h	F	0.010	2.9×10^{-11}	5.1×10^{-11}	0.010	1.6×10^{-10}
		M	0.010	7.5×10^{-11}	1.1×10^{-10}		
		S	0.010	8.0×10^{-11}	1.1×10^{-10}		
Si-32	4.50×10^2 a	F	0.010	3.2×10^{-9}	3.7×10^{-9}	0.010	5.6×10^{-10}
		M	0.010	1.5×10^{-8}	9.6×10^{-9}		
		S	0.010	1.1×10^{-7}	5.5×10^{-8}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\ \mu m}$	$e(g)_{5\ \mu m}$	f_i	$e(g)$
Phosphorus							
P-32	14.3 d	F	0.800	8.0×10^{-10}	1.1×10^{-9}	0.800	2.4×10^{-9}
		M	0.800	3.2×10^{-9}	2.9×10^{-9}		
P-33	25.4 d	F	0.800	9.6×10^{-11}	1.4×10^{-10}	0.800	2.4×10^{-10}
		M	0.800	1.4×10^{-9}	1.3×10^{-9}		
Sulphur							
S-35 (inorganic)	87.4 d	F	0.800	5.3×10^{-11}	8.0×10^{-11}	0.800	1.4×10^{-10}
		M	0.800	1.3×10^{-9}	1.1×10^{-9}	0.100	1.9×10^{-10}
S-35 (organic)	87.4 d					1.000	7.7×10^{-10}
Chlorine							
Cl-36	3.01×10^5 a	F	1.000	3.4×10^{-10}	4.9×10^{-10}	1.000	9.3×10^{-10}
		M	1.000	6.9×10^{-9}	5.1×10^{-9}		
Cl-38	0.620 h	F	1.000	2.7×10^{-11}	4.6×10^{-11}	1.000	1.2×10^{-10}
		M	1.000	4.7×10^{-11}	7.3×10^{-11}		
Cl-39	0.927 h	F	1.000	2.7×10^{-11}	4.8×10^{-11}	1.000	8.5×10^{-11}
		M	1.000	4.8×10^{-11}	7.6×10^{-11}		
Potassium							
K-40	1.28×10^9 a	F	1.000	2.1×10^{-9}	3.0×10^{-9}	1.000	6.2×10^{-9}
		F	1.000	1.3×10^{-10}	2.0×10^{-10}	1.000	4.3×10^{-10}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\ \mu m}$	$e(g)_{5\ \mu m}$	f_i	$e(g)$
K-43	22.6 h	F	1.000	1.5×10^{-10}	2.6×10^{-10}	1.000	2.5×10^{-10}
K-44	0.369 h	F	1.000	2.1×10^{-11}	3.7×10^{-11}	1.000	8.4×10^{-11}
K-45	0.333 h	F	1.000	1.6×10^{-11}	2.8×10^{-11}	1.000	5.4×10^{-11}
Calcium							
Ca-41	1.40×10^5 a	M	0.300	1.7×10^{-10}	1.9×10^{-10}	0.300	2.9×10^{-10}
Ca-45	163 d	M	0.300	2.7×10^{-9}	2.3×10^{-9}	0.300	7.6×10^{-10}
Ca-47	4.53 d	M	0.300	1.8×10^{-9}	2.1×10^{-9}	0.300	1.6×10^{-9}
Scandium							
Sc-43	3.89 h	S	1.0×10^{-4}	1.2×10^{-10}	1.8×10^{-10}	1.0×10^{-4}	1.9×10^{-10}
Sc-44	3.93 h	S	1.0×10^{-4}	1.9×10^{-10}	3.0×10^{-10}	1.0×10^{-4}	3.5×10^{-10}
Sc-44m	2.44 d	S	1.0×10^{-4}	1.5×10^{-9}	2.0×10^{-9}	1.0×10^{-4}	2.4×10^{-9}
Sc-46	83.8 d	S	1.0×10^{-4}	6.4×10^{-9}	4.8×10^{-9}	1.0×10^{-4}	1.5×10^{-9}
Sc-47	3.35 d	S	1.0×10^{-4}	7.0×10^{-10}	7.3×10^{-10}	1.0×10^{-4}	5.4×10^{-10}
Sc-48	1.82 d	S	1.0×10^{-4}	1.1×10^{-9}	1.6×10^{-9}	1.0×10^{-4}	1.7×10^{-9}
Sc-49	0.956 h	S	1.0×10^{-4}	4.1×10^{-11}	6.1×10^{-11}	1.0×10^{-4}	8.2×10^{-11}
Titanium							
Ti-44	47.3 a	F	0.010	6.1×10^{-8}	7.2×10^{-8}	0.010	5.8×10^{-9}
		M	0.010	4.0×10^{-8}	2.7×10^{-8}		
		S	0.010	1.2×10^{-7}	6.2×10^{-8}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Ti-45	3.08 h	F	0.010	4.6×10^{-11}	8.3×10^{-11}	0.010	1.5×10^{-10}
		M	0.010	9.1×10^{-11}	1.4×10^{-10}		
		S	0.010	9.6×10^{-11}	1.5×10^{-10}		
Vanadium	0.543 h	F	0.010	1.9×10^{-11}	3.2×10^{-11}	0.010	6.3×10^{-11}
		M	0.010	3.1×10^{-11}	5.0×10^{-11}		
		F	0.010	1.1×10^{-9}	1.7×10^{-9}		
V-48	16.2 d	M	0.010	2.3×10^{-9}	2.7×10^{-9}	0.010	2.0×10^{-9}
		F	0.010	2.1×10^{-11}	2.6×10^{-11}		
		M	0.010	3.2×10^{-11}	2.3×10^{-11}		
V-49	330 d	F	0.010	2.1×10^{-11}	2.6×10^{-11}	0.010	1.8×10^{-11}
		M	0.010	3.2×10^{-11}	2.3×10^{-11}		
Chromium	23.0 h	F	0.100	1.0×10^{-10}	1.7×10^{-10}	0.100	2.0×10^{-10}
		M	0.100	2.0×10^{-10}	2.3×10^{-10}		
		S	0.100	2.2×10^{-10}	2.5×10^{-10}		
Cr-49	0.702 h	F	0.100	2.0×10^{-11}	3.5×10^{-11}	0.100	6.1×10^{-11}
		M	0.100	3.5×10^{-11}	5.6×10^{-11}		
		S	0.100	3.7×10^{-11}	5.9×10^{-11}		
Cr-51	27.7 d	F	0.100	2.1×10^{-11}	3.0×10^{-11}	0.100	3.8×10^{-11}
		M	0.100	3.1×10^{-11}	3.4×10^{-11}		
		S	0.100	3.6×10^{-11}	3.6×10^{-11}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Manganese							
Mn-51	0.770 h	F	0.100	2.4×10^{-11}	4.2×10^{-11}	0.100	9.3×10^{-11}
		M	0.100	4.3×10^{-11}	6.8×10^{-11}		
Mn-52	5.59 d	F	0.100	9.9×10^{-10}	1.6×10^{-9}	0.100	1.8×10^{-9}
		M	0.100	1.4×10^{-9}	1.8×10^{-9}		
Mn-52m	0.352 h	F	0.100	2.0×10^{-11}	3.5×10^{-11}	0.100	6.9×10^{-11}
		M	0.100	3.0×10^{-11}	5.0×10^{-11}		
Mn-53	3.70×10^6 a	F	0.100	2.9×10^{-11}	3.6×10^{-11}	0.100	3.0×10^{-11}
		M	0.100	5.2×10^{-11}	3.6×10^{-11}		
Mn-54	312 d	F	0.100	8.7×10^{-10}	1.1×10^{-9}	0.100	7.1×10^{-10}
		M	0.100	1.5×10^{-9}	1.2×10^{-9}		
Mn-56	2.58 h	F	0.100	6.9×10^{-11}	1.2×10^{-10}	0.100	2.5×10^{-10}
		M	0.100	1.3×10^{-10}	2.0×10^{-10}		
Iron							
Fe-52	8.28 h	F	0.100	4.1×10^{-10}	6.9×10^{-10}	0.100	1.4×10^{-9}
		M	0.100	6.3×10^{-10}	9.5×10^{-10}		
Fe-55	2.70 a	F	0.100	7.7×10^{-10}	9.2×10^{-10}	0.100	3.3×10^{-10}
		M	0.100	3.7×10^{-10}	3.3×10^{-10}		
Fe-59	44.5 d	F	0.100	2.2×10^{-9}	3.0×10^{-9}	0.100	1.8×10^{-9}
		M	0.100	3.5×10^{-9}	3.2×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Fe-60	1.00×10^5 a	F	0.100	2.8×10^{-7}	3.3×10^{-7}	0.100	1.1×10^{-7}
		M	0.100	1.3×10^{-7}	1.2×10^{-7}		
Cobalt	17.5 h	M	0.100	5.1×10^{-10}	7.8×10^{-10}	0.100	1.0×10^{-9}
		S	0.050	5.5×10^{-10}	8.3×10^{-10}		
Co-56	78.7 d	M	0.100	4.6×10^{-9}	4.0×10^{-9}	0.100	2.5×10^{-9}
		S	0.050	6.3×10^{-9}	4.9×10^{-9}		
Co-57	271 d	M	0.100	5.2×10^{-10}	3.9×10^{-10}	0.100	2.1×10^{-10}
		S	0.050	9.4×10^{-10}	6.0×10^{-10}		
Co-58	70.8 d	M	0.100	1.5×10^{-9}	1.4×10^{-9}	0.100	7.4×10^{-10}
		S	0.050	2.0×10^{-9}	1.7×10^{-9}		
Co-58m	9.15 h	M	0.100	1.3×10^{-11}	1.5×10^{-11}	0.100	2.4×10^{-11}
		S	0.050	1.6×10^{-11}	1.7×10^{-11}		
Co-60	5.27 a	M	0.100	9.6×10^{-9}	7.1×10^{-9}	0.100	3.4×10^{-9}
		S	0.050	2.9×10^{-8}	1.7×10^{-8}		
Co-60m	0.174 h	M	0.100	1.1×10^{-12}	1.2×10^{-12}	0.100	1.7×10^{-12}
		S	0.050	1.3×10^{-12}	1.2×10^{-12}		
Co-61	1.65 h	M	0.100	4.8×10^{-11}	7.1×10^{-11}	0.100	7.4×10^{-11}
		S	0.050	5.1×10^{-11}	7.5×10^{-11}		
Co-62m	0.232 h	M	0.100	2.1×10^{-11}	3.6×10^{-11}	0.100	4.7×10^{-11}
		S	0.050	2.2×10^{-11}	3.7×10^{-11}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Nickel							
Ni-56	6.10 d	F	0.050	5.1×10^{-10}	7.9×10^{-10}	0.050	8.6×10^{-10}
		M	0.050	8.6×10^{-10}	9.6×10^{-10}		
Ni-57	1.50 d	F	0.050	2.8×10^{-10}	5.0×10^{-10}	0.050	8.7×10^{-10}
		M	0.050	5.1×10^{-10}	7.6×10^{-10}		
Ni-59	7.50×10^4 a	F	0.050	1.8×10^{-10}	2.2×10^{-10}	0.050	6.3×10^{-11}
		M	0.050	1.3×10^{-10}	9.4×10^{-11}		
Ni-63	96.0 a	F	0.050	4.4×10^{-10}	5.2×10^{-10}	0.050	1.5×10^{-10}
		M	0.050	4.4×10^{-10}	3.1×10^{-10}		
Ni-65	2.52 h	F	0.050	4.4×10^{-11}	7.5×10^{-11}	0.050	1.8×10^{-10}
		M	0.050	8.7×10^{-11}	1.3×10^{-10}		
Ni-66	2.27 d	F	0.050	4.5×10^{-10}	7.6×10^{-10}	0.050	3.0×10^{-9}
		M	0.050	1.6×10^{-9}	1.9×10^{-9}		
Copper							
Cu-60	0.387 h	F	0.500	2.4×10^{-11}	4.4×10^{-11}	0.500	7.0×10^{-11}
		M	0.500	3.5×10^{-11}	6.0×10^{-11}		
		S	0.500	3.6×10^{-11}	6.2×10^{-11}		
Cu-61	3.41 h	F	0.500	4.0×10^{-11}	7.3×10^{-11}	0.500	1.2×10^{-10}
		M	0.500	7.6×10^{-11}	1.2×10^{-10}		
		S	0.500	8.0×10^{-11}	1.2×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Cu-64	12.7 h	F	0.500	3.8×10^{-11}	6.8×10^{-11}	0.500	1.2×10^{-10}
		M	0.500	1.1×10^{-10}	1.5×10^{-10}		
		S	0.500	1.2×10^{-10}	1.5×10^{-10}		
Cu-67	2.58 d	F	0.500	1.1×10^{-10}	1.8×10^{-10}	0.500	3.4×10^{-10}
		M	0.500	5.2×10^{-10}	5.3×10^{-10}		
		S	0.500	5.8×10^{-10}	5.8×10^{-10}		
Zinc							
Zn-62	9.26 h	S	0.500	4.7×10^{-10}	6.6×10^{-10}	0.500	9.4×10^{-10}
Zn-63	0.635 h	S	0.500	3.8×10^{-11}	6.1×10^{-11}	0.500	7.9×10^{-11}
Zn-65	244 d	S	0.500	2.9×10^{-9}	2.8×10^{-9}	0.500	3.9×10^{-9}
Zn-69	0.950 h	S	0.500	2.8×10^{-11}	4.3×10^{-11}	0.500	3.1×10^{-11}
Zn-69m	13.8 h	S	0.500	2.6×10^{-10}	3.3×10^{-10}	0.500	3.3×10^{-10}
Zn-71m	3.92 h	S	0.500	1.6×10^{-10}	2.4×10^{-10}	0.500	2.4×10^{-10}
Zn-72	1.94 d	S	0.500	1.2×10^{-9}	1.5×10^{-9}	0.500	1.4×10^{-9}
Gallium							
Ga-65	0.253 h	F	0.001	1.2×10^{-11}	2.0×10^{-11}	0.001	3.7×10^{-11}
		M	0.001	1.8×10^{-11}	2.9×10^{-11}		
Ga-66	9.40 h	F	0.001	2.7×10^{-10}	4.7×10^{-10}	0.001	1.2×10^{-9}
		M	0.001	4.6×10^{-10}	7.1×10^{-10}		
Ga-67	3.26 d	F	0.001	6.8×10^{-11}	1.1×10^{-10}	0.001	1.9×10^{-10}
		M	0.001	2.3×10^{-10}	2.8×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Ga-68	1.13 h	F	0.001	2.8×10^{-11}	4.9×10^{-11}	0.001	1.0×10^{-10}
		M	0.001	5.1×10^{-11}	8.1×10^{-11}		
Ga-70	0.353 h	F	0.001	9.3×10^{-12}	1.6×10^{-11}	0.001	3.1×10^{-11}
		M	0.001	1.6×10^{-11}	2.6×10^{-11}		
Ga-72	14.1 h	F	0.001	3.1×10^{-10}	5.6×10^{-10}	0.001	1.1×10^{-9}
		M	0.001	5.5×10^{-10}	8.4×10^{-10}		
Ga-73	4.91 h	F	0.001	5.8×10^{-11}	1.0×10^{-10}	0.001	2.6×10^{-10}
		M	0.001	1.5×10^{-10}	2.0×10^{-10}		
Germanium							
Ge-66	2.27 h	F	1.000	5.7×10^{-11}	9.9×10^{-11}	1.000	1.0×10^{-10}
		M	1.000	9.2×10^{-11}	1.3×10^{-10}		
Ge-67	0.312 h	F	1.000	1.6×10^{-11}	2.8×10^{-11}	1.000	6.5×10^{-11}
		M	1.000	2.6×10^{-11}	4.2×10^{-11}		
Ge-68	288 d	F	1.000	5.4×10^{-10}	8.3×10^{-10}	1.000	1.3×10^{-9}
		M	1.000	1.3×10^{-8}	7.9×10^{-9}		
Ge-69	1.63 d	F	1.000	1.4×10^{-10}	2.5×10^{-10}	1.000	2.4×10^{-10}
		M	1.000	2.9×10^{-10}	3.7×10^{-10}		
Ge-71	11.8 d	F	1.000	5.0×10^{-12}	7.8×10^{-12}	1.000	1.2×10^{-11}
		M	1.000	1.0×10^{-11}	1.1×10^{-11}		
Ge-75	1.38 h	F	1.000	1.6×10^{-11}	2.7×10^{-11}	1.000	4.6×10^{-11}
		M	1.000	3.7×10^{-11}	5.4×10^{-11}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Ge-77	11.3 h	F	1.000	1.5×10^{-10}	2.5×10^{-10}	1.000	3.3×10^{-10}
		M	1.000	3.6×10^{-10}	4.5×10^{-10}		
Ge-78	1.45 h	F	1.000	4.8×10^{-11}	8.1×10^{-11}	1.000	1.2×10^{-10}
		M	1.000	9.7×10^{-11}	1.4×10^{-10}		
Arsenic							
As-69	0.253 h	M	0.500	2.2×10^{-11}	3.5×10^{-11}	0.500	5.7×10^{-11}
As-70	0.876 h	M	0.500	7.2×10^{-11}	1.2×10^{-10}	0.500	1.3×10^{-10}
As-71	2.70 d	M	0.500	4.0×10^{-10}	5.0×10^{-10}	0.500	4.6×10^{-10}
As-72	1.08 d	M	0.500	9.2×10^{-10}	1.3×10^{-9}	0.500	1.8×10^{-9}
As-73	80.3 d	M	0.500	9.3×10^{-10}	6.5×10^{-10}	0.500	2.6×10^{-10}
As-74	17.8 d	M	0.500	2.1×10^{-9}	1.8×10^{-9}	0.500	1.3×10^{-9}
As-76	1.10 d	M	0.500	7.4×10^{-10}	9.2×10^{-10}	0.500	1.6×10^{-9}
As-77	1.62 d	M	0.500	3.8×10^{-10}	4.2×10^{-10}	0.500	4.0×10^{-10}
As-78	1.51 h	M	0.500	9.2×10^{-11}	1.4×10^{-10}	0.500	2.1×10^{-10}
Selenium							
Se-70	0.683 h	F	0.800	4.5×10^{-11}	8.2×10^{-11}	0.800	1.2×10^{-10}
		M	0.800	7.3×10^{-11}	1.2×10^{-10}	0.050	1.4×10^{-10}
Se-73	7.15 h	F	0.800	8.6×10^{-11}	1.5×10^{-10}	0.800	2.1×10^{-10}
		M	0.800	1.6×10^{-10}	2.4×10^{-10}	0.050	3.9×10^{-10}
Se-73m	0.650 h	F	0.800	9.9×10^{-12}	1.7×10^{-11}	0.800	2.8×10^{-11}
		M	0.800	1.8×10^{-11}	2.7×10^{-11}	0.050	4.1×10^{-11}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Se-75	120 d	F	0.800	1.0×10^{-9}	1.4×10^{-9}	0.800	2.6×10^{-9}
		M	0.800	1.4×10^{-9}	1.7×10^{-9}	0.050	4.1×10^{-10}
Se-79	6.50×10^4 a	F	0.800	1.2×10^{-9}	1.6×10^{-9}	0.800	2.9×10^{-9}
		M	0.800	2.9×10^{-9}	3.1×10^{-9}	0.050	3.9×10^{-10}
Se-81	0.308 h	F	0.800	8.6×10^{-12}	1.4×10^{-11}	0.800	2.7×10^{-11}
		M	0.800	1.5×10^{-11}	2.4×10^{-11}	0.050	2.7×10^{-11}
Se-81m	0.954 h	F	0.800	1.7×10^{-11}	3.0×10^{-11}	0.800	5.3×10^{-11}
		M	0.800	4.7×10^{-11}	6.8×10^{-11}	0.050	5.9×10^{-11}
Se-83	0.375 h	F	0.800	1.9×10^{-11}	3.4×10^{-11}	0.800	4.7×10^{-11}
		M	0.800	3.3×10^{-11}	5.3×10^{-11}	0.050	5.1×10^{-11}
Bromine							
Br-74	0.422 h	F	1.000	2.8×10^{-11}	5.0×10^{-11}	1.000	8.4×10^{-11}
		M	1.000	4.1×10^{-11}	6.8×10^{-11}		
Br-74m	0.691 h	F	1.000	4.2×10^{-11}	7.5×10^{-11}	1.000	1.4×10^{-10}
		M	1.000	6.5×10^{-11}	1.1×10^{-10}		
Br-75	1.63 h	F	1.000	3.1×10^{-11}	5.6×10^{-11}	1.000	7.9×10^{-11}
		M	1.000	5.5×10^{-11}	8.5×10^{-11}		
Br-76	16.2 h	F	1.000	2.6×10^{-10}	4.5×10^{-10}	1.000	4.6×10^{-10}
		M	1.000	4.2×10^{-10}	5.8×10^{-10}		
Br-77	2.33 d	F	1.000	6.7×10^{-11}	1.2×10^{-10}	1.000	9.6×10^{-11}
		M	1.000	8.7×10^{-11}	1.3×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Br-80	0.290 h	F	1.000	6.3×10^{-12}	1.1×10^{-11}	1.000	3.1×10^{-11}
		M	1.000	1.0×10^{-11}	1.7×10^{-11}		
Br-80m	4.42 h	F	1.000	3.5×10^{-11}	5.8×10^{-11}	1.000	1.1×10^{-10}
		M	1.000	7.6×10^{-11}	1.0×10^{-10}		
Br-82	1.47 d	F	1.000	3.7×10^{-10}	6.4×10^{-10}	1.000	5.4×10^{-10}
		M	1.000	6.4×10^{-10}	8.8×10^{-10}		
Br-83	2.39 h	F	1.000	1.7×10^{-11}	2.9×10^{-11}	1.000	4.3×10^{-11}
		M	1.000	4.8×10^{-11}	6.7×10^{-11}		
Br-84	0.530 h	F	1.000	2.3×10^{-11}	4.0×10^{-11}	1.000	8.8×10^{-11}
		M	1.000	3.9×10^{-11}	6.2×10^{-11}		
Rubidium							
Rb-79	0.382 h	F	1.000	1.7×10^{-11}	3.0×10^{-11}	1.000	5.0×10^{-11}
Rb-81	4.58 h	F	1.000	3.7×10^{-11}	6.8×10^{-11}	1.000	5.4×10^{-11}
Rb-81m	0.533 h	F	1.000	7.3×10^{-12}	1.3×10^{-11}	1.000	9.7×10^{-12}
Rb-82m	6.20 h	F	1.000	1.2×10^{-10}	2.2×10^{-10}	1.000	1.3×10^{-10}
Rb-83	86.2 d	F	1.000	7.1×10^{-10}	1.0×10^{-9}	1.000	1.9×10^{-9}
Rb-84	32.8 d	F	1.000	1.1×10^{-9}	1.5×10^{-9}	1.000	2.8×10^{-9}
Rb-86	18.6 d	F	1.000	9.6×10^{-10}	1.3×10^{-9}	1.000	2.8×10^{-9}
Rb-87	4.70×10^{10} a	F	1.000	5.1×10^{-10}	7.6×10^{-10}	1.000	1.5×10^{-9}
Rb-88	0.297 h	F	1.000	1.7×10^{-11}	2.8×10^{-11}	1.000	9.0×10^{-11}
Rb-89	0.253 h	F	1.000	1.4×10^{-11}	2.5×10^{-11}	1.000	4.7×10^{-11}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Strontium							
Sr-80	1.67 h	F	0.300	7.6×10^{-11}	1.3×10^{-10}	0.300	3.4×10^{-10}
		S	0.010	1.4×10^{-10}	2.1×10^{-10}	0.010	3.5×10^{-10}
Sr-81	0.425 h	F	0.300	2.2×10^{-11}	3.9×10^{-11}	0.300	7.7×10^{-11}
		S	0.010	3.8×10^{-11}	6.1×10^{-11}	0.010	7.8×10^{-11}
Sr-82	25.0 d	F	0.300	2.2×10^{-9}	3.3×10^{-9}	0.300	6.1×10^{-9}
		S	0.010	1.0×10^{-8}	7.7×10^{-9}	0.010	6.0×10^{-9}
Sr-83	1.35 d	F	0.300	1.7×10^{-10}	3.0×10^{-10}	0.300	4.9×10^{-10}
		S	0.010	3.4×10^{-10}	4.9×10^{-10}	0.010	5.8×10^{-10}
Sr-85	64.8 d	F	0.300	3.9×10^{-10}	5.6×10^{-10}	0.300	5.6×10^{-10}
		S	0.010	7.7×10^{-10}	6.4×10^{-10}	0.010	3.3×10^{-10}
Sr-85m	1.16 h	F	0.300	3.1×10^{-12}	5.6×10^{-12}	0.300	6.1×10^{-12}
		S	0.010	4.5×10^{-12}	7.4×10^{-12}	0.010	6.1×10^{-12}
Sr-87m	2.80 h	F	0.300	1.2×10^{-11}	2.2×10^{-11}	0.300	3.0×10^{-11}
		S	0.010	2.2×10^{-11}	3.5×10^{-11}	0.010	3.3×10^{-11}
Sr-89	50.5 d	F	0.300	1.0×10^{-9}	1.4×10^{-9}	0.300	2.6×10^{-9}
		S	0.010	7.5×10^{-9}	5.6×10^{-9}	0.010	2.3×10^{-9}
Sr-90	29.1 a	F	0.300	2.4×10^{-8}	3.0×10^{-8}	0.300	2.8×10^{-8}
		S	0.010	1.5×10^{-7}	7.7×10^{-8}	0.010	2.7×10^{-9}
Sr-91	9.50 h	F	0.300	1.7×10^{-10}	2.9×10^{-10}	0.300	6.5×10^{-10}
		S	0.010	4.1×10^{-10}	5.7×10^{-10}	0.010	7.6×10^{-10}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Sr-92	2.71 h	F	0.300	1.1×10^{-10}	1.8×10^{-10}	0.300	4.3×10^{-10}
		S	0.010	2.3×10^{-10}	3.4×10^{-10}	0.010	4.9×10^{-10}
Yttrium							
Y-86	14.7 h	M	1.0×10^{-4}	4.8×10^{-10}	8.0×10^{-10}	1.0×10^{-4}	9.6×10^{-10}
		S	1.0×10^{-4}	4.9×10^{-10}	8.1×10^{-10}		
Y-86m	0.800 h	M	1.0×10^{-4}	2.9×10^{-11}	4.8×10^{-11}	1.0×10^{-4}	5.6×10^{-11}
		S	1.0×10^{-4}	3.0×10^{-11}	4.9×10^{-11}		
Y-87	3.35 d	M	1.0×10^{-4}	3.8×10^{-10}	5.2×10^{-10}	1.0×10^{-4}	5.5×10^{-10}
		S	1.0×10^{-4}	4.0×10^{-10}	5.3×10^{-10}		
Y-88	107 d	M	1.0×10^{-4}	3.9×10^{-9}	3.3×10^{-9}	1.0×10^{-4}	1.3×10^{-9}
		S	1.0×10^{-4}	4.1×10^{-9}	3.0×10^{-9}		
Y-90	2.67 d	M	1.0×10^{-4}	1.4×10^{-9}	1.6×10^{-9}	1.0×10^{-4}	2.7×10^{-9}
		S	1.0×10^{-4}	1.5×10^{-9}	1.7×10^{-9}		
Y-90m	3.19 h	M	1.0×10^{-4}	9.6×10^{-11}	1.3×10^{-10}	1.0×10^{-4}	1.7×10^{-10}
		S	1.0×10^{-4}	1.0×10^{-10}	1.3×10^{-10}		
Y-91	58.5 d	M	1.0×10^{-4}	6.7×10^{-9}	5.2×10^{-9}	1.0×10^{-4}	2.4×10^{-9}
		S	1.0×10^{-4}	8.4×10^{-9}	6.1×10^{-9}		
Y-91m	0.828 h	M	1.0×10^{-4}	1.0×10^{-11}	1.4×10^{-11}	1.0×10^{-4}	1.1×10^{-11}
		S	1.0×10^{-4}	1.1×10^{-11}	1.5×10^{-11}		
Y-92	3.54 h	M	1.0×10^{-4}	1.9×10^{-10}	2.7×10^{-10}	1.0×10^{-4}	4.9×10^{-10}
		S	1.0×10^{-4}	2.0×10^{-10}	2.8×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Y-93	10.1 h	M	1.0×10^{-4}	4.1×10^{-10}	5.7×10^{-10}	1.0×10^{-4}	1.2×10^{-9}
		S	1.0×10^{-4}	4.3×10^{-10}	6.0×10^{-10}		
Y-94	0.318 h	M	1.0×10^{-4}	2.8×10^{-11}	4.4×10^{-11}	1.0×10^{-4}	8.1×10^{-11}
		S	1.0×10^{-4}	2.9×10^{-11}	4.6×10^{-11}		
Y-95	0.178 h	M	1.0×10^{-4}	1.6×10^{-11}	2.5×10^{-11}	1.0×10^{-4}	4.6×10^{-11}
		S	1.0×10^{-4}	1.7×10^{-11}	2.6×10^{-11}		
Zirconium							
Zr-86	16.5 h	F	0.002	3.0×10^{-10}	5.2×10^{-10}	0.002	8.6×10^{-10}
		M	0.002	4.3×10^{-10}	6.8×10^{-10}		
		S	0.002	4.5×10^{-10}	7.0×10^{-10}		
Zr-88	83.4 d	F	0.002	3.5×10^{-9}	4.1×10^{-9}	0.002	3.3×10^{-10}
		M	0.002	2.5×10^{-9}	1.7×10^{-9}		
		S	0.002	3.3×10^{-9}	1.8×10^{-9}		
Zr-89	3.27 d	F	0.002	3.1×10^{-10}	5.2×10^{-10}	0.002	7.9×10^{-10}
		M	0.002	5.3×10^{-10}	7.2×10^{-10}		
		S	0.002	5.5×10^{-10}	7.5×10^{-10}		
Zr-93	1.53×10^6 a	F	0.002	2.5×10^{-8}	2.9×10^{-8}	0.002	2.8×10^{-10}
		M	0.002	9.6×10^{-9}	6.6×10^{-9}		
		S	0.002	3.1×10^{-9}	1.7×10^{-9}		
Zr-95	64.0 d	F	0.002	2.5×10^{-9}	3.0×10^{-9}	0.002	8.8×10^{-10}
		M	0.002	4.5×10^{-9}	3.6×10^{-9}		
		S	0.002	5.5×10^{-9}	4.2×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Zr-97	16.9 h	F	0.002	4.2×10^{-10}	7.4×10^{-10}	0.002	2.1×10^{-9}
		M	0.002	9.4×10^{-10}	1.3×10^{-9}		
		S	0.002	1.0×10^{-9}	1.4×10^{-9}		
Niobium							
Nb-88	0.238 h	M	0.010	2.9×10^{-11}	4.8×10^{-11}	0.010	6.3×10^{-11}
		S	0.010	3.0×10^{-11}	5.0×10^{-11}		
Nb-89	2.03 h	M	0.010	1.2×10^{-10}	1.8×10^{-10}	0.010	3.0×10^{-10}
		S	0.010	1.3×10^{-10}	1.9×10^{-10}		
Nb-89m	1.10 h	M	0.010	7.1×10^{-11}	1.1×10^{-10}	0.010	1.4×10^{-10}
		S	0.010	7.4×10^{-11}	1.2×10^{-10}		
Nb-90	14.6 h	M	0.010	6.6×10^{-10}	1.0×10^{-9}	0.010	1.2×10^{-9}
		S	0.010	6.9×10^{-10}	1.1×10^{-9}		
Nb-93m	13.6 a	M	0.010	4.6×10^{-10}	2.9×10^{-10}	0.010	1.2×10^{-10}
		S	0.010	1.6×10^{-9}	8.6×10^{-10}		
Nb-94	2.03×10^4 a	M	0.010	1.0×10^{-8}	7.2×10^{-9}	0.010	1.7×10^{-9}
		S	0.010	4.5×10^{-8}	2.5×10^{-8}		
Nb-95	35.1 d	M	0.010	1.4×10^{-9}	1.3×10^{-9}	0.010	5.8×10^{-10}
		S	0.010	1.6×10^{-9}	1.3×10^{-9}		
Nb-95m	3.61 d	M	0.010	7.6×10^{-10}	7.7×10^{-10}	0.010	5.6×10^{-10}
		S	0.010	8.5×10^{-10}	8.5×10^{-10}		
Nb-96	23.3 h	M	0.010	6.5×10^{-10}	9.7×10^{-10}	0.010	1.1×10^{-9}
		S	0.010	6.8×10^{-10}	1.0×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Nb-97	1.20 h	M	0.010	4.4×10^{-11}	6.9×10^{-11}	0.010	6.8×10^{-11}
		S	0.010	4.7×10^{-11}	7.2×10^{-11}		
	0.858 h	M	0.010	5.9×10^{-11}	9.6×10^{-11}	0.010	1.1×10^{-10}
		S	0.010	6.1×10^{-11}	9.9×10^{-11}		
Molybdenum							
Mo-90	5.67 h	F	0.800	1.7×10^{-10}	2.9×10^{-10}	0.800	3.1×10^{-10}
		S	0.050	3.7×10^{-10}	5.6×10^{-10}	0.050	6.2×10^{-10}
Mo-93	3.50×10^3 a	F	0.800	1.0×10^{-9}	1.4×10^{-9}	0.800	2.6×10^{-9}
		S	0.050	2.2×10^{-9}	1.2×10^{-9}	0.050	2.0×10^{-10}
Mo-93m	6.85 h	F	0.800	1.0×10^{-10}	1.9×10^{-10}	0.800	1.6×10^{-10}
		S	0.050	1.8×10^{-10}	3.0×10^{-10}	0.050	2.8×10^{-10}
Mo-99	2.75 d	F	0.800	2.3×10^{-10}	3.6×10^{-10}	0.800	7.4×10^{-10}
		S	0.050	9.7×10^{-10}	1.1×10^{-9}	0.050	1.2×10^{-9}
Mo-101	0.244 h	F	0.800	1.5×10^{-11}	2.7×10^{-11}	0.800	4.2×10^{-11}
		S	0.050	2.7×10^{-11}	4.5×10^{-11}	0.050	4.2×10^{-11}
Technetium							
Tc-93	2.75 h	F	0.800	3.4×10^{-11}	6.2×10^{-11}	0.800	4.9×10^{-11}
		M	0.800	3.6×10^{-11}	6.5×10^{-11}		
Tc-93m	0.725 h	F	0.800	1.5×10^{-11}	2.6×10^{-11}	0.800	2.4×10^{-11}
		M	0.800	1.7×10^{-11}	3.1×10^{-11}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Tc-94	4.88 h	F	0.800	1.2×10^{-10}	2.1×10^{-10}	0.800	1.8×10^{-10}
		M	0.800	1.3×10^{-10}	2.2×10^{-10}		
Tc-94m	0.867 h	F	0.800	4.3×10^{-11}	6.9×10^{-11}	0.800	1.1×10^{-10}
		M	0.800	4.9×10^{-11}	8.0×10^{-11}		
Tc-95	20.0 h	F	0.800	1.0×10^{-10}	1.8×10^{-10}	0.800	1.6×10^{-10}
		M	0.800	1.0×10^{-10}	1.8×10^{-10}		
Tc-95m	61.0 d	F	0.800	3.1×10^{-10}	4.8×10^{-10}	0.800	6.2×10^{-10}
		M	0.800	8.7×10^{-10}	8.6×10^{-10}		
Tc-96	4.28 d	F	0.800	6.0×10^{-10}	9.8×10^{-10}	0.800	1.1×10^{-9}
		M	0.800	7.1×10^{-10}	1.0×10^{-9}		
Tc-96m	0.858 h	F	0.800	6.5×10^{-12}	1.1×10^{-11}	0.800	1.3×10^{-11}
		M	0.800	7.7×10^{-12}	1.1×10^{-11}		
Tc-97	2.60×10^6 a	F	0.800	4.5×10^{-11}	7.2×10^{-11}	0.800	8.3×10^{-11}
		M	0.800	2.1×10^{-10}	1.6×10^{-10}		
Tc-97m	87.0 d	F	0.800	2.8×10^{-10}	4.0×10^{-10}	0.800	6.6×10^{-10}
		M	0.800	3.1×10^{-9}	2.7×10^{-9}		
Tc-98	4.20×10^6 a	F	0.800	1.0×10^{-9}	1.5×10^{-9}	0.800	2.3×10^{-9}
		M	0.800	8.1×10^{-9}	6.1×10^{-9}		
Tc-99	2.13×10^5 a	F	0.800	2.9×10^{-10}	4.0×10^{-10}	0.800	7.8×10^{-10}
		M	0.800	3.9×10^{-9}	3.2×10^{-9}		
Tc-99m	6.02 h	F	0.800	1.2×10^{-11}	2.0×10^{-11}	0.800	2.2×10^{-11}
		M	0.800	1.9×10^{-11}	2.9×10^{-11}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Tc-101	0.237 h	F	0.800	8.7×10^{-12}	1.5×10^{-11}	0.800	1.9×10^{-11}
		M	0.800	1.3×10^{-11}	2.1×10^{-11}		
Tc-104	0.303 h	F	0.800	2.4×10^{-11}	3.9×10^{-11}	0.800	8.1×10^{-11}
		M	0.800	3.0×10^{-11}	4.8×10^{-11}		
Ruthenium							
Ru-94	0.863 h	F	0.050	2.7×10^{-11}	4.9×10^{-11}	0.050	9.4×10^{-11}
		M	0.050	4.4×10^{-11}	7.2×10^{-11}		
		S	0.050	4.6×10^{-11}	7.4×10^{-11}		
Ru-97	2.90 d	F	0.050	6.7×10^{-11}	1.2×10^{-10}	0.050	1.5×10^{-10}
		M	0.050	1.1×10^{-10}	1.6×10^{-10}		
		S	0.050	1.1×10^{-10}	1.6×10^{-10}		
Ru-103	39.3 d	F	0.050	4.9×10^{-10}	6.8×10^{-10}	0.050	7.3×10^{-10}
		M	0.050	2.3×10^{-9}	1.9×10^{-9}		
		S	0.050	2.8×10^{-9}	2.2×10^{-9}		
Ru-105	4.44 h	F	0.050	7.1×10^{-11}	1.3×10^{-10}	0.050	2.6×10^{-10}
		M	0.050	1.7×10^{-10}	2.4×10^{-10}		
		S	0.050	1.8×10^{-10}	2.5×10^{-10}		
Ru-106	1.01 a	F	0.050	8.0×10^{-9}	9.8×10^{-9}	0.050	7.0×10^{-9}
		M	0.050	2.6×10^{-8}	1.7×10^{-8}		
		S	0.050	6.2×10^{-8}	3.5×10^{-8}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Rhodium							
Rh-99	16.0 d	F	0.050	3.3×10^{-10}	4.9×10^{-10}	0.050	5.1×10^{-10}
		M	0.050	7.3×10^{-10}	8.2×10^{-10}		
		S	0.050	8.3×10^{-10}	8.9×10^{-10}		
Rh-99m	4.70 h	F	0.050	3.0×10^{-11}	5.7×10^{-11}	0.050	6.6×10^{-11}
		M	0.050	4.1×10^{-11}	7.2×10^{-11}		
		S	0.050	4.3×10^{-11}	7.3×10^{-11}		
Rh-100	20.8 h	F	0.050	2.8×10^{-10}	5.1×10^{-10}	0.050	7.1×10^{-10}
		M	0.050	3.6×10^{-10}	6.2×10^{-10}		
		S	0.050	3.7×10^{-10}	6.3×10^{-10}		
Rh-101	3.20 a	F	0.050	1.4×10^{-9}	1.7×10^{-9}	0.050	5.5×10^{-10}
		M	0.050	2.2×10^{-9}	1.7×10^{-9}		
		S	0.050	5.0×10^{-9}	3.1×10^{-9}		
Rh-101m	4.34 d	F	0.050	1.0×10^{-10}	1.7×10^{-10}	0.050	2.2×10^{-10}
		M	0.050	2.0×10^{-10}	2.5×10^{-10}		
		S	0.050	2.1×10^{-10}	2.7×10^{-10}		
Rh-102	2.90 a	F	0.050	7.3×10^{-9}	8.9×10^{-9}	0.050	2.6×10^{-9}
		M	0.050	6.5×10^{-9}	5.0×10^{-9}		
		S	0.050	1.6×10^{-8}	9.0×10^{-9}		
Rh-102m	207 d	F	0.050	1.5×10^{-9}	1.9×10^{-9}	0.050	1.2×10^{-9}
		M	0.050	3.8×10^{-9}	2.7×10^{-9}		
		S	0.050	6.7×10^{-9}	4.2×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Rh-103m	0.935 h	F	0.050	8.6×10^{-13}	1.2×10^{-12}	0.050	3.8×10^{-12}
		M	0.050	2.3×10^{-12}	2.4×10^{-12}		
		S	0.050	2.5×10^{-12}	2.5×10^{-12}		
Rh-105	1.47 d	F	0.050	8.7×10^{-11}	1.5×10^{-10}	0.050	3.7×10^{-10}
		M	0.050	3.1×10^{-10}	4.1×10^{-10}		
		S	0.050	3.4×10^{-10}	4.4×10^{-10}		
Rh-106m	2.20 h	F	0.050	7.0×10^{-11}	1.3×10^{-10}	0.050	1.6×10^{-10}
		M	0.050	1.1×10^{-10}	1.8×10^{-10}		
		S	0.050	1.2×10^{-10}	1.9×10^{-10}		
Rh-107	0.362 h	F	0.050	9.6×10^{-12}	1.6×10^{-11}	0.050	2.4×10^{-11}
		M	0.050	1.7×10^{-11}	2.7×10^{-11}		
		S	0.050	1.7×10^{-11}	2.8×10^{-11}		
Palladium							
Pd-100	3.63 d	F	0.005	4.9×10^{-10}	7.6×10^{-10}	0.005	9.4×10^{-10}
		M	0.005	7.9×10^{-10}	9.5×10^{-10}		
		S	0.005	8.3×10^{-10}	9.7×10^{-10}		
Pd-101	8.27 h	F	0.005	4.2×10^{-11}	7.5×10^{-11}	0.005	9.4×10^{-11}
		M	0.005	6.2×10^{-11}	9.8×10^{-11}		
		S	0.005	6.4×10^{-11}	1.0×10^{-10}		
Pd-103	17.0 d	F	0.005	9.0×10^{-11}	1.2×10^{-10}	0.005	1.9×10^{-10}
		M	0.005	3.5×10^{-10}	3.0×10^{-10}		
		S	0.005	4.0×10^{-10}	2.9×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Pd-107	6.50×10^6 a	F	0.005	2.6×10^{-11}	3.3×10^{-11}	0.005	3.7×10^{-11}
		M	0.005	8.0×10^{-11}	5.2×10^{-11}		
		S	0.005	5.5×10^{-10}	2.9×10^{-10}		
Pd-109	13.4 h	F	0.005	1.2×10^{-10}	2.1×10^{-10}	0.005	5.5×10^{-10}
		M	0.005	3.4×10^{-10}	4.7×10^{-10}		
		S	0.005	3.6×10^{-10}	5.0×10^{-10}		
Silver							
Ag-102	0.215 h	F	0.050	1.4×10^{-11}	2.4×10^{-11}	0.050	4.0×10^{-11}
		M	0.050	1.8×10^{-11}	3.2×10^{-11}		
		S	0.050	1.9×10^{-11}	3.2×10^{-11}		
Ag-103	1.09 h	F	0.050	1.6×10^{-11}	2.8×10^{-11}	0.050	4.3×10^{-11}
		M	0.050	2.7×10^{-11}	4.3×10^{-11}		
		S	0.050	2.8×10^{-11}	4.5×10^{-11}		
Ag-104	1.15 h	F	0.050	3.0×10^{-11}	5.7×10^{-11}	0.050	6.0×10^{-11}
		M	0.050	3.9×10^{-11}	6.9×10^{-11}		
		S	0.050	4.0×10^{-11}	7.1×10^{-11}		
Ag-104m	0.558 h	F	0.050	1.7×10^{-11}	3.1×10^{-11}	0.050	5.4×10^{-11}
		M	0.050	2.6×10^{-11}	4.4×10^{-11}		
		S	0.050	2.7×10^{-11}	4.5×10^{-11}		
Ag-105	41.0 d	F	0.050	5.4×10^{-10}	8.0×10^{-10}	0.050	4.7×10^{-10}
		M	0.050	6.9×10^{-10}	7.0×10^{-10}		
		S	0.050	7.8×10^{-10}	7.3×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Ag-106	0.399 h	F	0.050	9.8×10^{-12}	1.7×10^{-11}	0.050	3.2×10^{-11}
		M	0.050	1.6×10^{-11}	2.6×10^{-11}		
		S	0.050	1.6×10^{-11}	2.7×10^{-11}		
Ag-106m	8.41 d	F	0.050	1.1×10^{-9}	1.6×10^{-9}	0.050	1.5×10^{-9}
		M	0.050	1.1×10^{-9}	1.5×10^{-9}		
		S	0.050	1.1×10^{-9}	1.4×10^{-9}		
Ag-108m	1.27×10^2 a	F	0.050	6.1×10^{-9}	7.3×10^{-9}	0.050	2.3×10^{-9}
		M	0.050	7.0×10^{-9}	5.2×10^{-9}		
		S	0.050	3.5×10^{-8}	1.9×10^{-8}		
Ag-110m	250 d	F	0.050	5.5×10^{-9}	6.7×10^{-9}	0.050	2.8×10^{-9}
		M	0.050	7.2×10^{-9}	5.9×10^{-9}		
		S	0.050	1.2×10^{-8}	7.3×10^{-9}		
Ag-111	7.45 d	F	0.050	4.1×10^{-10}	5.7×10^{-10}	0.050	1.3×10^{-9}
		M	0.050	1.5×10^{-9}	1.5×10^{-9}		
		S	0.050	1.7×10^{-9}	1.6×10^{-9}		
Ag-112	3.12 h	F	0.050	8.2×10^{-11}	1.4×10^{-10}	0.050	4.3×10^{-10}
		M	0.050	1.7×10^{-10}	2.5×10^{-10}		
		S	0.050	1.8×10^{-10}	2.6×10^{-10}		
Ag-115	0.333 h	F	0.050	1.6×10^{-11}	2.6×10^{-11}	0.050	6.0×10^{-11}
		M	0.050	2.8×10^{-11}	4.3×10^{-11}		
		S	0.050	3.0×10^{-11}	4.4×10^{-11}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\ \mu m}$	$e(g)_{5\ \mu m}$	f_i	$e(g)$
Cadmium							
Cd-104	0.961 h	F	0.050	2.7×10^{-11}	5.0×10^{-11}	0.050	5.8×10^{-11}
		M	0.050	3.6×10^{-11}	6.2×10^{-11}		
		S	0.050	3.7×10^{-11}	6.3×10^{-11}		
Cd-107	6.49 h	F	0.050	2.3×10^{-11}	4.2×10^{-11}	0.050	6.2×10^{-11}
		M	0.050	8.1×10^{-11}	1.0×10^{-10}		
		S	0.050	8.7×10^{-11}	1.1×10^{-10}		
Cd-109	1.27 a	F	0.050	8.1×10^{-9}	9.6×10^{-9}	0.050	2.0×10^{-9}
		M	0.050	6.2×10^{-9}	5.1×10^{-9}		
		S	0.050	5.8×10^{-9}	4.4×10^{-9}		
Cd-113	9.30×10^{15} a	F	0.050	1.2×10^{-7}	1.4×10^{-7}	0.050	2.5×10^{-8}
		M	0.050	5.3×10^{-8}	4.3×10^{-8}		
		S	0.050	2.5×10^{-8}	2.1×10^{-8}		
Cd-113m	13.6 a	F	0.050	1.1×10^{-7}	1.3×10^{-7}	0.050	2.3×10^{-8}
		M	0.050	5.0×10^{-8}	4.0×10^{-8}		
		S	0.050	3.0×10^{-8}	2.4×10^{-8}		
Cd-115	2.23 d	F	0.050	3.7×10^{-10}	5.4×10^{-10}	0.050	1.4×10^{-9}
		M	0.050	9.7×10^{-10}	1.2×10^{-9}		
		S	0.050	1.1×10^{-9}	1.3×10^{-9}		
Cd-115m	44.6 d	F	0.050	5.3×10^{-9}	6.4×10^{-9}	0.050	3.3×10^{-9}
		M	0.050	5.9×10^{-9}	5.5×10^{-9}		
		S	0.050	7.3×10^{-9}	5.5×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Cd-117	2.49 h	F	0.050	7.3×10^{-11}	1.3×10^{-10}	0.050	2.8×10^{-10}
		M	0.050	1.6×10^{-10}	2.4×10^{-10}		
		S	0.050	1.7×10^{-10}	2.5×10^{-10}		
Cd-117m	3.36 h	F	0.050	1.0×10^{-10}	1.9×10^{-10}	0.050	2.8×10^{-10}
		M	0.050	2.0×10^{-10}	3.1×10^{-10}		
		S	0.050	2.1×10^{-10}	3.2×10^{-10}		
Indium							
In-109	4.20 h	F	0.020	3.2×10^{-11}	5.7×10^{-11}	0.020	6.6×10^{-11}
		M	0.020	4.4×10^{-11}	7.3×10^{-11}		
In-110	4.90 h	F	0.020	1.2×10^{-10}	2.2×10^{-10}	0.020	2.4×10^{-10}
		M	0.020	1.4×10^{-10}	2.5×10^{-10}		
In-110m	1.15 h	F	0.020	3.1×10^{-11}	5.5×10^{-11}	0.020	1.0×10^{-10}
		M	0.020	5.0×10^{-11}	8.1×10^{-11}		
In-111	2.83 d	F	0.020	1.3×10^{-10}	2.2×10^{-10}	0.020	2.9×10^{-10}
		M	0.020	2.3×10^{-10}	3.1×10^{-10}		
In-112	0.240 h	F	0.020	5.0×10^{-12}	8.6×10^{-12}	0.020	1.0×10^{-11}
		M	0.020	7.8×10^{-12}	1.3×10^{-11}		
In-113m	1.66 h	F	0.020	1.0×10^{-11}	1.9×10^{-11}	0.020	2.8×10^{-11}
		M	0.020	2.0×10^{-11}	3.2×10^{-11}		
In-114m	49.5 d	F	0.020	9.3×10^{-9}	1.1×10^{-8}	0.020	4.1×10^{-9}
		M	0.020	5.9×10^{-9}	5.9×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
In-115	5.10×10^{15} a	F	0.020	3.9×10^{-7}	4.5×10^{-7}	0.020	3.2×10^{-8}
		M	0.020	1.5×10^{-7}	1.1×10^{-7}		
In-115m	4.49 h	F	0.020	2.5×10^{-11}	4.5×10^{-11}	0.020	8.6×10^{-11}
		M	0.020	6.0×10^{-11}	8.7×10^{-11}		
In-116m	0.902 h	F	0.020	3.0×10^{-11}	5.5×10^{-11}	0.020	6.4×10^{-11}
		M	0.020	4.8×10^{-11}	8.0×10^{-11}		
In-117	0.730 h	F	0.020	1.6×10^{-11}	2.8×10^{-11}	0.020	3.1×10^{-11}
		M	0.020	3.0×10^{-11}	4.8×10^{-11}		
In-117m	1.94 h	F	0.020	3.1×10^{-11}	5.5×10^{-11}	0.020	1.2×10^{-10}
		M	0.020	7.3×10^{-11}	1.1×10^{-10}		
In-119m	0.300 h	F	0.020	1.1×10^{-11}	1.8×10^{-11}	0.020	4.7×10^{-11}
		M	0.020	1.8×10^{-11}	2.9×10^{-11}		
Tin							
Sn-110	4.00 h	F	0.020	1.1×10^{-10}	1.9×10^{-10}	0.020	3.5×10^{-10}
		M	0.020	1.6×10^{-10}	2.6×10^{-10}		
Sn-111	0.588 h	F	0.020	8.3×10^{-12}	1.5×10^{-11}	0.020	2.3×10^{-11}
		M	0.020	1.4×10^{-11}	2.2×10^{-11}		
Sn-113	115 d	F	0.020	5.4×10^{-10}	7.9×10^{-10}	0.020	7.3×10^{-10}
		M	0.020	2.5×10^{-9}	1.9×10^{-9}		
Sn-117m	13.6 d	F	0.020	2.9×10^{-10}	3.9×10^{-10}	0.020	7.1×10^{-10}
		M	0.020	2.3×10^{-9}	2.2×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Sn-119m	293 d	F	0.020	2.9×10^{-10}	3.6×10^{-10}	0.020	3.4×10^{-10}
		M	0.020	2.0×10^{-9}	1.5×10^{-9}		
Sn-121	1.13 d	F	0.020	6.4×10^{-11}	1.0×10^{-10}	0.020	2.3×10^{-10}
		M	0.020	2.2×10^{-10}	2.8×10^{-10}		
Sn-121m	55.0 a	F	0.020	8.0×10^{-10}	9.7×10^{-10}	0.020	3.8×10^{-10}
		M	0.020	4.2×10^{-9}	3.3×10^{-9}		
Sn-123	129 d	F	0.020	1.2×10^{-9}	1.6×10^{-9}	0.020	2.1×10^{-9}
		M	0.020	7.7×10^{-9}	5.6×10^{-9}		
Sn-123m	0.668 h	F	0.020	1.4×10^{-11}	2.4×10^{-11}	0.020	3.8×10^{-11}
		M	0.020	2.8×10^{-11}	4.4×10^{-11}		
Sn-125	9.64 d	F	0.020	9.2×10^{-10}	1.3×10^{-9}	0.020	3.1×10^{-9}
		M	0.020	3.0×10^{-9}	2.8×10^{-9}		
Sn-126	1.00×10^5 a	F	0.020	1.1×10^{-8}	1.4×10^{-8}	0.020	4.7×10^{-9}
		M	0.020	2.7×10^{-8}	1.8×10^{-8}		
Sn-127	2.10 h	F	0.020	6.9×10^{-11}	1.2×10^{-10}	0.020	2.0×10^{-10}
		M	0.020	1.3×10^{-10}	2.0×10^{-10}		
Sn-128	0.985 h	F	0.020	5.4×10^{-11}	9.5×10^{-11}	0.020	1.5×10^{-10}
		M	0.020	9.6×10^{-11}	1.5×10^{-10}		
Antimony							
Sb-115	0.530 h	F	0.100	9.2×10^{-12}	1.7×10^{-11}	0.100	2.4×10^{-11}
		M	0.010	1.4×10^{-11}	2.3×10^{-11}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Sb-116	0.263 h	F	0.100	9.9×10^{-12}	1.8×10^{-11}	0.100	2.6×10^{-11}
		M	0.010	1.4×10^{-11}	2.3×10^{-11}		
Sb-116m	1.00 h	F	0.100	3.5×10^{-11}	6.4×10^{-11}	0.100	6.7×10^{-11}
		M	0.010	5.0×10^{-11}	8.5×10^{-11}		
Sb-117	2.80 h	F	0.100	9.3×10^{-12}	1.7×10^{-11}	0.100	1.8×10^{-11}
		M	0.010	1.7×10^{-11}	2.7×10^{-11}		
Sb-118m	5.00 h	F	0.100	1.0×10^{-10}	1.9×10^{-10}	0.100	2.1×10^{-10}
		M	0.010	1.3×10^{-10}	2.3×10^{-10}		
Sb-119	1.59 d	F	0.100	2.5×10^{-11}	4.5×10^{-11}	0.100	8.1×10^{-11}
		M	0.010	3.7×10^{-11}	5.9×10^{-11}		
Sb-120	0.265 h	F	0.100	4.9×10^{-12}	8.5×10^{-12}	0.100	1.4×10^{-11}
		M	0.010	7.4×10^{-12}	1.2×10^{-11}		
Sb-120m	5.76 d	F	0.100	5.9×10^{-10}	9.8×10^{-10}	0.100	1.2×10^{-9}
		M	0.010	1.0×10^{-9}	1.3×10^{-9}		
Sb-122	2.70 d	F	0.100	3.9×10^{-10}	6.3×10^{-10}	0.100	1.7×10^{-9}
		M	0.010	1.0×10^{-9}	1.2×10^{-9}		
Sb-124	60.2 d	F	0.100	1.3×10^{-9}	1.9×10^{-9}	0.100	2.5×10^{-9}
		M	0.010	6.1×10^{-9}	4.7×10^{-9}		
Sb-124m	0.337 h	F	0.100	3.0×10^{-12}	5.3×10^{-12}	0.100	8.0×10^{-12}
		M	0.010	5.5×10^{-12}	8.3×10^{-12}		
Sb-125	2.77 a	F	0.100	1.4×10^{-9}	1.7×10^{-9}	0.100	1.1×10^{-9}
		M	0.010	4.5×10^{-9}	3.3×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Sb-126	12.4 d	F	0.100	1.1×10^{-9}	1.7×10^{-9}	0.100	2.4×10^{-9}
		M	0.010	2.7×10^{-9}	3.2×10^{-9}		
Sb-126m	0.317 h	F	0.100	1.3×10^{-11}	2.3×10^{-11}	0.100	3.6×10^{-11}
		M	0.010	2.0×10^{-11}	3.3×10^{-11}		
Sb-127	3.85 d	F	0.100	4.6×10^{-10}	7.4×10^{-10}	0.100	1.7×10^{-9}
		M	0.010	1.6×10^{-9}	1.7×10^{-9}		
Sb-128	9.01 h	F	0.100	2.5×10^{-10}	4.6×10^{-10}	0.100	7.6×10^{-10}
		M	0.010	4.2×10^{-10}	6.7×10^{-10}		
Sb-128m	0.173 h	F	0.100	1.1×10^{-11}	1.9×10^{-11}	0.100	3.3×10^{-11}
		M	0.010	1.5×10^{-11}	2.6×10^{-11}		
Sb-129	4.32 h	F	0.100	1.1×10^{-10}	2.0×10^{-10}	0.100	4.2×10^{-10}
		M	0.010	2.4×10^{-10}	3.5×10^{-10}		
Sb-130	0.667 h	F	0.100	3.5×10^{-11}	6.3×10^{-11}	0.100	9.1×10^{-11}
		M	0.010	5.4×10^{-11}	9.1×10^{-11}		
Sb-131	0.383 h	F	0.100	3.7×10^{-11}	5.9×10^{-11}	0.100	1.0×10^{-10}
		M	0.010	5.2×10^{-11}	8.3×10^{-11}		
Tellurium							
Te-116	2.49 h	F	0.300	6.3×10^{-11}	1.2×10^{-10}	0.300	1.7×10^{-10}
		M	0.300	1.1×10^{-10}	1.7×10^{-10}		
Te-121	17.0 d	F	0.300	2.5×10^{-10}	3.9×10^{-10}	0.300	4.3×10^{-10}
		M	0.300	3.9×10^{-10}	4.4×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Te-121m	154 d	F	0.300	1.8×10^{-9}	2.3×10^{-9}	0.300	2.3×10^{-9}
		M	0.300	4.2×10^{-9}	3.6×10^{-9}		
Te-123	1.00×10^{13} a	F	0.300	4.0×10^{-9}	5.0×10^{-9}	0.300	4.4×10^{-9}
		M	0.300	2.6×10^{-9}	2.8×10^{-9}		
Te-123m	120 d	F	0.300	9.7×10^{-10}	1.2×10^{-9}	0.300	1.4×10^{-9}
		M	0.300	3.9×10^{-9}	3.4×10^{-9}		
Te-125m	58.0 d	F	0.300	5.1×10^{-10}	6.7×10^{-10}	0.300	8.7×10^{-10}
		M	0.300	3.3×10^{-9}	2.9×10^{-9}		
Te-127	9.35 h	F	0.300	4.2×10^{-11}	7.2×10^{-11}	0.300	1.7×10^{-10}
		M	0.300	1.2×10^{-10}	1.8×10^{-10}		
Te-127m	109 d	F	0.300	1.6×10^{-9}	2.0×10^{-9}	0.300	2.3×10^{-9}
		M	0.300	7.2×10^{-9}	6.2×10^{-9}		
Te-129	1.16 h	F	0.300	1.7×10^{-11}	2.9×10^{-11}	0.300	6.3×10^{-11}
		M	0.300	3.8×10^{-11}	5.7×10^{-11}		
Te-129m	33.6 d	F	0.300	1.3×10^{-9}	1.8×10^{-9}	0.300	3.0×10^{-9}
		M	0.300	6.3×10^{-9}	5.4×10^{-9}		
Te-131	0.417 h	F	0.300	2.3×10^{-11}	4.6×10^{-11}	0.300	8.7×10^{-11}
		M	0.300	3.8×10^{-11}	6.1×10^{-11}		
Te-131m	1.25 d	F	0.300	8.7×10^{-10}	1.2×10^{-9}	0.300	1.9×10^{-9}
		M	0.300	1.1×10^{-9}	1.6×10^{-9}		
Te-132	3.26 d	F	0.300	1.8×10^{-9}	2.4×10^{-9}	0.300	3.7×10^{-9}
		M	0.300	2.2×10^{-9}	3.0×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Te-133	0.207 h	F	0.300	2.0×10^{-11}	3.8×10^{-11}	0.300	7.2×10^{-11}
		M	0.300	2.7×10^{-11}	4.4×10^{-11}		
Te-133m	0.923 h	F	0.300	8.4×10^{-11}	1.2×10^{-10}	0.300	2.8×10^{-10}
		M	0.300	1.2×10^{-10}	1.9×10^{-10}		
Te-134	0.696 h	F	0.300	5.0×10^{-11}	8.3×10^{-11}	0.300	1.1×10^{-10}
		M	0.300	7.1×10^{-11}	1.1×10^{-10}		
Iodine							
I-120	1.35 h	F	1.000	1.0×10^{-10}	1.9×10^{-10}	1.000	3.4×10^{-10}
I-120m	0.883 h	F	1.000	8.7×10^{-11}	1.4×10^{-10}	1.000	2.1×10^{-10}
I-121	2.12 h	F	1.000	2.8×10^{-11}	3.9×10^{-11}	1.000	8.2×10^{-11}
I-123	13.2 h	F	1.000	7.6×10^{-11}	1.1×10^{-10}	1.000	2.1×10^{-10}
I-124	4.18 d	F	1.000	4.5×10^{-9}	6.3×10^{-9}	1.000	1.3×10^{-8}
I-125	60.1 d	F	1.000	5.3×10^{-9}	7.3×10^{-9}	1.000	1.5×10^{-8}
I-126	13.0 d	F	1.000	1.0×10^{-8}	1.4×10^{-8}	1.000	2.9×10^{-8}
I-128	0.416 h	F	1.000	1.4×10^{-11}	2.2×10^{-11}	1.000	4.6×10^{-11}
I-129	1.57×10^7 a	F	1.000	3.7×10^{-8}	5.1×10^{-8}	1.000	1.1×10^{-7}
I-130	12.4 h	F	1.000	6.9×10^{-10}	9.6×10^{-10}	1.000	2.0×10^{-9}
I-131	8.04 d	F	1.000	7.6×10^{-9}	1.1×10^{-8}	1.000	2.2×10^{-8}
I-132	2.30 h	F	1.000	9.6×10^{-11}	2.0×10^{-10}	1.000	2.9×10^{-10}
I-132m	1.39 h	F	1.000	8.1×10^{-11}	1.1×10^{-10}	1.000	2.2×10^{-10}
I-133	20.8 h	F	1.000	1.5×10^{-9}	2.1×10^{-9}	1.000	4.3×10^{-9}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
I-134	0.876 h	F	1.000	4.8×10^{-11}	7.9×10^{-11}	1.000	1.1×10^{-10}
I-135	6.61 h	F	1.000	3.3×10^{-10}	4.6×10^{-10}	1.000	9.3×10^{-10}
Caesium							
Cs-125	0.750 h	F	1.000	1.3×10^{-11}	2.3×10^{-11}	1.000	3.5×10^{-11}
Cs-127	6.25 h	F	1.000	2.2×10^{-11}	4.0×10^{-11}	1.000	2.4×10^{-11}
Cs-129	1.34 d	F	1.000	4.5×10^{-11}	8.1×10^{-11}	1.000	6.0×10^{-11}
Cs-130	0.498 h	F	1.000	8.4×10^{-12}	1.5×10^{-11}	1.000	2.8×10^{-11}
Cs-131	9.69 d	F	1.000	2.8×10^{-11}	4.5×10^{-11}	1.000	5.8×10^{-11}
Cs-132	6.48 d	F	1.000	2.4×10^{-10}	3.8×10^{-10}	1.000	5.0×10^{-10}
Cs-134	2.06 a	F	1.000	6.8×10^{-9}	9.6×10^{-9}	1.000	1.9×10^{-8}
Cs-134m	2.90 h	F	1.000	1.5×10^{-11}	2.6×10^{-11}	1.000	2.0×10^{-11}
Cs-135	2.30×10^6 a	F	1.000	7.1×10^{-10}	9.9×10^{-10}	1.000	2.0×10^{-9}
Cs-135m	0.883 h	F	1.000	1.3×10^{-11}	2.4×10^{-11}	1.000	1.9×10^{-11}
Cs-136	13.1 d	F	1.000	1.3×10^{-9}	1.9×10^{-9}	1.000	3.0×10^{-9}
Cs-137	30.0 a	F	1.000	4.8×10^{-9}	6.7×10^{-9}	1.000	1.3×10^{-8}
Cs-138	0.536 h	F	1.000	2.6×10^{-11}	4.6×10^{-11}	1.000	9.2×10^{-11}
Barium							
Ba-126	1.61 h	F	0.100	7.8×10^{-11}	1.2×10^{-10}	0.100	2.6×10^{-10}
Ba-128	2.43 d	F	0.100	8.0×10^{-10}	1.3×10^{-9}	0.100	2.7×10^{-9}
Ba-131	11.8 d	F	0.100	2.3×10^{-10}	3.5×10^{-10}	0.100	4.5×10^{-10}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Ba-131m	0.243 h	F	0.100	4.1×10^{-12}	6.4×10^{-12}	0.100	4.9×10^{-12}
Ba-133	10.7 a	F	0.100	1.5×10^{-9}	1.8×10^{-9}	0.100	1.0×10^{-9}
Ba-133m	1.62 d	F	0.100	1.9×10^{-10}	2.8×10^{-10}	0.100	5.5×10^{-10}
Ba-135m	1.20 d	F	0.100	1.5×10^{-10}	2.3×10^{-10}	0.100	4.5×10^{-10}
Ba-139	1.38 h	F	0.100	3.5×10^{-11}	5.5×10^{-11}	0.100	1.2×10^{-10}
Ba-140	12.7 d	F	0.100	1.0×10^{-9}	1.6×10^{-9}	0.100	2.5×10^{-9}
Ba-141	0.305 h	F	0.100	2.2×10^{-11}	3.5×10^{-11}	0.100	7.0×10^{-11}
Ba-142	0.177 h	F	0.100	1.6×10^{-11}	2.7×10^{-11}	0.100	3.5×10^{-11}
Lanthanum							
La-131	0.983 h	F	5.0×10^{-4}	1.4×10^{-11}	2.4×10^{-11}	5.0×10^{-4}	3.5×10^{-11}
		M	5.0×10^{-4}	2.3×10^{-11}	3.6×10^{-11}		
La-132	4.80 h	F	5.0×10^{-4}	1.1×10^{-10}	2.0×10^{-10}	5.0×10^{-4}	3.9×10^{-10}
		M	5.0×10^{-4}	1.7×10^{-10}	2.8×10^{-10}		
La-135	19.5 h	F	5.0×10^{-4}	1.1×10^{-11}	2.0×10^{-11}	5.0×10^{-4}	3.0×10^{-11}
		M	5.0×10^{-4}	1.5×10^{-11}	2.5×10^{-11}		
La-137	6.00×10^4 a	F	5.0×10^{-4}	8.6×10^{-9}	1.0×10^{-8}	5.0×10^{-4}	8.1×10^{-11}
		M	5.0×10^{-4}	3.4×10^{-9}	2.3×10^{-9}		
La-138	1.35×10^{11} a	F	5.0×10^{-4}	1.5×10^{-7}	1.8×10^{-7}	5.0×10^{-4}	1.1×10^{-9}
		M	5.0×10^{-4}	6.1×10^{-8}	4.2×10^{-8}		
La-140	1.68 d	F	5.0×10^{-4}	6.0×10^{-10}	1.0×10^{-9}	5.0×10^{-4}	2.0×10^{-9}
		M	5.0×10^{-4}	1.1×10^{-9}	1.5×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
La-141	3.93 h	F	5.0×10^{-4}	6.7×10^{-11}	1.1×10^{-10}	5.0×10^{-4}	3.6×10^{-10}
		M	5.0×10^{-4}	1.5×10^{-10}	2.2×10^{-10}		
La-142	1.54 h	F	5.0×10^{-4}	5.6×10^{-11}	1.0×10^{-10}	5.0×10^{-4}	1.8×10^{-10}
		M	5.0×10^{-4}	9.3×10^{-11}	1.5×10^{-10}		
La-143	0.237 h	F	5.0×10^{-4}	1.2×10^{-11}	2.0×10^{-11}	5.0×10^{-4}	5.6×10^{-11}
		M	5.0×10^{-4}	2.2×10^{-11}	3.3×10^{-11}		
Cerium							
Ce-134	3.00 d	M	5.0×10^{-4}	1.3×10^{-9}	1.5×10^{-9}	5.0×10^{-4}	2.5×10^{-9}
		S	5.0×10^{-4}	1.3×10^{-9}	1.6×10^{-9}		
Ce-135	17.6 h	M	5.0×10^{-4}	4.9×10^{-10}	7.3×10^{-10}	5.0×10^{-4}	7.9×10^{-10}
		S	5.0×10^{-4}	5.1×10^{-10}	7.6×10^{-10}		
Ce-137	9.00 h	M	5.0×10^{-4}	1.0×10^{-11}	1.8×10^{-11}	5.0×10^{-4}	2.5×10^{-11}
		S	5.0×10^{-4}	1.1×10^{-11}	1.9×10^{-11}		
Ce-137m	1.43 d	M	5.0×10^{-4}	4.0×10^{-10}	5.5×10^{-10}	5.0×10^{-4}	5.4×10^{-10}
		S	5.0×10^{-4}	4.3×10^{-10}	5.9×10^{-10}		
Ce-139	138 d	M	5.0×10^{-4}	1.6×10^{-9}	1.3×10^{-9}	5.0×10^{-4}	2.6×10^{-10}
		S	5.0×10^{-4}	1.8×10^{-9}	1.4×10^{-9}		
Ce-141	32.5 d	M	5.0×10^{-4}	3.1×10^{-9}	2.7×10^{-9}	5.0×10^{-4}	7.1×10^{-10}
		S	5.0×10^{-4}	3.6×10^{-9}	3.1×10^{-9}		
Ce-143	1.38 d	M	5.0×10^{-4}	7.4×10^{-10}	9.5×10^{-10}	5.0×10^{-4}	1.1×10^{-9}
		S	5.0×10^{-4}	8.1×10^{-10}	1.0×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Ce-144	284 d	M	5.0×10^{-4}	3.4×10^{-8}	2.3×10^{-8}	5.0×10^{-4}	5.2×10^{-9}
		S	5.0×10^{-4}	4.9×10^{-8}	2.9×10^{-8}		
Praseodymium							
Pr-136	0.218 h	M	5.0×10^{-4}	1.4×10^{-11}	2.4×10^{-11}	5.0×10^{-4}	3.3×10^{-11}
		S	5.0×10^{-4}	1.5×10^{-11}	2.5×10^{-11}		
Pr-137	1.28 h	M	5.0×10^{-4}	2.1×10^{-11}	3.4×10^{-11}	5.0×10^{-4}	4.0×10^{-11}
		S	5.0×10^{-4}	2.2×10^{-11}	3.5×10^{-11}		
Pr-138m	2.10 h	M	5.0×10^{-4}	7.6×10^{-11}	1.3×10^{-10}	5.0×10^{-4}	1.3×10^{-10}
		S	5.0×10^{-4}	7.9×10^{-11}	1.3×10^{-10}		
Pr-139	4.51 h	M	5.0×10^{-4}	1.9×10^{-11}	2.9×10^{-11}	5.0×10^{-4}	3.1×10^{-11}
		S	5.0×10^{-4}	2.0×10^{-11}	3.0×10^{-11}		
Pr-142	19.1 h	M	5.0×10^{-4}	5.3×10^{-10}	7.0×10^{-10}	5.0×10^{-4}	1.3×10^{-9}
		S	5.0×10^{-4}	5.6×10^{-10}	7.4×10^{-10}		
Pr-142m	0.243 h	M	5.0×10^{-4}	6.7×10^{-12}	8.9×10^{-12}	5.0×10^{-4}	1.7×10^{-11}
		S	5.0×10^{-4}	7.1×10^{-12}	9.4×10^{-12}		
Pr-143	13.6 d	M	5.0×10^{-4}	2.1×10^{-9}	1.9×10^{-9}	5.0×10^{-4}	1.2×10^{-9}
		S	5.0×10^{-4}	2.3×10^{-9}	2.2×10^{-9}		
Pr-144	0.288 h	M	5.0×10^{-4}	1.8×10^{-11}	2.9×10^{-11}	5.0×10^{-4}	5.0×10^{-11}
		S	5.0×10^{-4}	1.9×10^{-11}	3.0×10^{-11}		
Pr-145	5.98 h	M	5.0×10^{-4}	1.6×10^{-10}	2.5×10^{-10}	5.0×10^{-4}	3.9×10^{-10}
		S	5.0×10^{-4}	1.7×10^{-10}	2.6×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Pr-147	0.227 h	M	5.0×10^{-4}	1.8×10^{-11}	2.9×10^{-11}	5.0×10^{-4}	3.3×10^{-11}
		S	5.0×10^{-4}	1.9×10^{-11}	3.0×10^{-11}		
Neodymium							
Nd-136	0.844 h	M	5.0×10^{-4}	5.3×10^{-11}	8.5×10^{-11}	5.0×10^{-4}	9.9×10^{-11}
		S	5.0×10^{-4}	5.6×10^{-11}	8.9×10^{-11}		
Nd-138	5.04 h	M	5.0×10^{-4}	2.4×10^{-10}	3.7×10^{-10}	5.0×10^{-4}	6.4×10^{-10}
		S	5.0×10^{-4}	2.6×10^{-10}	3.8×10^{-10}		
Nd-139	0.495 h	M	5.0×10^{-4}	1.0×10^{-11}	1.7×10^{-11}	5.0×10^{-4}	2.0×10^{-11}
		S	5.0×10^{-4}	1.1×10^{-11}	1.7×10^{-11}		
Nd-139m	5.50 h	M	5.0×10^{-4}	1.5×10^{-10}	2.5×10^{-10}	5.0×10^{-4}	2.5×10^{-10}
		S	5.0×10^{-4}	1.6×10^{-10}	2.5×10^{-10}		
Nd-141	2.49 h	M	5.0×10^{-4}	5.1×10^{-12}	8.5×10^{-12}	5.0×10^{-4}	8.3×10^{-12}
		S	5.0×10^{-4}	5.3×10^{-12}	8.8×10^{-12}		
Nd-147	11.0 d	M	5.0×10^{-4}	2.0×10^{-9}	1.9×10^{-9}	5.0×10^{-4}	1.1×10^{-9}
		S	5.0×10^{-4}	2.3×10^{-9}	2.1×10^{-9}		
Nd-149	1.73 h	M	5.0×10^{-4}	8.5×10^{-11}	1.2×10^{-10}	5.0×10^{-4}	1.2×10^{-10}
		S	5.0×10^{-4}	9.0×10^{-11}	1.3×10^{-10}		
Nd-151	0.207 h	M	5.0×10^{-4}	1.7×10^{-11}	2.8×10^{-11}	5.0×10^{-4}	3.0×10^{-11}
		S	5.0×10^{-4}	1.8×10^{-11}	2.9×10^{-11}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Promethium							
Pm-141	0.348 h	M	5.0×10^{-4}	1.5×10^{-11}	2.4×10^{-11}	5.0×10^{-4}	3.6×10^{-11}
		S	5.0×10^{-4}	1.6×10^{-11}	2.5×10^{-11}		
Pm-143	265 d	M	5.0×10^{-4}	1.4×10^{-9}	9.6×10^{-10}	5.0×10^{-4}	2.3×10^{-10}
		S	5.0×10^{-4}	1.3×10^{-9}	8.3×10^{-10}		
Pm-144	363 d	M	5.0×10^{-4}	7.8×10^{-9}	5.4×10^{-9}	5.0×10^{-4}	9.7×10^{-10}
		S	5.0×10^{-4}	7.0×10^{-9}	3.9×10^{-9}		
Pm-145	17.7 a	M	5.0×10^{-4}	3.4×10^{-9}	2.4×10^{-9}	5.0×10^{-4}	1.1×10^{-10}
		S	5.0×10^{-4}	2.1×10^{-9}	1.2×10^{-9}		
Pm-146	5.53 a	M	5.0×10^{-4}	1.9×10^{-8}	1.3×10^{-8}	5.0×10^{-4}	9.0×10^{-10}
		S	5.0×10^{-4}	1.6×10^{-8}	9.0×10^{-9}		
Pm-147	2.62 a	M	5.0×10^{-4}	4.7×10^{-9}	3.5×10^{-9}	5.0×10^{-4}	2.6×10^{-10}
		S	5.0×10^{-4}	4.6×10^{-9}	3.2×10^{-9}		
Pm-148	5.37 d	M	5.0×10^{-4}	2.0×10^{-9}	2.1×10^{-9}	5.0×10^{-4}	2.7×10^{-9}
		S	5.0×10^{-4}	2.1×10^{-9}	2.2×10^{-9}		
Pm-148m	41.3 d	M	5.0×10^{-4}	4.9×10^{-9}	4.1×10^{-9}	5.0×10^{-4}	1.8×10^{-9}
		S	5.0×10^{-4}	5.4×10^{-9}	4.3×10^{-9}		
Pm-149	2.21 d	M	5.0×10^{-4}	6.6×10^{-10}	7.6×10^{-10}	5.0×10^{-4}	9.9×10^{-10}
		S	5.0×10^{-4}	7.2×10^{-10}	8.2×10^{-10}		
Pm-150	2.68 h	M	5.0×10^{-4}	1.3×10^{-10}	2.0×10^{-10}	5.0×10^{-4}	2.6×10^{-10}
		S	5.0×10^{-4}	1.4×10^{-10}	2.1×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Pm-151	1.18 d	M	5.0×10^{-4}	4.2×10^{-10}	6.1×10^{-10}	5.0×10^{-4}	7.3×10^{-10}
		S	5.0×10^{-4}	4.5×10^{-10}	6.4×10^{-10}		
Samarium							
Sm-141	0.170 h	M	5.0×10^{-4}	1.6×10^{-11}	2.7×10^{-11}	5.0×10^{-4}	3.9×10^{-11}
Sm-141m	0.377 h	M	5.0×10^{-4}	3.4×10^{-11}	5.6×10^{-11}	5.0×10^{-4}	6.5×10^{-11}
Sm-142	1.21 h	M	5.0×10^{-4}	7.4×10^{-11}	1.1×10^{-10}	5.0×10^{-4}	1.9×10^{-10}
Sm-145	340 d	M	5.0×10^{-4}	1.5×10^{-9}	1.1×10^{-9}	5.0×10^{-4}	2.1×10^{-10}
Sm-146	1.03×10^8 a	M	5.0×10^{-4}	9.9×10^{-6}	6.7×10^{-6}	5.0×10^{-4}	5.4×10^{-8}
Sm-147	1.06×10^{11} a	M	5.0×10^{-4}	8.9×10^{-6}	6.1×10^{-6}	5.0×10^{-4}	4.9×10^{-8}
Sm-151	90.0 a	M	5.0×10^{-4}	3.7×10^{-9}	2.6×10^{-9}	5.0×10^{-4}	9.8×10^{-11}
Sm-153	1.95 d	M	5.0×10^{-4}	6.1×10^{-10}	6.8×10^{-10}	5.0×10^{-4}	7.4×10^{-10}
Sm-155	0.368 h	M	5.0×10^{-4}	1.7×10^{-11}	2.8×10^{-11}	5.0×10^{-4}	2.9×10^{-11}
Sm-156	9.40 h	M	5.0×10^{-4}	2.1×10^{-10}	2.8×10^{-10}	5.0×10^{-4}	2.5×10^{-10}
Europium							
Eu-145	5.94 d	M	5.0×10^{-4}	5.6×10^{-10}	7.3×10^{-10}	5.0×10^{-4}	7.5×10^{-10}
Eu-146	4.61 d	M	5.0×10^{-4}	8.2×10^{-10}	1.2×10^{-9}	5.0×10^{-4}	1.3×10^{-9}
Eu-147	24.0 d	M	5.0×10^{-4}	1.0×10^{-9}	1.0×10^{-9}	5.0×10^{-4}	4.4×10^{-10}
Eu-148	54.5 d	M	5.0×10^{-4}	2.7×10^{-9}	2.3×10^{-9}	5.0×10^{-4}	1.3×10^{-9}
Eu-149	93.1 d	M	5.0×10^{-4}	2.7×10^{-10}	2.3×10^{-10}	5.0×10^{-4}	1.0×10^{-10}
Eu-150	34.2 a	M	5.0×10^{-4}	5.0×10^{-8}	3.4×10^{-8}	5.0×10^{-4}	1.3×10^{-9}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Eu-150m	12.6 h	M	5.0×10^{-4}	1.9×10^{-10}	2.8×10^{-10}	5.0×10^{-4}	3.8×10^{-10}
Eu-152	13.3 a	M	5.0×10^{-4}	3.9×10^{-8}	2.7×10^{-8}	5.0×10^{-4}	1.4×10^{-9}
Eu-152m	9.32 h	M	5.0×10^{-4}	2.2×10^{-10}	3.2×10^{-10}	5.0×10^{-4}	5.0×10^{-10}
Eu-154	8.80 a	M	5.0×10^{-4}	5.0×10^{-8}	3.5×10^{-8}	5.0×10^{-4}	2.0×10^{-9}
Eu-155	4.96 a	M	5.0×10^{-4}	6.5×10^{-9}	4.7×10^{-9}	5.0×10^{-4}	3.2×10^{-10}
Eu-156	15.2 d	M	5.0×10^{-4}	3.3×10^{-9}	3.0×10^{-9}	5.0×10^{-4}	2.2×10^{-9}
Eu-157	15.1 h	M	5.0×10^{-4}	3.2×10^{-10}	4.4×10^{-10}	5.0×10^{-4}	6.0×10^{-10}
Eu-158	0.765 h	M	5.0×10^{-4}	4.8×10^{-11}	7.5×10^{-11}	5.0×10^{-4}	9.4×10^{-11}
Gadolinium							
Gd-145	0.382 h	F	5.0×10^{-4}	1.5×10^{-11}	2.6×10^{-11}	5.0×10^{-4}	4.4×10^{-11}
		M	5.0×10^{-4}	2.1×10^{-11}	3.5×10^{-11}		
Gd-146	48.3 d	F	5.0×10^{-4}	4.4×10^{-9}	5.2×10^{-9}	5.0×10^{-4}	9.6×10^{-10}
		M	5.0×10^{-4}	6.0×10^{-9}	4.6×10^{-9}		
Gd-147	1.59 d	F	5.0×10^{-4}	2.7×10^{-10}	4.5×10^{-10}	5.0×10^{-4}	6.1×10^{-10}
		M	5.0×10^{-4}	4.1×10^{-10}	5.9×10^{-10}		
Gd-148	93.0 a	F	5.0×10^{-4}	2.5×10^{-5}	3.0×10^{-5}	5.0×10^{-4}	5.5×10^{-8}
		M	5.0×10^{-4}	1.1×10^{-5}	7.2×10^{-6}		
Gd-149	9.40 d	F	5.0×10^{-4}	2.6×10^{-10}	4.5×10^{-10}	5.0×10^{-4}	4.5×10^{-10}
		M	5.0×10^{-4}	7.0×10^{-10}	7.9×10^{-10}		
Gd-151	120 d	F	5.0×10^{-4}	7.8×10^{-10}	9.3×10^{-10}	5.0×10^{-4}	2.0×10^{-10}
		M	5.0×10^{-4}	8.1×10^{-10}	6.5×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Gd-152	1.08 × 10 ¹⁴ a	F	5.0 × 10 ⁻⁴	1.9 × 10 ⁻⁵	2.2 × 10 ⁻⁵	5.0 × 10 ⁻⁴	4.1 × 10 ⁻⁸
		M	5.0 × 10 ⁻⁴	7.4 × 10 ⁻⁶	5.0 × 10 ⁻⁶		
Gd-153	242 d	F	5.0 × 10 ⁻⁴	2.1 × 10 ⁻⁹	2.5 × 10 ⁻⁹	5.0 × 10 ⁻⁴	2.7 × 10 ⁻¹⁰
		M	5.0 × 10 ⁻⁴	1.9 × 10 ⁻⁹	1.4 × 10 ⁻⁹		
Gd-159	18.6 h	F	5.0 × 10 ⁻⁴	1.1 × 10 ⁻¹⁰	1.8 × 10 ⁻¹⁰	5.0 × 10 ⁻⁴	4.9 × 10 ⁻¹⁰
		M	5.0 × 10 ⁻⁴	2.7 × 10 ⁻¹⁰	3.9 × 10 ⁻¹⁰		
Terbium							
Tb-147	1.65 h	M	5.0 × 10 ⁻⁴	7.9 × 10 ⁻¹¹	1.2 × 10 ⁻¹⁰	5.0 × 10 ⁻⁴	1.6 × 10 ⁻¹⁰
Tb-149	4.15 h	M	5.0 × 10 ⁻⁴	4.3 × 10 ⁻⁹	3.1 × 10 ⁻⁹	5.0 × 10 ⁻⁴	2.5 × 10 ⁻¹⁰
Tb-150	3.27 h	M	5.0 × 10 ⁻⁴	1.1 × 10 ⁻¹⁰	1.8 × 10 ⁻¹⁰	5.0 × 10 ⁻⁴	2.5 × 10 ⁻¹⁰
Tb-151	17.6 h	M	5.0 × 10 ⁻⁴	2.3 × 10 ⁻¹⁰	3.3 × 10 ⁻¹⁰	5.0 × 10 ⁻⁴	3.4 × 10 ⁻¹⁰
Tb-153	2.34 d	M	5.0 × 10 ⁻⁴	2.0 × 10 ⁻¹⁰	2.4 × 10 ⁻¹⁰	5.0 × 10 ⁻⁴	2.5 × 10 ⁻¹⁰
Tb-154	21.4 h	M	5.0 × 10 ⁻⁴	3.8 × 10 ⁻¹⁰	6.0 × 10 ⁻¹⁰	5.0 × 10 ⁻⁴	6.5 × 10 ⁻¹⁰
Tb-155	5.32 d	M	5.0 × 10 ⁻⁴	2.1 × 10 ⁻¹⁰	2.5 × 10 ⁻¹⁰	5.0 × 10 ⁻⁴	2.1 × 10 ⁻¹⁰
Tb-156	5.34 d	M	5.0 × 10 ⁻⁴	1.2 × 10 ⁻⁹	1.4 × 10 ⁻⁹	5.0 × 10 ⁻⁴	1.2 × 10 ⁻⁹
Tb-156m	1.02 d	M	5.0 × 10 ⁻⁴	2.0 × 10 ⁻¹⁰	2.3 × 10 ⁻¹⁰	5.0 × 10 ⁻⁴	1.7 × 10 ⁻¹⁰
Tb-156m'	5.00 h	M	5.0 × 10 ⁻⁴	9.2 × 10 ⁻¹¹	1.3 × 10 ⁻¹⁰	5.0 × 10 ⁻⁴	8.1 × 10 ⁻¹¹
Tb-157	7.1 × 10 ¹ a	M	5.0 × 10 ⁻⁴	1.1 × 10 ⁻⁹	7.9 × 10 ⁻¹⁰	5.0 × 10 ⁻⁴	3.4 × 10 ⁻¹¹
Tb-158	1.80 × 10 ² a	M	5.0 × 10 ⁻⁴	4.3 × 10 ⁻⁸	3.0 × 10 ⁻⁸	5.0 × 10 ⁻⁴	1.1 × 10 ⁻⁹
Tb-160	72.3 d	M	5.0 × 10 ⁻⁴	6.6 × 10 ⁻⁹	5.4 × 10 ⁻⁹	5.0 × 10 ⁻⁴	1.6 × 10 ⁻⁹
Tb-161	6.91 d	M	5.0 × 10 ⁻⁴	1.2 × 10 ⁻⁹	1.2 × 10 ⁻⁹	5.0 × 10 ⁻⁴	7.2 × 10 ⁻¹⁰

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\ \mu m}$	$e(g)_{5\ \mu m}$	f_i	$e(g)$
Dysprosium							
Dy-155	10.0 h	M	5.0×10^{-4}	8.0×10^{-11}	1.2×10^{-10}	5.0×10^{-4}	1.3×10^{-10}
Dy-157	8.10 h	M	5.0×10^{-4}	3.2×10^{-11}	5.5×10^{-11}	5.0×10^{-4}	6.1×10^{-11}
Dy-159	144 d	M	5.0×10^{-4}	3.5×10^{-10}	2.5×10^{-10}	5.0×10^{-4}	1.0×10^{-10}
Dy-165	2.33 h	M	5.0×10^{-4}	6.1×10^{-11}	8.7×10^{-11}	5.0×10^{-4}	1.1×10^{-10}
Dy-166	3.40 d	M	5.0×10^{-4}	1.8×10^{-9}	1.8×10^{-9}	5.0×10^{-4}	1.6×10^{-9}
Holmium							
Ho-155	0.800 h	M	5.0×10^{-4}	2.0×10^{-11}	3.2×10^{-11}	5.0×10^{-4}	3.7×10^{-11}
Ho-157	0.210 h	M	5.0×10^{-4}	4.5×10^{-12}	7.6×10^{-12}	5.0×10^{-4}	6.5×10^{-12}
Ho-159	0.550 h	M	5.0×10^{-4}	6.3×10^{-12}	1.0×10^{-11}	5.0×10^{-4}	7.9×10^{-12}
Ho-161	2.50 h	M	5.0×10^{-4}	6.3×10^{-12}	1.0×10^{-11}	5.0×10^{-4}	1.3×10^{-11}
Ho-162	0.250 h	M	5.0×10^{-4}	2.9×10^{-12}	4.5×10^{-12}	5.0×10^{-4}	3.3×10^{-12}
Ho-162m	1.13 h	M	5.0×10^{-4}	2.2×10^{-11}	3.3×10^{-11}	5.0×10^{-4}	2.6×10^{-11}
Ho-164	0.483 h	M	5.0×10^{-4}	8.6×10^{-12}	1.3×10^{-11}	5.0×10^{-4}	9.5×10^{-12}
Ho-164m	0.625 h	M	5.0×10^{-4}	1.2×10^{-11}	1.6×10^{-11}	5.0×10^{-4}	1.6×10^{-11}
Ho-166	1.12 d	M	5.0×10^{-4}	6.6×10^{-10}	8.3×10^{-10}	5.0×10^{-4}	1.4×10^{-9}
Ho-166m	1.20×10^3 a	M	5.0×10^{-4}	1.1×10^{-7}	7.8×10^{-8}	5.0×10^{-4}	2.0×10^{-9}
Ho-167	3.10 h	M	5.0×10^{-4}	7.1×10^{-11}	1.0×10^{-10}	5.0×10^{-4}	8.3×10^{-11}
Erbium							
Er-161	3.24 h	M	5.0×10^{-4}	5.1×10^{-11}	8.5×10^{-11}	5.0×10^{-4}	8.0×10^{-11}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Er-165	10.4 h	M	5.0×10^{-4}	8.3×10^{-12}	1.4×10^{-11}	5.0×10^{-4}	1.9×10^{-11}
Er-169	9.30 d	M	5.0×10^{-4}	9.8×10^{-10}	9.2×10^{-10}	5.0×10^{-4}	3.7×10^{-10}
Er-171	7.52 h	M	5.0×10^{-4}	2.2×10^{-10}	3.0×10^{-10}	5.0×10^{-4}	3.6×10^{-10}
Er-172	2.05 d	M	5.0×10^{-4}	1.1×10^{-9}	1.2×10^{-9}	5.0×10^{-4}	1.0×10^{-9}
Thulium							
Tm-162	0.362 h	M	5.0×10^{-4}	1.6×10^{-11}	2.7×10^{-11}	5.0×10^{-4}	2.9×10^{-11}
Tm-166	7.70 h	M	5.0×10^{-4}	1.8×10^{-10}	2.8×10^{-10}	5.0×10^{-4}	2.8×10^{-10}
Tm-167	9.24 d	M	5.0×10^{-4}	1.1×10^{-9}	1.0×10^{-9}	5.0×10^{-4}	5.6×10^{-10}
Tm-170	129 d	M	5.0×10^{-4}	6.6×10^{-9}	5.2×10^{-9}	5.0×10^{-4}	1.3×10^{-9}
Tm-171	1.92 a	M	5.0×10^{-4}	1.3×10^{-9}	9.1×10^{-10}	5.0×10^{-4}	1.1×10^{-10}
Tm-172	2.65 d	M	5.0×10^{-4}	1.1×10^{-9}	1.4×10^{-9}	5.0×10^{-4}	1.7×10^{-9}
Tm-173	8.24 h	M	5.0×10^{-4}	1.8×10^{-10}	2.6×10^{-10}	5.0×10^{-4}	3.1×10^{-10}
Tm-175	0.253 h	M	5.0×10^{-4}	1.9×10^{-11}	3.1×10^{-11}	5.0×10^{-4}	2.7×10^{-11}
Ytterbium							
Yb-162	0.315 h	M	5.0×10^{-4}	1.4×10^{-11}	2.2×10^{-11}	5.0×10^{-4}	2.3×10^{-11}
		S	5.0×10^{-4}	1.4×10^{-11}	2.3×10^{-11}		
Yb-166	2.36 d	M	5.0×10^{-4}	7.2×10^{-10}	9.1×10^{-10}	5.0×10^{-4}	9.5×10^{-10}
		S	5.0×10^{-4}	7.6×10^{-10}	9.5×10^{-10}		
Yb-167	0.292 h	M	5.0×10^{-4}	6.5×10^{-12}	9.0×10^{-12}	5.0×10^{-4}	6.7×10^{-12}
		S	5.0×10^{-4}	6.9×10^{-12}	9.5×10^{-12}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Yb-169	32.0 d	M	5.0×10^{-4}	2.4×10^{-9}	2.1×10^{-9}	5.0×10^{-4}	7.1×10^{-10}
		S	5.0×10^{-4}	2.8×10^{-9}	2.4×10^{-9}		
Yb-175	4.19 d	M	5.0×10^{-4}	6.3×10^{-10}	6.4×10^{-10}	5.0×10^{-4}	4.4×10^{-10}
		S	5.0×10^{-4}	7.0×10^{-10}	7.0×10^{-10}		
Yb-177	1.90 h	M	5.0×10^{-4}	6.4×10^{-11}	8.8×10^{-11}	5.0×10^{-4}	9.7×10^{-11}
		S	5.0×10^{-4}	6.9×10^{-11}	9.4×10^{-11}		
Yb-178	1.23 h	M	5.0×10^{-4}	7.1×10^{-11}	1.0×10^{-10}	5.0×10^{-4}	1.2×10^{-10}
		S	5.0×10^{-4}	7.6×10^{-11}	1.1×10^{-10}		
Lutetium							
Lu-169	1.42 d	M	5.0×10^{-4}	3.5×10^{-10}	4.7×10^{-10}	5.0×10^{-4}	4.6×10^{-10}
		S	5.0×10^{-4}	3.8×10^{-10}	4.9×10^{-10}		
Lu-170	2.00 d	M	5.0×10^{-4}	6.4×10^{-10}	9.3×10^{-10}	5.0×10^{-4}	9.9×10^{-10}
		S	5.0×10^{-4}	6.7×10^{-10}	9.5×10^{-10}		
Lu-171	8.22 d	M	5.0×10^{-4}	7.6×10^{-10}	8.8×10^{-10}	5.0×10^{-4}	6.7×10^{-10}
		S	5.0×10^{-4}	8.3×10^{-10}	9.3×10^{-10}		
Lu-172	6.70 d	M	5.0×10^{-4}	1.4×10^{-9}	1.7×10^{-9}	5.0×10^{-4}	1.3×10^{-9}
		S	5.0×10^{-4}	1.5×10^{-9}	1.8×10^{-9}		
Lu-173	1.37 a	M	5.0×10^{-4}	2.0×10^{-9}	1.5×10^{-9}	5.0×10^{-4}	2.6×10^{-10}
		S	5.0×10^{-4}	2.3×10^{-9}	1.4×10^{-9}		
Lu-174	3.31 a	M	5.0×10^{-4}	4.0×10^{-9}	2.9×10^{-9}	5.0×10^{-4}	2.7×10^{-10}
		S	5.0×10^{-4}	3.9×10^{-9}	2.5×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Lu-174m	142 d	M	5.0×10^{-4}	3.4×10^{-9}	2.4×10^{-9}	5.0×10^{-4}	5.3×10^{-10}
Lu-176	3.60×10^{10} a	S	5.0×10^{-4}	3.8×10^{-9}	2.6×10^{-9}		
		M	5.0×10^{-4}	6.6×10^{-8}	4.6×10^{-8}	5.0×10^{-4}	1.8×10^{-9}
Lu-176m	3.68 h	S	5.0×10^{-4}	5.2×10^{-8}	3.0×10^{-8}		
		M	5.0×10^{-4}	1.1×10^{-10}	1.5×10^{-10}	5.0×10^{-4}	1.7×10^{-10}
Lu-177	6.71 d	S	5.0×10^{-4}	1.2×10^{-10}	1.6×10^{-10}		
		M	5.0×10^{-4}	1.0×10^{-9}	1.0×10^{-9}	5.0×10^{-4}	5.3×10^{-10}
Lu-177m	161 d	S	5.0×10^{-4}	1.1×10^{-9}	1.1×10^{-9}		
		M	5.0×10^{-4}	1.2×10^{-8}	1.0×10^{-8}	5.0×10^{-4}	1.7×10^{-9}
Lu-178	0.473 h	S	5.0×10^{-4}	1.5×10^{-8}	1.2×10^{-8}		
		M	5.0×10^{-4}	2.5×10^{-11}	3.9×10^{-11}	5.0×10^{-4}	4.7×10^{-11}
Lu-178m	0.378 h	S	5.0×10^{-4}	2.6×10^{-11}	4.1×10^{-11}		
		M	5.0×10^{-4}	3.3×10^{-11}	5.4×10^{-11}	5.0×10^{-4}	3.8×10^{-11}
Lu-179	4.59 h	S	5.0×10^{-4}	3.5×10^{-11}	5.6×10^{-11}		
		M	5.0×10^{-4}	1.1×10^{-10}	1.6×10^{-10}	5.0×10^{-4}	2.1×10^{-10}
Hafnium	16.0 h	S	5.0×10^{-4}	1.2×10^{-10}	1.6×10^{-10}		
		F	0.002	1.7×10^{-10}	2.9×10^{-10}	0.002	4.8×10^{-10}
	1.87 a	M	0.002	3.2×10^{-10}	4.3×10^{-10}		
		F	0.002	3.2×10^{-8}	3.7×10^{-8}	0.002	1.0×10^{-9}
		M	0.002	1.9×10^{-8}	1.3×10^{-8}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Hf-173	24.0 h	F	0.002	7.9×10^{-11}	1.3×10^{-10}	0.002	2.3×10^{-10}
		M	0.002	1.6×10^{-10}	2.2×10^{-10}		
Hf-175	70.0 d	F	0.002	7.2×10^{-10}	8.7×10^{-10}	0.002	4.1×10^{-10}
		M	0.002	1.1×10^{-9}	8.8×10^{-10}		
Hf-177m	0.856 h	F	0.002	4.7×10^{-11}	8.4×10^{-11}	0.002	8.1×10^{-11}
		M	0.002	9.2×10^{-11}	1.5×10^{-10}		
Hf-178m	31.0 a	F	0.002	2.6×10^{-7}	3.1×10^{-7}	0.002	4.7×10^{-9}
		M	0.002	1.1×10^{-7}	7.8×10^{-8}		
Hf-179m	25.1 d	F	0.002	1.1×10^{-9}	1.4×10^{-9}	0.002	1.2×10^{-9}
		M	0.002	3.6×10^{-9}	3.2×10^{-9}		
Hf-180m	5.50 h	F	0.002	6.4×10^{-11}	1.2×10^{-10}	0.002	1.7×10^{-10}
		M	0.002	1.4×10^{-10}	2.0×10^{-10}		
Hf-181	42.4 d	F	0.002	1.4×10^{-9}	1.8×10^{-9}	0.002	1.1×10^{-9}
		M	0.002	4.7×10^{-9}	4.1×10^{-9}		
Hf-182	9.00×10^6 a	F	0.002	3.0×10^{-7}	3.6×10^{-7}	0.002	3.0×10^{-9}
		M	0.002	1.2×10^{-7}	8.3×10^{-8}		
Hf-182m	1.02 h	F	0.002	2.3×10^{-11}	4.0×10^{-11}	0.002	4.2×10^{-11}
		M	0.002	4.7×10^{-11}	7.1×10^{-11}		
Hf-183	1.07 h	F	0.002	2.6×10^{-11}	4.4×10^{-11}	0.002	7.3×10^{-11}
		M	0.002	5.8×10^{-11}	8.3×10^{-11}		
Hf-184	4.12 h	F	0.002	1.3×10^{-10}	2.3×10^{-10}	0.002	5.2×10^{-10}
		M	0.002	3.3×10^{-10}	4.5×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\ \mu m}$	$e(g)_{5\ \mu m}$	f_i	$e(g)$
Tantalum							
Ta-172	0.613 h	M	0.001	3.4×10^{-11}	5.5×10^{-11}	0.001	5.3×10^{-11}
		S	0.001	3.6×10^{-11}	5.7×10^{-11}		
Ta-173	3.65 h	M	0.001	1.1×10^{-10}	1.6×10^{-10}	0.001	1.9×10^{-10}
		S	0.001	1.2×10^{-10}	1.6×10^{-10}		
Ta-174	1.20 h	M	0.001	4.2×10^{-11}	6.3×10^{-11}	0.001	5.7×10^{-11}
		S	0.001	4.4×10^{-11}	6.6×10^{-11}		
Ta-175	10.5 h	M	0.001	1.3×10^{-10}	2.0×10^{-10}	0.001	2.1×10^{-10}
		S	0.001	1.4×10^{-10}	2.0×10^{-10}		
Ta-176	8.08 h	M	0.001	2.0×10^{-10}	3.2×10^{-10}	0.001	3.1×10^{-10}
		S	0.001	2.1×10^{-10}	3.3×10^{-10}		
Ta-177	2.36 d	M	0.001	9.3×10^{-11}	1.2×10^{-10}	0.001	1.1×10^{-10}
		S	0.001	1.0×10^{-10}	1.3×10^{-10}		
Ta-178	2.20 h	M	0.001	6.6×10^{-11}	1.0×10^{-10}	0.001	7.8×10^{-11}
		S	0.001	6.9×10^{-11}	1.1×10^{-10}		
Ta-179	1.82 a	M	0.001	2.0×10^{-10}	1.3×10^{-10}	0.001	6.5×10^{-11}
		S	0.001	5.2×10^{-10}	2.9×10^{-10}		
Ta-180	1.00×10^{13} a	M	0.001	6.0×10^{-9}	4.6×10^{-9}	0.001	8.4×10^{-10}
		S	0.001	2.4×10^{-8}	1.4×10^{-8}		
Ta-180m	8.10 h	M	0.001	4.4×10^{-11}	5.8×10^{-11}	0.001	5.4×10^{-11}
		S	0.001	4.7×10^{-11}	6.2×10^{-11}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Ta-182	115 d	M	0.001	7.2×10^{-9}	5.8×10^{-9}	0.001	1.5×10^{-9}
		S	0.001	9.7×10^{-9}	7.4×10^{-9}		
Ta-182m	0.264 h	M	0.001	2.1×10^{-11}	3.4×10^{-11}	0.001	1.2×10^{-11}
		S	0.001	2.2×10^{-11}	3.6×10^{-11}		
Ta-183	5.10 d	M	0.001	1.8×10^{-9}	1.8×10^{-9}	0.001	1.3×10^{-9}
		S	0.001	2.0×10^{-9}	2.0×10^{-9}		
Ta-184	8.70 h	M	0.001	4.1×10^{-10}	6.0×10^{-10}	0.001	6.8×10^{-10}
		S	0.001	4.4×10^{-10}	6.3×10^{-10}		
Ta-185	0.816 h	M	0.001	4.6×10^{-11}	6.8×10^{-11}	0.001	6.8×10^{-11}
		S	0.001	4.9×10^{-11}	7.2×10^{-11}		
Ta-186	0.175 h	M	0.001	1.8×10^{-11}	3.0×10^{-11}	0.001	3.3×10^{-11}
		S	0.001	1.9×10^{-11}	3.1×10^{-11}		
Tungsten							
W-176	2.30 h	F	0.300	4.4×10^{-11}	7.6×10^{-11}	0.300	1.0×10^{-10}
						0.010	1.1×10^{-10}
W-177	2.25 h	F	0.300	2.6×10^{-11}	4.6×10^{-11}	0.300	5.8×10^{-11}
						0.010	6.1×10^{-11}
W-178	21.7 d	F	0.300	7.6×10^{-11}	1.2×10^{-10}	0.300	2.2×10^{-10}
						0.010	2.5×10^{-10}
W-179	0.625 h	F	0.300	9.9×10^{-13}	1.8×10^{-12}	0.300	3.3×10^{-12}
						0.010	3.3×10^{-12}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
W-181	121 d	F	0.300	2.8×10^{-11}	4.3×10^{-11}	0.300	7.6×10^{-11}
						0.010	8.2×10^{-11}
W-185	75.1 d	F	0.300	1.4×10^{-10}	2.2×10^{-10}	0.300	4.4×10^{-10}
						0.010	5.0×10^{-10}
W-187	23.9 h	F	0.300	2.0×10^{-10}	3.3×10^{-10}	0.300	6.3×10^{-10}
						0.010	7.1×10^{-10}
W-188	69.4 d	F	0.300	5.9×10^{-10}	8.4×10^{-10}	0.300	2.1×10^{-9}
						0.010	2.3×10^{-9}
Rhenium							
Re-177	0.233 h	F	0.800	1.0×10^{-11}	1.7×10^{-11}	0.800	2.2×10^{-11}
		M	0.800	1.4×10^{-11}	2.2×10^{-11}		
Re-178	0.220 h	F	0.800	1.1×10^{-11}	1.8×10^{-11}	0.800	2.5×10^{-11}
		M	0.800	1.5×10^{-11}	2.4×10^{-11}		
Re-181	20.0 h	F	0.800	1.9×10^{-10}	3.0×10^{-10}	0.800	4.2×10^{-10}
		M	0.800	2.5×10^{-10}	3.7×10^{-10}		
Re-182	2.67 d	F	0.800	6.8×10^{-10}	1.1×10^{-9}	0.800	1.4×10^{-9}
		M	0.800	1.3×10^{-9}	1.7×10^{-9}		
Re-182m	12.7 h	F	0.800	1.5×10^{-10}	2.4×10^{-10}	0.800	2.7×10^{-10}
		M	0.800	2.0×10^{-10}	3.0×10^{-10}		
Re-184	38.0 d	F	0.800	4.6×10^{-10}	7.0×10^{-10}	0.800	1.0×10^{-9}
		M	0.800	1.8×10^{-9}	1.8×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Re-184m	165 d	F	0.800	6.1×10^{-10}	8.8×10^{-10}	0.800	1.5×10^{-9}
		M	0.800	6.1×10^{-9}	4.8×10^{-9}		
Re-186	3.78 d	F	0.800	5.3×10^{-10}	7.3×10^{-10}	0.800	1.5×10^{-9}
		M	0.800	1.1×10^{-9}	1.2×10^{-9}		
Re-186m	2.00×10^5 a	F	0.800	8.5×10^{-10}	1.2×10^{-9}	0.800	2.2×10^{-9}
		M	0.800	1.1×10^{-8}	7.9×10^{-9}		
Re-187	5.00×10^{10} a	F	0.800	1.9×10^{-12}	2.6×10^{-12}	0.800	5.1×10^{-12}
		M	0.800	6.0×10^{-12}	4.6×10^{-12}		
Re-188	17.0 h	F	0.800	4.7×10^{-10}	6.6×10^{-10}	0.800	1.4×10^{-9}
		M	0.800	5.5×10^{-10}	7.4×10^{-10}		
Re-188m	0.3 h	F	0.800	1.0×10^{-11}	1.6×10^{-11}	0.800	3.0×10^{-11}
		M	0.800	1.4×10^{-11}	2.0×10^{-11}		
Re-189	1.01 d	F	0.800	2.7×10^{-10}	4.3×10^{-10}	0.800	7.8×10^{-10}
		M	0.800	4.3×10^{-10}	6.0×10^{-10}		
Osmium							
Os-180	0.366 h	F	0.010	8.8×10^{-12}	1.6×10^{-11}	0.010	1.7×10^{-11}
		M	0.010	1.4×10^{-11}	2.4×10^{-11}		
		S	0.010	1.5×10^{-11}	2.5×10^{-11}		
Os-181	1.75 h	F	0.010	3.6×10^{-11}	6.4×10^{-11}	0.010	8.9×10^{-11}
		M	0.010	6.3×10^{-11}	9.6×10^{-11}		
		S	0.010	6.6×10^{-11}	1.0×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Os-182	22.0 h	F	0.010	1.9×10^{-10}	3.2×10^{-10}	0.010	5.6×10^{-10}
		M	0.010	3.7×10^{-10}	5.0×10^{-10}		
		S	0.010	3.9×10^{-10}	5.2×10^{-10}		
Os-185	94.0 d	F	0.010	1.1×10^{-9}	1.4×10^{-9}	0.010	5.1×10^{-10}
		M	0.010	1.2×10^{-9}	1.0×10^{-9}		
		S	0.010	1.5×10^{-9}	1.1×10^{-9}		
Os-189m	6.00 h	F	0.010	2.7×10^{-12}	5.2×10^{-12}	0.010	1.8×10^{-11}
		M	0.010	5.1×10^{-12}	7.6×10^{-12}		
		S	0.010	5.4×10^{-12}	7.9×10^{-12}		
Os-191	15.4 d	F	0.010	2.5×10^{-10}	3.5×10^{-10}	0.010	5.7×10^{-10}
		M	0.010	1.5×10^{-9}	1.3×10^{-9}		
		S	0.010	1.8×10^{-9}	1.5×10^{-9}		
Os-191m	13.0 h	F	0.010	2.6×10^{-11}	4.1×10^{-11}	0.010	9.6×10^{-11}
		M	0.010	1.3×10^{-10}	1.3×10^{-10}		
		S	0.010	1.5×10^{-10}	1.4×10^{-10}		
Os-193	1.25 d	F	0.010	1.7×10^{-10}	2.8×10^{-10}	0.010	8.1×10^{-10}
		M	0.010	4.7×10^{-10}	6.4×10^{-10}		
		S	0.010	5.1×10^{-10}	6.8×10^{-10}		
Os-194	6.00 a	F	0.010	1.1×10^{-8}	1.3×10^{-8}	0.010	2.4×10^{-9}
		M	0.010	2.0×10^{-8}	1.3×10^{-8}		
		S	0.010	7.9×10^{-8}	4.2×10^{-8}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{l\ \mu m}$	$e(g)_{s\ \mu m}$	f_i	$e(g)$
Iridium							
Ir-182	0.250 h	F	0.010	1.5×10^{-11}	2.6×10^{-11}	0.010	4.8×10^{-11}
		M	0.010	2.4×10^{-11}	3.9×10^{-11}		
		S	0.010	2.5×10^{-11}	4.0×10^{-11}		
Ir-184	3.02 h	F	0.010	6.7×10^{-11}	1.2×10^{-10}	0.010	1.7×10^{-10}
		M	0.010	1.1×10^{-10}	1.8×10^{-10}		
		S	0.010	1.2×10^{-10}	1.9×10^{-10}		
Ir-185	14.0 h	F	0.010	8.8×10^{-11}	1.5×10^{-10}	0.010	2.6×10^{-10}
		M	0.010	1.8×10^{-10}	2.5×10^{-10}		
		S	0.010	1.9×10^{-10}	2.6×10^{-10}		
Ir-186	15.8 h	F	0.010	1.8×10^{-10}	3.3×10^{-10}	0.010	4.9×10^{-10}
		M	0.010	3.2×10^{-10}	4.8×10^{-10}		
		S	0.010	3.3×10^{-10}	5.0×10^{-10}		
Ir-186m	1.75 h	F	0.010	2.5×10^{-11}	4.5×10^{-11}	0.010	6.1×10^{-11}
		M	0.010	4.3×10^{-11}	6.9×10^{-11}		
		S	0.010	4.5×10^{-11}	7.1×10^{-11}		
Ir-187	10.5 h	F	0.010	4.0×10^{-11}	7.2×10^{-11}	0.010	1.2×10^{-10}
		M	0.010	7.5×10^{-11}	1.1×10^{-10}		
		S	0.010	7.9×10^{-11}	1.2×10^{-10}		
Ir-188	1.73 d	F	0.010	2.6×10^{-10}	4.4×10^{-10}	0.010	6.3×10^{-10}
		M	0.010	4.1×10^{-10}	6.0×10^{-10}		
		S	0.010	4.3×10^{-10}	6.2×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Ir-189	13.3 d	F	0.010	1.1×10^{-10}	1.7×10^{-10}	0.010	2.4×10^{-10}
		M	0.010	4.8×10^{-10}	4.1×10^{-10}		
		S	0.010	5.5×10^{-10}	4.6×10^{-10}		
Ir-190	12.1 d	F	0.010	7.9×10^{-10}	1.2×10^{-9}	0.010	1.2×10^{-9}
		M	0.010	2.0×10^{-9}	2.3×10^{-9}		
		S	0.010	2.3×10^{-9}	2.5×10^{-9}		
Ir-190m	3.10 h	F	0.010	5.3×10^{-11}	9.7×10^{-11}	0.010	1.2×10^{-10}
		M	0.010	8.3×10^{-11}	1.4×10^{-10}		
		S	0.010	8.6×10^{-11}	1.4×10^{-10}		
Ir-190m'	1.20 h	F	0.010	3.7×10^{-12}	5.6×10^{-12}	0.010	8.0×10^{-12}
		M	0.010	9.0×10^{-12}	1.0×10^{-11}		
		S	0.010	1.0×10^{-11}	1.1×10^{-11}		
Ir-192	74.0 d	F	0.010	1.8×10^{-9}	2.2×10^{-9}	0.010	1.4×10^{-9}
		M	0.010	4.9×10^{-9}	4.1×10^{-9}		
		S	0.010	6.2×10^{-9}	4.9×10^{-9}		
Ir-192m	2.41×10^2 a	F	0.010	4.8×10^{-9}	5.6×10^{-9}	0.010	3.1×10^{-10}
		M	0.010	5.4×10^{-9}	3.4×10^{-9}		
		S	0.010	3.6×10^{-8}	1.9×10^{-8}		
Ir-193m	11.9 d	F	0.010	1.0×10^{-10}	1.6×10^{-10}	0.010	2.7×10^{-10}
		M	0.010	1.0×10^{-9}	9.1×10^{-10}		
		S	0.010	1.2×10^{-9}	1.0×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_1	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_1	$e(g)$
Ir-194	19.1 h	F	0.010	2.2×10^{-10}	3.6×10^{-10}	0.010	1.3×10^{-9}
		M	0.010	5.3×10^{-10}	7.1×10^{-10}		
		S	0.010	5.6×10^{-10}	7.5×10^{-10}		
Ir-194m	171 d	F	0.010	5.4×10^{-9}	6.5×10^{-9}	0.010	2.1×10^{-9}
		M	0.010	8.5×10^{-9}	6.5×10^{-9}		
		S	0.010	1.2×10^{-8}	8.2×10^{-9}		
Ir-195	2.50 h	F	0.010	2.6×10^{-11}	4.5×10^{-11}	0.010	1.0×10^{-10}
		M	0.010	6.7×10^{-11}	9.6×10^{-11}		
		S	0.010	7.2×10^{-11}	1.0×10^{-10}		
Ir-195m	3.80 h	F	0.010	6.5×10^{-11}	1.1×10^{-10}	0.010	2.1×10^{-10}
		M	0.010	1.6×10^{-10}	2.3×10^{-10}		
		S	0.010	1.7×10^{-10}	2.4×10^{-10}		
Platinum							
Pt-186	2.00 h	F	0.010	3.6×10^{-11}	6.6×10^{-11}	0.010	9.3×10^{-11}
Pt-188	10.2 d	F	0.010	4.3×10^{-10}	6.3×10^{-10}	0.010	7.6×10^{-10}
Pt-189	10.9 h	F	0.010	4.1×10^{-11}	7.3×10^{-11}	0.010	1.2×10^{-10}
Pt-191	2.80 d	F	0.010	1.1×10^{-10}	1.9×10^{-10}	0.010	3.4×10^{-10}
Pt-193	50.0 a	F	0.010	2.1×10^{-11}	2.7×10^{-11}	0.010	3.1×10^{-11}
Pt-193m	4.33 d	F	0.010	1.3×10^{-10}	2.1×10^{-10}	0.010	4.5×10^{-10}
Pt-195m	4.02 d	F	0.010	1.9×10^{-10}	3.1×10^{-10}	0.010	6.3×10^{-10}
Pt-197	18.3 h	F	0.010	9.1×10^{-11}	1.6×10^{-10}	0.010	4.0×10^{-10}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Pt-197m	1.57 h	F	0.010	2.5×10^{-11}	4.3×10^{-11}	0.010	8.4×10^{-11}
Pt-199	0.513 h	F	0.010	1.3×10^{-11}	2.2×10^{-11}	0.010	3.9×10^{-11}
Pt-200	12.5 h	F	0.010	2.4×10^{-10}	4.0×10^{-10}	0.010	1.2×10^{-9}
Gold							
Au-193	17.6 h	F	0.100	3.9×10^{-11}	7.1×10^{-11}	0.100	1.3×10^{-10}
		M	0.100	1.1×10^{-10}	1.5×10^{-10}		
		S	0.100	1.2×10^{-10}	1.6×10^{-10}		
Au-194	1.64 d	F	0.100	1.5×10^{-10}	2.8×10^{-10}	0.100	4.2×10^{-10}
		M	0.100	2.4×10^{-10}	3.7×10^{-10}		
		S	0.100	2.5×10^{-10}	3.8×10^{-10}		
Au-195	183 d	F	0.100	7.1×10^{-11}	1.2×10^{-10}	0.100	2.5×10^{-10}
		M	0.100	1.0×10^{-9}	8.0×10^{-10}		
		S	0.100	1.6×10^{-9}	1.2×10^{-9}		
Au-198	2.69 d	F	0.100	2.3×10^{-10}	3.9×10^{-10}	0.100	1.0×10^{-9}
		M	0.100	7.6×10^{-10}	9.8×10^{-10}		
		S	0.100	8.4×10^{-10}	1.1×10^{-9}		
Au-198m	2.30 d	F	0.100	3.4×10^{-10}	5.9×10^{-10}	0.100	1.3×10^{-9}
		M	0.100	1.7×10^{-9}	2.0×10^{-9}		
		S	0.100	1.9×10^{-9}	1.9×10^{-9}		
Au-199	3.14 d	F	0.100	1.1×10^{-10}	1.9×10^{-10}	0.100	4.4×10^{-10}
		M	0.100	6.8×10^{-10}	6.8×10^{-10}		
		S	0.100	7.5×10^{-10}	7.6×10^{-10}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Au-200	0.807 h	F	0.100	1.7×10^{-11}	3.0×10^{-11}	0.100	6.8×10^{-11}
		M	0.100	3.5×10^{-11}	5.3×10^{-11}		
		S	0.100	3.6×10^{-11}	5.6×10^{-11}		
Au-200m	18.7 h	F	0.100	3.2×10^{-10}	5.7×10^{-10}	0.100	1.1×10^{-9}
		M	0.100	6.9×10^{-10}	9.8×10^{-10}		
		S	0.100	7.3×10^{-10}	1.0×10^{-9}		
Au-201	0.440 h	F	0.100	9.2×10^{-12}	1.6×10^{-11}	0.100	2.4×10^{-11}
		M	0.100	1.7×10^{-11}	2.8×10^{-11}		
		S	0.100	1.8×10^{-11}	2.9×10^{-11}		
Mercury							
Hg-193 (organic)	3.50 h	F	0.400	2.6×10^{-11}	4.7×10^{-11}	1.000	3.1×10^{-11}
Hg-193 (inorganic)	3.50 h	F	0.020	2.8×10^{-11}	5.0×10^{-11}	0.400	6.6×10^{-11}
		M	0.020	7.5×10^{-11}	1.0×10^{-10}	0.020	8.2×10^{-11}
Hg-193m (organic)	11.1 h	F	0.400	1.1×10^{-10}	2.0×10^{-10}	1.000	1.3×10^{-10}
						0.400	3.0×10^{-10}
Hg-193m (inorganic)	11.1 h	F	0.020	1.2×10^{-10}	2.3×10^{-10}	0.020	4.0×10^{-10}
		M	0.020	2.6×10^{-10}	3.8×10^{-10}		
Hg-194 (organic)	2.60×10^2 a	F	0.400	1.5×10^{-8}	1.9×10^{-8}	1.000	5.1×10^{-8}
						0.400	2.1×10^{-8}
Hg-194 (inorganic)	2.60×10^2 a	F	0.020	1.3×10^{-8}	1.5×10^{-8}	0.020	1.4×10^{-9}
		M	0.020	7.8×10^{-9}	5.3×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Hg-195 (organic)	9.90 h	F	0.400	2.4×10^{-11}	4.4×10^{-11}	1.000	3.4×10^{-11}
Hg-195 (inorganic)	9.90 h	F	0.020	2.7×10^{-11}	4.8×10^{-11}	0.400	7.5×10^{-11}
Hg-195m	1.73 d	M	0.020	7.2×10^{-11}	9.2×10^{-11}	0.020	9.7×10^{-11}
Hg-195m (organic)	1.73 d	F	0.400	1.3×10^{-10}	2.2×10^{-10}	1.000	2.2×10^{-10}
Hg-195m (inorganic)	1.73 d	F	0.020	1.5×10^{-10}	2.6×10^{-10}	0.400	4.1×10^{-10}
Hg-197 (inorganic)	2.67 d	M	0.020	5.1×10^{-10}	6.5×10^{-10}	0.020	5.6×10^{-10}
Hg-197 (organic)	2.67 d	F	0.400	5.0×10^{-11}	8.5×10^{-11}	1.000	9.9×10^{-11}
Hg-197 (inorganic)	2.67 d	F	0.020	6.0×10^{-11}	1.0×10^{-10}	0.400	1.7×10^{-10}
Hg-197m	23.8 h	M	0.020	2.9×10^{-10}	2.8×10^{-10}	0.020	2.3×10^{-10}
Hg-197m (organic)	23.8 h	F	0.400	1.0×10^{-10}	1.8×10^{-10}	1.000	1.5×10^{-10}
Hg-197m (inorganic)	23.8 h	F	0.020	1.2×10^{-10}	2.1×10^{-10}	0.400	3.4×10^{-10}
Hg-199m	0.710 h	M	0.020	5.1×10^{-10}	6.6×10^{-10}	0.020	4.7×10^{-10}
Hg-199m (organic)	0.710 h	F	0.400	1.6×10^{-11}	2.7×10^{-11}	1.000	2.8×10^{-11}
Hg-199m (inorganic)	0.710 h	F	0.020	1.6×10^{-11}	2.7×10^{-11}	0.400	3.1×10^{-11}
Hg-203 (organic)	46.6 d	M	0.020	3.3×10^{-11}	5.2×10^{-11}	0.020	3.1×10^{-11}
		F	0.400	5.7×10^{-10}	7.5×10^{-10}	1.000	1.9×10^{-9}
						0.400	1.1×10^{-9}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Hg-203 (inorganic)	46.6 d	F	0.020	4.7×10^{-10}	5.9×10^{-10}	0.020	5.4×10^{-10}
		M	0.020	2.3×10^{-9}	1.9×10^{-9}		
Thallium							
Tl-194	0.550 h	F	1.000	4.8×10^{-12}	8.9×10^{-12}	1.000	8.1×10^{-12}
Tl-194m	0.546 h	F	1.000	2.0×10^{-11}	3.6×10^{-11}	1.000	4.0×10^{-11}
Tl-195	1.16 h	F	1.000	1.6×10^{-11}	3.0×10^{-11}	1.000	2.7×10^{-11}
Tl-197	2.84 h	F	1.000	1.5×10^{-11}	2.7×10^{-11}	1.000	2.3×10^{-11}
Tl-198	5.30 h	F	1.000	6.6×10^{-11}	1.2×10^{-10}	1.000	7.3×10^{-11}
Tl-198m	1.87 h	F	1.000	4.0×10^{-11}	7.3×10^{-11}	1.000	5.4×10^{-11}
Tl-199	7.42 h	F	1.000	2.0×10^{-11}	3.7×10^{-11}	1.000	2.6×10^{-11}
Tl-200	1.09 d	F	1.000	1.4×10^{-10}	2.5×10^{-10}	1.000	2.0×10^{-10}
Tl-201	3.04 d	F	1.000	4.7×10^{-11}	7.6×10^{-11}	1.000	9.5×10^{-11}
Tl-202	12.2 d	F	1.000	2.0×10^{-10}	3.1×10^{-10}	1.000	4.5×10^{-10}
Tl-204	3.78 a	F	1.000	4.4×10^{-10}	6.2×10^{-10}	1.000	1.3×10^{-9}
Lead							
Pb-195m	0.263 h	F	0.200	1.7×10^{-11}	3.0×10^{-11}	0.200	2.9×10^{-11}
Pb-198	2.40 h	F	0.200	4.7×10^{-11}	8.7×10^{-11}	0.200	1.0×10^{-10}
Pb-199	1.50 h	F	0.200	2.6×10^{-11}	4.8×10^{-11}	0.200	5.4×10^{-11}
Pb-200	21.5 h	F	0.200	1.5×10^{-10}	2.6×10^{-10}	0.200	4.0×10^{-10}
Pb-201	9.40 h	F	0.200	6.5×10^{-11}	1.2×10^{-10}	0.200	1.6×10^{-10}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\ \mu m}$	$e(g)_{5\ \mu m}$	f_i	$e(g)$
Pb-202	3.00×10^5 a	F	0.200	1.1×10^{-8}	1.4×10^{-8}	0.200	8.7×10^{-9}
Pb-202m	3.62 h	F	0.200	6.7×10^{-11}	1.2×10^{-10}	0.200	1.3×10^{-10}
Pb-203	2.17 d	F	0.200	9.1×10^{-11}	1.6×10^{-10}	0.200	2.4×10^{-10}
Pb-205	1.43×10^7 a	F	0.200	3.4×10^{-10}	4.1×10^{-10}	0.200	2.8×10^{-10}
Pb-209	3.25 h	F	0.200	1.8×10^{-11}	3.2×10^{-11}	0.200	5.7×10^{-11}
Pb-210	22.3 a	F	0.200	8.9×10^{-7}	1.1×10^{-6}	0.200	6.8×10^{-7}
Pb-211	0.601 h	F	0.200	3.9×10^{-9}	5.6×10^{-9}	0.200	1.8×10^{-10}
Pb-212	10.6 h	F	0.200	1.9×10^{-8}	3.3×10^{-8}	0.200	5.9×10^{-9}
Pb-214	0.447 h	F	0.200	2.9×10^{-9}	4.8×10^{-9}	0.200	1.4×10^{-10}
Bismuth							
Bi-200	0.606 h	F	0.050	2.4×10^{-11}	4.2×10^{-11}	0.050	5.1×10^{-11}
		M	0.050	3.4×10^{-11}	5.6×10^{-11}		
Bi-201	1.80 h	F	0.050	4.7×10^{-11}	8.3×10^{-11}	0.050	1.2×10^{-10}
		M	0.050	7.0×10^{-11}	1.1×10^{-10}		
Bi-202	1.67 h	F	0.050	4.6×10^{-11}	8.4×10^{-11}	0.050	8.9×10^{-11}
		M	0.050	5.8×10^{-11}	1.0×10^{-10}		
Bi-203	11.8 h	F	0.050	2.0×10^{-10}	3.6×10^{-10}	0.050	4.8×10^{-10}
		M	0.050	2.8×10^{-10}	4.5×10^{-10}		
Bi-205	15.3 d	F	0.050	4.0×10^{-10}	6.8×10^{-10}	0.050	9.0×10^{-10}
		M	0.050	9.2×10^{-10}	1.0×10^{-9}		
Bi-206	6.24 d	F	0.050	7.9×10^{-10}	1.3×10^{-9}	0.050	1.9×10^{-9}
		M	0.050	1.7×10^{-9}	2.1×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Bi-207	38.0 a	F	0.050	5.2×10^{-10}	8.4×10^{-10}	0.050	1.3×10^{-9}
		M	0.050	5.2×10^{-9}	3.2×10^{-9}		
Bi-210	5.01 d	F	0.050	1.1×10^{-9}	1.4×10^{-9}	0.050	1.3×10^{-9}
		M	0.050	8.4×10^{-8}	6.0×10^{-8}		
Bi-210m	3.00×10^6 a	F	0.050	4.5×10^{-8}	5.3×10^{-8}	0.050	1.5×10^{-8}
		M	0.050	3.1×10^{-6}	2.1×10^{-6}		
Bi-212	1.01 h	F	0.050	9.3×10^{-9}	1.5×10^{-8}	0.050	2.6×10^{-10}
		M	0.050	3.0×10^{-8}	3.9×10^{-8}		
Bi-213	0.761 h	F	0.050	1.1×10^{-8}	1.8×10^{-8}	0.050	2.0×10^{-10}
		M	0.050	2.9×10^{-8}	4.1×10^{-8}		
Bi-214	0.332 h	F	0.050	7.2×10^{-9}	1.2×10^{-8}	0.050	1.1×10^{-10}
		M	0.050	1.4×10^{-8}	2.1×10^{-8}		
Polonium							
Po-203	0.612 h	F	0.100	2.5×10^{-11}	4.5×10^{-11}	0.100	5.2×10^{-11}
		M	0.100	3.6×10^{-11}	6.1×10^{-11}		
Po-205	1.80 h	F	0.100	3.5×10^{-11}	6.0×10^{-11}	0.100	5.9×10^{-11}
		M	0.100	6.4×10^{-11}	8.9×10^{-11}		
Po-207	5.83 h	F	0.100	6.3×10^{-11}	1.2×10^{-10}	0.100	1.4×10^{-10}
		M	0.100	8.4×10^{-11}	1.5×10^{-10}		
Po-210	138 d	F	0.100	6.0×10^{-7}	7.1×10^{-7}	0.100	2.4×10^{-7}
		M	0.100	3.0×10^{-6}	2.2×10^{-6}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\ \mu m}$	$e(g)_{5\ \mu m}$	f_i	$e(g)$
Astatine							
At-207	1.80 h	F	1.000	3.5×10^{-10}	4.4×10^{-10}	1.000	2.3×10^{-10}
		M	1.000	2.1×10^{-9}	1.9×10^{-9}		
At-211	7.21 h	F	1.000	1.6×10^{-8}	2.7×10^{-8}	1.000	1.1×10^{-8}
		M	1.000	9.8×10^{-8}	1.1×10^{-7}		
Francium							
Fr-222	0.240 h	F	1.000	1.4×10^{-8}	2.1×10^{-8}	1.000	7.1×10^{-10}
Fr-223	0.363 h	F	1.000	9.1×10^{-10}	1.3×10^{-9}	1.000	2.3×10^{-9}
Radium							
Ra-223	11.4 d	M	0.200	6.9×10^{-6}	5.7×10^{-6}	0.200	1.0×10^{-7}
Ra-224	3.66 d	M	0.200	2.9×10^{-6}	2.4×10^{-6}	0.200	6.5×10^{-8}
Ra-225	14.8 d	M	0.200	5.8×10^{-6}	4.8×10^{-6}	0.200	9.5×10^{-8}
Ra-226	1.60×10^3 a	M	0.200	3.2×10^{-6}	2.2×10^{-6}	0.200	2.8×10^{-7}
Ra-227	0.703 h	M	0.200	2.8×10^{-10}	2.1×10^{-10}	0.200	8.4×10^{-11}
Ra-228	5.75 a	M	0.200	2.6×10^{-6}	1.7×10^{-6}	0.200	6.7×10^{-7}
Actinium							
Ac-224	2.90 h	F	5.0×10^{-4}	1.1×10^{-8}	1.3×10^{-8}	5.0×10^{-4}	7.0×10^{-10}
		M	5.0×10^{-4}	1.0×10^{-7}	8.9×10^{-8}		
		S	5.0×10^{-4}	1.2×10^{-7}	9.9×10^{-8}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Ac-225	10.0 d	F	5.0×10^{-4}	8.7×10^{-7}	1.0×10^{-6}	5.0×10^{-4}	2.4×10^{-8}
		M	5.0×10^{-4}	6.9×10^{-6}	5.7×10^{-6}		
		S	5.0×10^{-4}	7.9×10^{-6}	6.5×10^{-6}		
Ac-226	1.21 d	F	5.0×10^{-4}	9.5×10^{-8}	2.2×10^{-7}	5.0×10^{-4}	1.0×10^{-8}
		M	5.0×10^{-4}	1.1×10^{-6}	9.2×10^{-7}		
		S	5.0×10^{-4}	1.2×10^{-6}	1.0×10^{-6}		
Ac-227	21.8 a	F	5.0×10^{-4}	5.4×10^{-4}	6.3×10^{-4}	5.0×10^{-4}	1.1×10^{-6}
		M	5.0×10^{-4}	2.1×10^{-4}	1.5×10^{-4}		
		S	5.0×10^{-4}	6.6×10^{-5}	4.7×10^{-5}		
Ac-228	6.13 h	F	5.0×10^{-4}	2.5×10^{-8}	2.9×10^{-8}	5.0×10^{-4}	4.3×10^{-10}
		M	5.0×10^{-4}	1.6×10^{-8}	1.2×10^{-8}		
		S	5.0×10^{-4}	1.4×10^{-8}	1.2×10^{-8}		
Thorium							
Th-226	0.515 h	M	5.0×10^{-4}	5.5×10^{-8}	7.4×10^{-8}	5.0×10^{-4}	3.5×10^{-10}
		S	2.0×10^{-4}	5.9×10^{-8}	7.8×10^{-8}	2.0×10^{-4}	3.6×10^{-10}
Th-227	18.7 d	M	5.0×10^{-4}	7.8×10^{-6}	6.2×10^{-6}	5.0×10^{-4}	8.9×10^{-9}
		S	2.0×10^{-4}	9.6×10^{-6}	7.6×10^{-6}	2.0×10^{-4}	8.4×10^{-9}
Th-228	1.91 a	M	5.0×10^{-4}	3.1×10^{-5}	2.3×10^{-5}	5.0×10^{-4}	7.0×10^{-8}
		S	2.0×10^{-4}	3.9×10^{-5}	3.2×10^{-5}	2.0×10^{-4}	3.5×10^{-8}
Th-229	7.34×10^3 a	M	5.0×10^{-4}	9.9×10^{-5}	6.9×10^{-5}	5.0×10^{-4}	4.8×10^{-7}
		S	2.0×10^{-4}	6.5×10^{-5}	4.8×10^{-5}	2.0×10^{-4}	2.0×10^{-7}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
Th-230	7.70×10^4 a	M	5.0×10^{-4}	4.0×10^{-5}	2.8×10^{-5}	5.0×10^{-4}	2.1×10^{-7}
		S	2.0×10^{-4}	1.3×10^{-5}	7.2×10^{-6}	2.0×10^{-4}	8.7×10^{-8}
Th-231	1.06 d	M	5.0×10^{-4}	2.9×10^{-10}	3.7×10^{-10}	5.0×10^{-4}	3.4×10^{-10}
		S	2.0×10^{-4}	3.2×10^{-10}	4.0×10^{-10}	2.0×10^{-4}	3.4×10^{-10}
Th-232	1.40×10^{10} a	M	5.0×10^{-4}	4.2×10^{-5}	2.9×10^{-5}	5.0×10^{-4}	2.2×10^{-7}
		S	2.0×10^{-4}	2.3×10^{-5}	1.2×10^{-5}	2.0×10^{-4}	9.2×10^{-8}
Th-234	24.1 d	M	5.0×10^{-4}	6.3×10^{-9}	5.3×10^{-9}	5.0×10^{-4}	3.4×10^{-9}
		S	2.0×10^{-4}	7.3×10^{-9}	5.8×10^{-9}	2.0×10^{-4}	3.4×10^{-9}
Protactinium							
Pa-227	0.638 h	M	5.0×10^{-4}	7.0×10^{-8}	9.0×10^{-8}	5.0×10^{-4}	4.5×10^{-10}
		S	5.0×10^{-4}	7.6×10^{-8}	9.7×10^{-8}		
Pa-228	22.0 h	M	5.0×10^{-4}	5.9×10^{-8}	4.6×10^{-8}	5.0×10^{-4}	7.8×10^{-10}
		S	5.0×10^{-4}	6.9×10^{-8}	5.1×10^{-8}		
Pa-230	17.4 d	M	5.0×10^{-4}	5.6×10^{-7}	4.6×10^{-7}	5.0×10^{-4}	9.2×10^{-10}
		S	5.0×10^{-4}	7.1×10^{-7}	5.7×10^{-7}		
Pa-231	3.27×10^4 a	M	5.0×10^{-4}	1.3×10^{-4}	8.9×10^{-5}	5.0×10^{-4}	7.1×10^{-7}
		S	5.0×10^{-4}	3.2×10^{-5}	1.7×10^{-5}		
Pa-232	1.31 d	M	5.0×10^{-4}	9.5×10^{-9}	6.8×10^{-9}	5.0×10^{-4}	7.2×10^{-10}
		S	5.0×10^{-4}	3.2×10^{-9}	2.0×10^{-9}		
Pa-233	27.0 d	M	5.0×10^{-4}	3.1×10^{-9}	2.8×10^{-9}	5.0×10^{-4}	8.7×10^{-10}
		S	5.0×10^{-4}	3.7×10^{-9}	3.2×10^{-9}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Pa-234	6.70 h	M	5.0×10^{-4}	3.8×10^{-10}	5.5×10^{-10}	5.0×10^{-4}	5.1×10^{-10}
		S	5.0×10^{-4}	4.0×10^{-10}	5.8×10^{-10}		
Uranium	20.8 d	F	0.020	3.6×10^{-7}	4.2×10^{-7}	0.020	5.5×10^{-8}
		M	0.020	1.2×10^{-5}	1.0×10^{-5}	0.002	2.8×10^{-8}
		S	0.002	1.5×10^{-5}	1.2×10^{-5}		
	4.20 d	F	0.020	8.3×10^{-11}	1.4×10^{-10}	0.020	2.8×10^{-10}
		M	0.020	3.4×10^{-10}	3.7×10^{-10}	0.002	2.8×10^{-10}
		S	0.002	3.7×10^{-10}	4.0×10^{-10}		
72.0 a	F	0.020	4.0×10^{-6}	4.7×10^{-6}	0.020	3.3×10^{-7}	
	M	0.020	7.2×10^{-6}	4.8×10^{-6}	0.002	3.7×10^{-8}	
	S	0.002	3.5×10^{-5}	2.6×10^{-5}			
U-233	1.58×10^5 a	F	0.020	5.7×10^{-7}	6.6×10^{-7}	0.020	5.0×10^{-8}
		M	0.020	3.2×10^{-6}	2.2×10^{-6}	0.002	8.5×10^{-9}
		S	0.002	8.7×10^{-6}	6.9×10^{-6}		
U-234	2.44×10^5 a	F	0.020	5.5×10^{-7}	6.4×10^{-7}	0.020	4.9×10^{-8}
		M	0.020	3.1×10^{-6}	2.1×10^{-6}	0.002	8.3×10^{-9}
		S	0.002	8.5×10^{-6}	6.8×10^{-6}		
U-235	7.04×10^8 a	F	0.020	5.1×10^{-7}	6.0×10^{-7}	0.020	4.6×10^{-8}
		M	0.020	2.8×10^{-6}	1.8×10^{-6}	0.002	8.3×10^{-9}
		S	0.002	7.7×10^{-6}	6.1×10^{-6}		

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\text{ }\mu\text{m}}$	$e(g)_{5\text{ }\mu\text{m}}$	f_i	$e(g)$
U-236	2.34×10^7 a	F	0.020	5.2×10^{-7}	6.1×10^{-7}	0.020	4.6×10^{-8}
		M	0.020	2.9×10^{-6}	1.9×10^{-6}	0.002	7.9×10^{-9}
		S	0.002	7.9×10^{-6}	6.3×10^{-6}		
U-237	6.75 d	F	0.020	1.9×10^{-10}	3.3×10^{-10}	0.020	7.6×10^{-10}
		M	0.020	1.6×10^{-9}	1.5×10^{-9}	0.002	7.7×10^{-10}
		S	0.002	1.8×10^{-9}	1.7×10^{-9}		
U-238	4.47×10^9 a	F	0.020	4.9×10^{-7}	5.8×10^{-7}	0.020	4.4×10^{-8}
		M	0.020	2.6×10^{-6}	1.6×10^{-6}	0.002	7.6×10^{-9}
		S	0.002	7.3×10^{-6}	5.7×10^{-6}		
U-239	0.392 h	F	0.020	1.1×10^{-11}	1.8×10^{-11}	0.020	2.7×10^{-11}
		M	0.020	2.3×10^{-11}	3.3×10^{-11}	0.002	2.8×10^{-11}
		S	0.002	2.4×10^{-11}	3.5×10^{-11}		
U-240	14.1 h	F	0.020	2.1×10^{-10}	3.7×10^{-10}	0.020	1.1×10^{-9}
		M	0.020	5.3×10^{-10}	7.9×10^{-10}	0.002	1.1×10^{-9}
		S	0.002	5.7×10^{-10}	8.4×10^{-10}		
Neptunium							
Np-232	0.245 h	M	5.0×10^{-4}	4.7×10^{-11}	3.5×10^{-11}	5.0×10^{-4}	9.7×10^{-12}
Np-233	0.603 h	M	5.0×10^{-4}	1.7×10^{-12}	3.0×10^{-12}	5.0×10^{-4}	2.2×10^{-12}
Np-234	4.40 d	M	5.0×10^{-4}	5.4×10^{-10}	7.3×10^{-10}	5.0×10^{-4}	8.1×10^{-10}
Np-235	1.08 a	M	5.0×10^{-4}	4.0×10^{-10}	2.7×10^{-10}	5.0×10^{-4}	5.3×10^{-11}
Np-236	1.15×10^5 a	M	5.0×10^{-4}	3.0×10^{-6}	2.0×10^{-6}	5.0×10^{-4}	1.7×10^{-8}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Np-236m	22.5 h	M	5.0×10^{-4}	5.0×10^{-9}	3.6×10^{-9}	5.0×10^{-4}	1.9×10^{-10}
Np-237	2.14×10^6 a	M	5.0×10^{-4}	2.1×10^{-5}	1.5×10^{-5}	5.0×10^{-4}	1.1×10^{-7}
Np-238	2.12 d	M	5.0×10^{-4}	2.0×10^{-9}	1.7×10^{-9}	5.0×10^{-4}	9.1×10^{-10}
Np-239	2.36 d	M	5.0×10^{-4}	9.0×10^{-10}	1.1×10^{-9}	5.0×10^{-4}	8.0×10^{-10}
Np-240	1.08 h	M	5.0×10^{-4}	8.7×10^{-11}	1.3×10^{-10}	5.0×10^{-4}	8.2×10^{-11}
Plutonium							
Pu-234	8.80 h	M	5.0×10^{-4}	1.9×10^{-8}	1.6×10^{-8}	5.0×10^{-4}	1.6×10^{-10}
		S	1.0×10^{-5}	2.2×10^{-8}	1.8×10^{-8}	1.0×10^{-5}	1.5×10^{-10}
Pu-235	0.422 h	M	5.0×10^{-4}	1.5×10^{-12}	2.5×10^{-12}	5.0×10^{-4}	2.1×10^{-12}
		S	1.0×10^{-5}	1.6×10^{-12}	2.6×10^{-12}	1.0×10^{-5}	2.1×10^{-12}
Pu-236	2.85 a	M	5.0×10^{-4}	1.8×10^{-5}	1.3×10^{-5}	5.0×10^{-4}	8.6×10^{-8}
		S	1.0×10^{-5}	9.6×10^{-6}	7.4×10^{-6}	1.0×10^{-5}	6.3×10^{-9}
Pu-237	45.3 d	M	5.0×10^{-4}	3.3×10^{-10}	2.9×10^{-10}	5.0×10^{-4}	1.0×10^{-10}
		S	1.0×10^{-5}	3.6×10^{-10}	3.0×10^{-10}	1.0×10^{-5}	1.0×10^{-10}
Pu-238	87.7 a	M	5.0×10^{-4}	4.3×10^{-5}	3.0×10^{-5}	5.0×10^{-4}	2.3×10^{-7}
		S	1.0×10^{-5}	1.5×10^{-5}	1.1×10^{-5}	1.0×10^{-5}	8.8×10^{-9}
						1.0×10^{-4}	4.9×10^{-8}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Pu-239	2.41×10^4 a	M	5.0×10^{-4}	4.7×10^{-5}	3.2×10^{-5}	5.0×10^{-4}	2.5×10^{-7}
		S	1.0×10^{-5}	1.5×10^{-5}	8.3×10^{-6}	1.0×10^{-5}	9.0×10^{-9}
Pu-240	6.54×10^3 a	M	5.0×10^{-4}	4.7×10^{-5}	3.2×10^{-5}	5.0×10^{-4}	5.3×10^{-8}
		S	1.0×10^{-5}	1.5×10^{-5}	8.3×10^{-6}	1.0×10^{-5}	2.5×10^{-7}
Pu-241	14.4 a	M	5.0×10^{-4}	8.5×10^{-7}	5.8×10^{-7}	5.0×10^{-4}	9.0×10^{-9}
		S	1.0×10^{-5}	1.6×10^{-7}	8.4×10^{-8}	1.0×10^{-5}	5.3×10^{-8}
Pu-242	3.76×10^5 a	M	5.0×10^{-4}	4.4×10^{-5}	3.1×10^{-5}	5.0×10^{-4}	4.7×10^{-9}
		S	1.0×10^{-5}	1.4×10^{-5}	7.7×10^{-6}	1.0×10^{-5}	1.1×10^{-10}
Pu-243	4.95 h	M	5.0×10^{-4}	8.2×10^{-11}	1.1×10^{-10}	5.0×10^{-4}	9.6×10^{-10}
		S	1.0×10^{-5}	8.5×10^{-11}	1.1×10^{-10}	5.0×10^{-4}	2.4×10^{-7}
Pu-244	8.26×10^7 a	M	5.0×10^{-4}	4.4×10^{-5}	3.0×10^{-5}	5.0×10^{-4}	8.6×10^{-9}
		S	1.0×10^{-5}	1.3×10^{-5}	7.4×10^{-6}	1.0×10^{-5}	5.0×10^{-8}
Pu-245	10.5 h	M	5.0×10^{-4}	4.5×10^{-10}	6.1×10^{-10}	5.0×10^{-4}	8.5×10^{-11}
		S	1.0×10^{-5}	4.8×10^{-10}	6.5×10^{-10}	5.0×10^{-4}	2.4×10^{-7}
						1.0×10^{-5}	1.1×10^{-8}
						1.0×10^{-4}	5.2×10^{-8}
						5.0×10^{-4}	7.2×10^{-10}
						1.0×10^{-5}	7.2×10^{-10}
						1.0×10^{-4}	7.2×10^{-10}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Pu-246	10.9 d	M	5.0×10^{-4}	7.0×10^{-9}	6.5×10^{-9}	5.0×10^{-4}	3.3×10^{-9}
		S	1.0×10^{-5}	7.6×10^{-9}	7.0×10^{-9}	1.0×10^{-5}	3.3×10^{-9}
						1.0×10^{-4}	3.3×10^{-9}
Americium							
Am-237	1.22 h	M	5.0×10^{-4}	2.5×10^{-11}	3.6×10^{-11}	5.0×10^{-4}	1.8×10^{-11}
Am-238	1.63 h	M	5.0×10^{-4}	8.5×10^{-11}	6.6×10^{-11}	5.0×10^{-4}	3.2×10^{-11}
Am-239	11.9 h	M	5.0×10^{-4}	2.2×10^{-10}	2.9×10^{-10}	5.0×10^{-4}	2.4×10^{-10}
Am-240	2.12 d	M	5.0×10^{-4}	4.4×10^{-10}	5.9×10^{-10}	5.0×10^{-4}	5.8×10^{-10}
Am-241	4.32×10^2 a	M	5.0×10^{-4}	3.9×10^{-5}	2.7×10^{-5}	5.0×10^{-4}	2.0×10^{-7}
Am-242	16.0 h	M	5.0×10^{-4}	1.6×10^{-8}	1.2×10^{-8}	5.0×10^{-4}	3.0×10^{-10}
Am-242m	1.52×10^2 a	M	5.0×10^{-4}	3.5×10^{-5}	2.4×10^{-5}	5.0×10^{-4}	1.9×10^{-7}
Am-243	7.38×10^3 a	M	5.0×10^{-4}	3.9×10^{-5}	2.7×10^{-5}	5.0×10^{-4}	2.0×10^{-7}
Am-244	10.1 h	M	5.0×10^{-4}	1.9×10^{-9}	1.5×10^{-9}	5.0×10^{-4}	4.6×10^{-10}
Am-244m	0.433 h	M	5.0×10^{-4}	7.9×10^{-11}	6.2×10^{-11}	5.0×10^{-4}	2.9×10^{-11}
Am-245	2.05 h	M	5.0×10^{-4}	5.3×10^{-11}	7.6×10^{-11}	5.0×10^{-4}	6.2×10^{-11}
Am-246	0.650 h	M	5.0×10^{-4}	6.8×10^{-11}	1.1×10^{-10}	5.0×10^{-4}	5.8×10^{-11}
Am-246m	0.417 h	M	5.0×10^{-4}	2.3×10^{-11}	3.8×10^{-11}	5.0×10^{-4}	3.4×10^{-11}
Curium							
Cm-238	2.40 h	M	5.0×10^{-4}	4.1×10^{-9}	4.8×10^{-9}	5.0×10^{-4}	8.0×10^{-11}
Cm-240	27.0 d	M	5.0×10^{-4}	2.9×10^{-6}	2.3×10^{-6}	5.0×10^{-4}	7.6×10^{-9}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Cm-241	32.8 d	M	5.0×10^{-4}	3.4×10^{-8}	2.6×10^{-8}	5.0×10^{-4}	9.1×10^{-10}
Cm-242	163 d	M	5.0×10^{-4}	4.8×10^{-6}	3.7×10^{-6}	5.0×10^{-4}	1.2×10^{-8}
Cm-243	28.5 a	M	5.0×10^{-4}	2.9×10^{-5}	2.0×10^{-5}	5.0×10^{-4}	1.5×10^{-7}
Cm-244	18.1 a	M	5.0×10^{-4}	2.5×10^{-5}	1.7×10^{-5}	5.0×10^{-4}	1.2×10^{-7}
Cm-245	8.50×10^3 a	M	5.0×10^{-4}	4.0×10^{-5}	2.7×10^{-5}	5.0×10^{-4}	2.1×10^{-7}
Cm-246	4.73×10^3 a	M	5.0×10^{-4}	4.0×10^{-5}	2.7×10^{-5}	5.0×10^{-4}	2.1×10^{-7}
Cm-247	1.56×10^7 a	M	5.0×10^{-4}	3.6×10^{-5}	2.5×10^{-5}	5.0×10^{-4}	1.9×10^{-7}
Cm-248	3.39×10^5 a	M	5.0×10^{-4}	1.4×10^{-4}	9.5×10^{-5}	5.0×10^{-4}	7.7×10^{-7}
Cm-249	1.07 h	M	5.0×10^{-4}	3.2×10^{-11}	5.1×10^{-11}	5.0×10^{-4}	3.1×10^{-11}
Cm-250	6.90×10^3 a	M	5.0×10^{-4}	7.9×10^{-4}	5.4×10^{-4}	5.0×10^{-4}	4.4×10^{-6}
Berkelium							
Bk-245	4.94 d	M	5.0×10^{-4}	2.0×10^{-9}	1.8×10^{-9}	5.0×10^{-4}	5.7×10^{-10}
Bk-246	1.83 d	M	5.0×10^{-4}	3.4×10^{-10}	4.6×10^{-10}	5.0×10^{-4}	4.8×10^{-10}
Bk-247	1.38×10^3 a	M	5.0×10^{-4}	6.5×10^{-5}	4.5×10^{-5}	5.0×10^{-4}	3.5×10^{-7}
Bk-249	320 d	M	5.0×10^{-4}	1.5×10^{-7}	1.0×10^{-7}	5.0×10^{-4}	9.7×10^{-10}
Bk-250	3.22 h	M	5.0×10^{-4}	9.6×10^{-10}	7.1×10^{-10}	5.0×10^{-4}	1.4×10^{-10}
Californium							
Cf-244	0.323 h	M	5.0×10^{-4}	1.3×10^{-8}	1.8×10^{-8}	5.0×10^{-4}	7.0×10^{-11}
Cf-246	1.49 d	M	5.0×10^{-4}	4.2×10^{-7}	3.5×10^{-7}	5.0×10^{-4}	3.3×10^{-9}
Cf-248	334 d	M	5.0×10^{-4}	8.2×10^{-6}	6.1×10^{-6}	5.0×10^{-4}	2.8×10^{-8}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_i	$e(g)_{1\mu m}$	$e(g)_{5\mu m}$	f_i	$e(g)$
Cf-249	3.50×10^2 a	M	5.0×10^{-4}	6.6×10^{-5}	4.5×10^{-5}	5.0×10^{-4}	3.5×10^{-7}
Cf-250	13.1 a	M	5.0×10^{-4}	3.2×10^{-5}	2.2×10^{-5}	5.0×10^{-4}	1.6×10^{-7}
Cf-251	8.98×10^2 a	M	5.0×10^{-4}	6.7×10^{-5}	4.6×10^{-5}	5.0×10^{-4}	3.6×10^{-7}
Cf-252	2.64 a	M	5.0×10^{-4}	1.8×10^{-5}	1.3×10^{-5}	5.0×10^{-4}	9.0×10^{-8}
Cf-253	17.8 d	M	5.0×10^{-4}	1.2×10^{-6}	1.0×10^{-6}	5.0×10^{-4}	1.4×10^{-9}
Cf-254	60.5 d	M	5.0×10^{-4}	3.7×10^{-5}	2.2×10^{-5}	5.0×10^{-4}	4.0×10^{-7}
Einsteinium							
Es-250	2.10 h	M	5.0×10^{-4}	5.9×10^{-10}	4.2×10^{-10}	5.0×10^{-4}	2.1×10^{-11}
Es-251	1.38 d	M	5.0×10^{-4}	2.0×10^{-9}	1.7×10^{-9}	5.0×10^{-4}	1.7×10^{-10}
Es-253	20.5 d	M	5.0×10^{-4}	2.5×10^{-6}	2.1×10^{-6}	5.0×10^{-4}	6.1×10^{-9}
Es-254	276 d	M	5.0×10^{-4}	8.0×10^{-6}	6.0×10^{-6}	5.0×10^{-4}	2.8×10^{-8}
Es-254m	1.64 d	M	5.0×10^{-4}	4.4×10^{-7}	3.7×10^{-7}	5.0×10^{-4}	4.2×10^{-9}
Fermium							
Fm-252	22.7 h	M	5.0×10^{-4}	3.0×10^{-7}	2.6×10^{-7}	5.0×10^{-4}	2.7×10^{-9}
Fm-253	3.00 d	M	5.0×10^{-4}	3.7×10^{-7}	3.0×10^{-7}	5.0×10^{-4}	9.1×10^{-10}
Fm-254	3.24 h	M	5.0×10^{-4}	5.6×10^{-8}	7.7×10^{-8}	5.0×10^{-4}	4.4×10^{-10}
Fm-255	20.1 h	M	5.0×10^{-4}	2.5×10^{-7}	2.6×10^{-7}	5.0×10^{-4}	2.5×10^{-9}
Fm-257	101 d	M	5.0×10^{-4}	6.6×10^{-6}	5.2×10^{-6}	5.0×10^{-4}	1.5×10^{-8}

TABLE III.2A. WORKERS: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION AND INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Inhalation			Ingestion		
		Type	f_1	$e(g)_{1\ \mu m}$	$e(g)_{5\ \mu m}$	f_1	$e(g)$
Mendelevium							
Md-257	5.20 h	M	5.0×10^{-4}	2.3×10^{-8}	2.0×10^{-8}	5.0×10^{-4}	1.2×10^{-10}
Md-258	55.0 d	M	5.0×10^{-4}	5.5×10^{-6}	4.4×10^{-6}	5.0×10^{-4}	1.3×10^{-8}

^a m and m' denote metastable states of the radionuclide. The metastable state m' is of higher energy than the metastable state m.

Note: Types F, M and S denote fast, moderate and slow absorption from the lung, respectively. f_i : gut transfer factor; $e(g)$: effective dose per unit intake by age group.

TABLE III.2B. COMPOUNDS AND VALUES OF GUT TRANSFER FACTOR f_1 USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INGESTION FOR WORKERS

Element	Gut transfer factor f_1	Compounds
Hydrogen	1.000	Tritiated water (ingested)
	1.000	Organically bound tritium
Beryllium	0.005	All compounds
Carbon	1.000	Labelled organic compounds
Fluorine	1.000	All compounds
Sodium	1.000	All compounds
Magnesium	0.500	All compounds
Aluminium	0.010	All compounds
Silicon	0.010	All compounds
Phosphorus	0.800	All compounds
Sulphur	0.800	Inorganic compounds
	0.100	Elemental sulphur
	1.000	Organic sulphur
Chlorine	1.000	All compounds
Potassium	1.000	All compounds
Calcium	0.300	All compounds
Scandium	1.0×10^{-4}	All compounds
Titanium	0.010	All compounds
Vanadium	0.010	All compounds
Chromium	0.100	Hexavalent compounds
	0.010	Trivalent compounds
Manganese	0.100	All compounds
Iron	0.100	All compounds
Cobalt	0.100	All unspecified compounds
	0.050	Oxides, hydroxides and inorganic compounds
Nickel	0.050	All compounds
Copper	0.500	All compounds

TABLE III.2B. COMPOUNDS AND VALUES OF GUT TRANSFER FACTOR f_1 USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INGESTION FOR WORKERS (cont.)

Element	Gut transfer factor f_1	Compounds
Zinc	0.500	All compounds
Gallium	0.001	All compounds
Germanium	1.000	All compounds
Arsenic	0.500	All compounds
Selenium	0.800	All unspecified compounds
	0.050	Elemental selenium and selenides
Bromine	1.000	All compounds
Rubidium	1.000	All compounds
Strontium	0.300	All unspecified compounds
	0.010	Strontium titanate (SrTiO_3)
Yttrium	1.0×10^{-4}	All compounds
Zirconium	0.002	All compounds
Niobium	0.010	All compounds
Molybdenum	0.800	All unspecified compounds
	0.050	Molybdenum sulphide
Technetium	0.800	All compounds
Ruthenium	0.050	All compounds
Rhodium	0.050	All compounds
Palladium	0.005	All compounds
Silver	0.050	All compounds
Cadmium	0.050	All inorganic compounds
Indium	0.020	All compounds
Tin	0.020	All compounds
Antimony	0.100	All compounds
Tellurium	0.300	All compounds
Iodine	1.000	All compounds
Caesium	1.000	All compounds

TABLE III.2B. COMPOUNDS AND VALUES OF GUT TRANSFER FACTOR f_1 USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INGESTION FOR WORKERS (cont.)

Element	Gut transfer factor f_1	Compounds
Barium	0.100	All compounds
Lanthanum	5.0×10^{-4}	All compounds
Cerium	5.0×10^{-4}	All compounds
Praseodymium	5.0×10^{-4}	All compounds
Neodymium	5.0×10^{-4}	All compounds
Promethium	5.0×10^{-4}	All compounds
Samarium	5.0×10^{-4}	All compounds
Europium	5.0×10^{-4}	All compounds
Gadolinium	5.0×10^{-4}	All compounds
Terbium	5.0×10^{-4}	All compounds
Dysprosium	5.0×10^{-4}	All compounds
Holmium	5.0×10^{-4}	All compounds
Erbium	5.0×10^{-4}	All compounds
Thulium	5.0×10^{-4}	All compounds
Ytterbium	5.0×10^{-4}	All compounds
Lutetium	5.0×10^{-4}	All compounds
Hafnium	0.002	All compounds
Tantalum	0.001	All compounds
Tungsten	0.300	All unspecified compounds
	0.010	Tungstic acid
Rhenium	0.800	All compounds
Osmium	0.010	All compounds
Iridium	0.010	All compounds
Platinum	0.010	All compounds
Gold	0.100	All compounds
Mercury	0.020	All inorganic compounds

TABLE III.2B. COMPOUNDS AND VALUES OF GUT TRANSFER FACTOR f_1 USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INGESTION FOR WORKERS (cont.)

Element	Gut transfer factor f_1	Compounds
Mercury	1.000	Methyl mercury
	0.400	All unspecified organic compounds
Thallium	1.000	All compounds
Lead	0.200	All compounds
Bismuth	0.050	All compounds
Polonium	0.100	All compounds
Astatine	1.000	All compounds
Francium	1.000	All compounds
Radium	0.200	All compounds
Actinium	5.0×10^{-4}	All compounds
Thorium	5.0×10^{-4}	All unspecified compounds
	2.0×10^{-4}	Oxides and hydroxides
Protactinium	5.0×10^{-4}	All compounds
Uranium	0.020	All unspecified compounds
	0.002	Most tetravalent compounds, e.g. UO_2 , U_3O_8 , UF_4
Neptunium	5.0×10^{-4}	All compounds
Plutonium	5.0×10^{-4}	All unspecified compounds
	1.0×10^{-4}	Nitrates
	1.0×10^{-5}	Insoluble oxides
Americium	5.0×10^{-4}	All compounds
Curium	5.0×10^{-4}	All compounds
Berkelium	5.0×10^{-4}	All compounds
Californium	5.0×10^{-4}	All compounds
Einsteinium	5.0×10^{-4}	All compounds
Fermium	5.0×10^{-4}	All compounds
Mendelevium	5.0×10^{-4}	All compounds

TABLE III.2C. COMPOUNDS, LUNG ABSORPTION TYPES AND VALUES OF GUT TRANSFER FACTOR f_1 USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR WORKERS

Element	Absorption type(s)	Gut transfer factor f_1	Compounds
Beryllium	M	0.005	All unspecified compounds Oxides, halides and nitrates
	S	0.005	
Fluorine	F	1.000	Determined by combining cation
	M	1.000	Determined by combining cation
	S	1.000	Determined by combining cation
Sodium	F	1.000	All compounds
Magnesium	F	0.500	All unspecified compounds Oxides, hydroxides, carbides, halides and nitrates
	M	0.500	
Aluminium	F	0.010	All unspecified compounds Oxides, hydroxides, carbides, halides, nitrates and metallic aluminium
	M	0.010	
Silicon	F	0.010	All unspecified compounds Oxides, hydroxides, carbides and nitrates Aluminosilicate glass aerosol
	M	0.010	
	S	0.010	
Phosphorus	F	0.800	All unspecified compounds Some phosphates: determined by combining cation
	M	0.800	

TABLE III.2C. COMPOUNDS, LUNG ABSORPTION TYPES AND VALUES OF GUT TRANSFER FACTOR f_i USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR WORKERS (cont.)

Element	Absorption type(s)	Gut transfer factor f_i	Compounds
Sulphur	F	0.800	Sulphides and sulphates: determined by combining cation
	M	0.800	Elemental sulphur; Sulphides and sulphates: determined by combining cation
Chlorine	F	1.000	Determined by combining cation
	M	1.000	Determined by combining cation
Potassium	F	1.000	All compounds
Calcium	M	0.300	All compounds
Scandium	S	1.0×10^{-4}	All compounds
Titanium	F	0.010	All unspecified compounds
	M	0.010	Oxides, hydroxides, carbides, halides and nitrates
	S	0.010	Strontium titanate (SrTiO_3)
Vanadium	F	0.010	All unspecified compounds
	M	0.010	Oxides, hydroxides, carbides and halides
Chromium	F	0.100	All unspecified compounds
	M	0.100	Halides and nitrates
	S	0.100	Oxides and hydroxides

TABLE III.2C. COMPOUNDS, LUNG ABSORPTION TYPES AND VALUES OF GUT TRANSFER FACTOR f_i USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR WORKERS (cont.)

Element	Absorption type(s)	Gut transfer factor f_i	Compounds
Manganese	F	0.100	All unspecified compounds Oxides, hydroxides, halides and nitrates
	M	0.100	
Iron	F	0.100	All unspecified compounds Oxides, hydroxides and halides
	M	0.100	
Cobalt	M	0.100	All unspecified compounds Oxides, hydroxides, halides and nitrates
	S	0.050	
Nickel	F	0.050	All unspecified compounds Oxides, hydroxides and carbides
	M	0.050	
Copper	F	0.500	All unspecified inorganic compounds Sulphides, halides and nitrates Oxides and hydroxides
	M	0.500	
	S	0.500	
Zinc	S	0.500	All compounds
Gallium	F	0.001	All unspecified compounds Oxides, hydroxides, carbides, halides and nitrates
	M	0.001	
Germanium	F	1.000	All unspecified compounds Oxides, sulphides and halides
	M	1.000	

TABLE III.2C. COMPOUNDS, LUNG ABSORPTION TYPES AND VALUES OF GUT TRANSFER FACTOR f_i USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR WORKERS (cont.)

Element	Absorption type(s)	Gut transfer factor f_i	Compounds
Arsenic	M	0.500	All compounds
Selenium	F	0.800	All unspecified inorganic compounds Elemental selenium, oxides, hydroxides and carbides
	M	0.800	
Bromine	F	1.000	Determined by combining cation Determined by combining cation
	M	1.000	
Rubidium	F	1.000	All compounds
Strontium	F	0.300	All unspecified compounds Strontium titanate (SrTiO_3)
	S	0.010	
Yttrium	M	1.0×10^{-4}	All unspecified compounds Oxides and hydroxides
	S	1.0×10^{-4}	
Zirconium	F	0.002	All unspecified compounds Oxides, hydroxides, halides and nitrates Zirconium carbide
	M	0.002	
	S	0.002	
Niobium	M	0.010	All unspecified compounds Oxides and hydroxides
	S	0.010	

TABLE III.2C. COMPOUNDS, LUNG ABSORPTION TYPES AND VALUES OF GUT TRANSFER FACTOR f_i USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR WORKERS (cont.)

Element	Absorption type(s)	Gut transfer factor f_i	Compounds
Molybdenum	F	0.800	All unspecified compounds
	S	0.050	Molybdenum sulphide, oxides and hydroxides
Technetium	F	0.800	All unspecified compounds
	M	0.800	Oxides, hydroxides, halides and nitrates
Ruthenium	F	0.050	All unspecified compounds
	M	0.050	Halides
	S	0.050	Oxides and hydroxides
Rhodium	F	0.050	All unspecified compounds
	M	0.050	Halides
	S	0.050	Oxides and hydroxides
Palladium	F	0.005	All unspecified compounds
	M	0.005	Nitrates and halides
	S	0.005	Oxides and hydroxides
Silver	F	0.050	All unspecified compounds and metallic silver
	M	0.050	Nitrates and sulphides
	S	0.050	Oxides, hydroxides and carbides

TABLE III.2C. COMPOUNDS, LUNG ABSORPTION TYPES AND VALUES OF GUT TRANSFER FACTOR f_i USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR WORKERS (cont.)

Element	Absorption type(s)	Gut transfer factor f_i	Compounds
Cadmium	F	0.050	All unspecified compounds
	M	0.050	Sulphides, halides and nitrates
	S	0.050	Oxides and hydroxides
Indium	F	0.020	All unspecified compounds
	M	0.020	Oxides, hydroxides, halides and nitrates
Tin	F	0.020	All unspecified compounds
	M	0.020	Stannic phosphate, sulphides, oxides, hydroxides, halides and nitrates
Antimony	F	0.100	All unspecified compounds
	M	0.010	Oxides, hydroxides, halides, sulphides, sulphates and nitrates
Tellurium	F	0.300	All unspecified compounds
	M	0.300	Oxides, hydroxides and nitrates
Iodine	F	1.000	All compounds
Caesium	F	1.000	All compounds
Barium	F	0.100	All compounds

TABLE III.2C. COMPOUNDS, LUNG ABSORPTION TYPES AND VALUES OF GUT TRANSFER FACTOR f_1 USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR WORKERS (cont.)

Element	Absorption type(s)	Gut transfer factor f_1	Compounds
Lanthanum	F	5.0×10^{-4}	All unspecified compounds Oxides and hydroxides
	M	5.0×10^{-4}	
Cerium	M	5.0×10^{-4}	All unspecified compounds Oxides, hydroxides and fluorides
	S	5.0×10^{-4}	
Praseodymium	M	5.0×10^{-4}	All unspecified compounds Oxides, hydroxides, carbides and fluorides
	S	5.0×10^{-4}	
Neodymium	M	5.0×10^{-4}	All unspecified compounds Oxides, hydroxides, carbides and fluorides
	S	5.0×10^{-4}	
Promethium	M	5.0×10^{-4}	All unspecified compounds Oxides, hydroxides, carbides and fluorides
	S	5.0×10^{-4}	
Samarium	M	5.0×10^{-4}	All compounds
Europium	M	5.0×10^{-4}	All compounds
Gadolinium	F	5.0×10^{-4}	All unspecified compounds Oxides, hydroxides and fluorides
	M	5.0×10^{-4}	
Terbium	M	5.0×10^{-4}	All compounds

TABLE III.2C. COMPOUNDS, LUNG ABSORPTION TYPES AND VALUES OF GUT TRANSFER FACTOR f_i USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR WORKERS (cont.)

Element	Absorption type(s)	Gut transfer factor f_i	Compounds
Dysprosium	M	5.0×10^{-4}	All compounds
Holmium	M	5.0×10^{-4}	All unspecified compounds
Erbium	M	5.0×10^{-4}	All compounds
Thulium	M	5.0×10^{-4}	All compounds
Ytterbium	M S	5.0×10^{-4} 5.0×10^{-4}	All unspecified compounds Oxides, hydroxides and fluorides
Lutetium	M S	5.0×10^{-4} 5.0×10^{-4}	All unspecified compounds Oxides, hydroxides and fluorides
Hafnium	F M	0.002 0.002	All unspecified compounds Oxides, hydroxides, halides, carbides and nitrates
Tantalum	M S	0.001 0.001	All unspecified compounds Elemental tantalum, oxides, hydroxides, halides, carbides, nitrates and nitrides
Tungsten	F	0.300	All compounds

TABLE III.2C. COMPOUNDS, LUNG ABSORPTION TYPES AND VALUES OF GUT TRANSFER FACTOR f_i USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR WORKERS (cont.)

Element	Absorption type(s)	Gut transfer factor f_i	Compounds
Rhenium	F	0.800	All unspecified compounds Oxides, hydroxides, halides and nitrates
	M	0.800	
Osmium	F	0.010	All unspecified compounds Halides and nitrates Oxides and hydroxides
	M	0.010	
	S	0.010	
Iridium	F	0.010	All unspecified compounds Metallic iridium, halides and nitrates Oxides and hydroxides
	M	0.010	
	S	0.010	
Platinum	F	0.010	All compounds
Gold	F	0.100	All unspecified compounds Halides and nitrates Oxides and hydroxides
	M	0.100	
	S	0.100	
Mercury	F	0.020	Sulphates Oxides, hydroxides, halides, nitrates and sulphides
	M	0.020	
Mercury	F	0.400	All organic compounds
Thallium	F	1.000	All compounds

TABLE III.2C. COMPOUNDS, LUNG ABSORPTION TYPES AND VALUES OF GUT TRANSFER FACTOR f_1 USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR WORKERS (cont.)

Element	Absorption type(s)	Gut transfer factor f_1	Compounds
Lead	F	0.200	All compounds
Bismuth	F	0.050	Bismuth nitrate
	M	0.050	All unspecified compounds
Polonium	F	0.100	All unspecified compounds
	M	0.100	Oxides, hydroxides and nitrates
Astatine	F	1.000	Determined by combining cation
	M	1.000	Determined by combining cation
Francium	F	1.000	All compounds
Radium	M	0.200	All compounds
Actinium	F	5.0×10^{-4}	All unspecified compounds
	M	5.0×10^{-4}	Halides and nitrates
	S	5.0×10^{-4}	Oxides and hydroxides
Thorium	M	5.0×10^{-4}	All unspecified compounds
	S	2.0×10^{-4}	Oxides and hydroxides

TABLE III.2C. COMPOUNDS, LUNG ABSORPTION TYPES AND VALUES OF GUT TRANSFER FACTOR f_1 USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR WORKERS (cont.)

Element	Absorption type(s)	Gut transfer factor f_1	Compounds
Protactinium	M	5.0×10^{-4}	All unspecified compounds Oxides and hydroxides
	S	5.0×10^{-4}	
Uranium	F	0.020	Most hexavalent compounds, e.g. UF_6 , UO_2F_2 and $\text{UO}_2(\text{NO}_3)_2$ Less soluble compounds, e.g. UO_3 , UF_4 , UCl_4 and most other hexavalent compounds Highly insoluble compounds, e.g. UO_2 and U_3O_8
	M	0.020	
	S	0.002	
Neptunium	M	5.0×10^{-4}	All compounds
Plutonium	M	5.0×10^{-4}	All unspecified compounds Insoluble oxides
	S	1.0×10^{-4}	
Americium	M	5.0×10^{-4}	All compounds
Curium	M	5.0×10^{-4}	All compounds
Berkelium	M	5.0×10^{-4}	All compounds
Californium	M	5.0×10^{-4}	All compounds
Einsteinium	M	5.0×10^{-4}	All compounds

TABLE III.2C. COMPOUNDS, LUNG ABSORPTION TYPES AND VALUES OF GUT TRANSFER FACTOR f_i USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR WORKERS (cont.)

Element	Absorption type(s)	Gut transfer factor f_i	Compounds
Fermium	M	5.0×10^{-4}	All compounds
Mendelevium	M	5.0×10^{-4}	All compounds

Note: Types F, M and S denote fast, moderate and slow absorption from the lung, respectively.

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Hydrogen														
Tritiated water	12.3 a	1.000	6.4 × 10 ^{−11}	1.000	4.8 × 10 ^{−11}	3.1 × 10 ^{−11}	2.3 × 10 ^{−11}	1.8 × 10 ^{−11}	1.8 × 10 ^{−11}	1.8 × 10 ^{−11}	1.8 × 10 ^{−11}	1.8 × 10 ^{−11}	1.8 × 10 ^{−11}	
Organically bound tritium	12.3 a	1.000	1.2 × 10 ^{−10}	1.000	1.2 × 10 ^{−10}	7.3 × 10 ^{−11}	5.7 × 10 ^{−11}	4.2 × 10 ^{−11}	4.2 × 10 ^{−11}	4.2 × 10 ^{−11}	4.2 × 10 ^{−11}	4.2 × 10 ^{−11}	4.2 × 10 ^{−11}	
Beryllium														
Be-7	53.3 d	0.020	1.8 × 10 ^{−10}	0.005	1.3 × 10 ^{−10}	7.7 × 10 ^{−11}	5.3 × 10 ^{−11}	3.5 × 10 ^{−11}	3.5 × 10 ^{−11}	3.5 × 10 ^{−11}	3.5 × 10 ^{−11}	2.8 × 10 ^{−11}	2.8 × 10 ^{−11}	
Be-10	1.60 × 10 ⁶ a	0.020	1.4 × 10 ^{−8}	0.005	8.0 × 10 ^{−9}	4.1 × 10 ^{−9}	2.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.1 × 10 ^{−9}	1.1 × 10 ^{−9}	
Carbon														
C-11	0.340 h	1.000	2.6 × 10 ^{−10}	1.000	1.5 × 10 ^{−10}	7.3 × 10 ^{−11}	4.3 × 10 ^{−11}	3.0 × 10 ^{−11}	3.0 × 10 ^{−11}	3.0 × 10 ^{−11}	3.0 × 10 ^{−11}	2.4 × 10 ^{−11}	2.4 × 10 ^{−11}	
C-14	5.73 × 10 ³ a	1.000	1.4 × 10 ^{−9}	1.000	1.6 × 10 ^{−9}	9.9 × 10 ^{−10}	8.0 × 10 ^{−10}	5.7 × 10 ^{−10}	5.7 × 10 ^{−10}	5.7 × 10 ^{−10}	5.7 × 10 ^{−10}	5.8 × 10 ^{−10}	5.8 × 10 ^{−10}	
Fluorine														
F-18	1.83 h	1.000	5.2 × 10 ^{−10}	1.000	3.0 × 10 ^{−10}	1.5 × 10 ^{−10}	9.1 × 10 ^{−11}	6.2 × 10 ^{−11}	6.2 × 10 ^{−11}	6.2 × 10 ^{−11}	6.2 × 10 ^{−11}	4.9 × 10 ^{−11}	4.9 × 10 ^{−11}	
Sodium														
Na-22	2.60 a	1.000	2.1 × 10 ^{−8}	1.000	1.5 × 10 ^{−8}	8.4 × 10 ^{−9}	5.5 × 10 ^{−9}	3.7 × 10 ^{−9}	3.7 × 10 ^{−9}	3.7 × 10 ^{−9}	3.7 × 10 ^{−9}	3.2 × 10 ^{−9}	3.2 × 10 ^{−9}	
Na-24	15.0 h	1.000	3.5 × 10 ^{−9}	1.000	2.3 × 10 ^{−9}	1.2 × 10 ^{−9}	7.7 × 10 ^{−10}	5.2 × 10 ^{−10}	5.2 × 10 ^{−10}	5.2 × 10 ^{−10}	5.2 × 10 ^{−10}	4.3 × 10 ^{−10}	4.3 × 10 ^{−10}	

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Magnesium														
Mg-28	20.9 h	1.000	1.2 × 10 ^{−8}	0.500	1.4 × 10 ^{−8}	7.4 × 10 ^{−9}	4.5 × 10 ^{−9}	2.7 × 10 ^{−9}	2.2 × 10 ^{−9}					
Aluminium														
Al-26	7.16 × 10 ⁵ a	0.020	3.4 × 10 ^{−8}	0.010	2.1 × 10 ^{−8}	1.1 × 10 ^{−8}	7.1 × 10 ^{−9}	4.3 × 10 ^{−9}	3.5 × 10 ^{−9}					
Silicon														
Si-31	2.62 h	0.020	1.9 × 10 ^{−9}	0.010	1.0 × 10 ^{−9}	5.1 × 10 ^{−10}	3.0 × 10 ^{−10}	1.8 × 10 ^{−10}	1.6 × 10 ^{−10}					
Si-32	4.50 × 10 ² a	0.020	7.3 × 10 ^{−9}	0.010	4.1 × 10 ^{−9}	2.0 × 10 ^{−9}	1.2 × 10 ^{−9}	7.0 × 10 ^{−10}	5.6 × 10 ^{−10}					
Phosphorus														
P-32	14.3 d	1.000	3.1 × 10 ^{−8}	0.800	1.9 × 10 ^{−8}	9.4 × 10 ^{−9}	5.3 × 10 ^{−9}	3.1 × 10 ^{−9}	2.4 × 10 ^{−9}					
P-33	25.4 d	1.000	2.7 × 10 ^{−9}	0.800	1.8 × 10 ^{−9}	9.1 × 10 ^{−10}	5.3 × 10 ^{−10}	3.1 × 10 ^{−10}	2.4 × 10 ^{−10}					
Sulphur														
S-35 (inorganic)	87.4 d	1.000	1.3 × 10 ^{−9}	1.000	8.7 × 10 ^{−10}	4.4 × 10 ^{−10}	2.7 × 10 ^{−10}	1.6 × 10 ^{−10}	1.3 × 10 ^{−10}					
S-35 (organic)	87.4 d	1.000	7.7 × 10 ^{−9}	1.000	5.4 × 10 ^{−9}	2.7 × 10 ^{−9}	1.6 × 10 ^{−9}	9.5 × 10 ^{−10}	7.7 × 10 ^{−10}					

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Chlorine														
Cl-36	3.01 × 10 ⁵ a	1.000	9.8 × 10 ^{−9}	1.000	6.3 × 10 ^{−9}	3.2 × 10 ^{−9}	1.9 × 10 ^{−9}	1.2 × 10 ^{−9}	1.2 × 10 ^{−9}	1.2 × 10 ^{−9}	1.2 × 10 ^{−9}	9.3 × 10 ^{−10}	9.3 × 10 ^{−10}	
Cl-38	0.620 h	1.000	1.4 × 10 ^{−9}	1.000	7.7 × 10 ^{−10}	3.8 × 10 ^{−10}	2.2 × 10 ^{−10}	1.5 × 10 ^{−10}	1.5 × 10 ^{−10}	1.5 × 10 ^{−10}	1.5 × 10 ^{−10}	1.2 × 10 ^{−10}	1.2 × 10 ^{−10}	
Cl-39	0.927 h	1.000	9.7 × 10 ^{−10}	1.000	5.5 × 10 ^{−10}	2.7 × 10 ^{−10}	1.6 × 10 ^{−10}	1.1 × 10 ^{−10}	1.6 × 10 ^{−10}	1.6 × 10 ^{−10}	1.1 × 10 ^{−10}	8.5 × 10 ^{−11}	8.5 × 10 ^{−11}	
Potassium														
K-40	1.28 × 10 ⁹ a	1.000	6.2 × 10 ^{−8}	1.000	4.2 × 10 ^{−8}	2.1 × 10 ^{−8}	1.3 × 10 ^{−8}	7.6 × 10 ^{−9}	1.3 × 10 ^{−8}	7.6 × 10 ^{−9}	7.6 × 10 ^{−9}	6.2 × 10 ^{−9}	6.2 × 10 ^{−9}	
K-42	12.4 h	1.000	5.1 × 10 ^{−9}	1.000	3.0 × 10 ^{−9}	1.5 × 10 ^{−9}	8.6 × 10 ^{−10}	5.4 × 10 ^{−10}	8.6 × 10 ^{−10}	5.4 × 10 ^{−10}	5.4 × 10 ^{−10}	4.3 × 10 ^{−10}	4.3 × 10 ^{−10}	
K-43	22.6 h	1.000	2.3 × 10 ^{−9}	1.000	1.4 × 10 ^{−9}	7.6 × 10 ^{−10}	4.7 × 10 ^{−10}	3.0 × 10 ^{−10}	4.7 × 10 ^{−10}	3.0 × 10 ^{−10}	3.0 × 10 ^{−10}	2.5 × 10 ^{−10}	2.5 × 10 ^{−10}	
K-44	0.369 h	1.000	1.0 × 10 ^{−9}	1.000	5.5 × 10 ^{−10}	2.7 × 10 ^{−10}	1.6 × 10 ^{−10}	1.1 × 10 ^{−10}	1.6 × 10 ^{−10}	1.1 × 10 ^{−10}	1.1 × 10 ^{−10}	8.4 × 10 ^{−11}	8.4 × 10 ^{−11}	
K-45	0.333 h	1.000	6.2 × 10 ^{−10}	1.000	3.5 × 10 ^{−10}	1.7 × 10 ^{−10}	9.9 × 10 ^{−11}	6.8 × 10 ^{−11}	9.9 × 10 ^{−11}	6.8 × 10 ^{−11}	6.8 × 10 ^{−11}	5.4 × 10 ^{−11}	5.4 × 10 ^{−11}	
Calcium^b														
Ca-41	1.40 × 10 ⁵ a	0.600	1.2 × 10 ^{−9}	0.300	5.2 × 10 ^{−10}	3.9 × 10 ^{−10}	4.8 × 10 ^{−10}	5.0 × 10 ^{−10}	4.8 × 10 ^{−10}	5.0 × 10 ^{−10}	5.0 × 10 ^{−10}	1.9 × 10 ^{−10}	1.9 × 10 ^{−10}	
Ca-45	163 d	0.600	1.1 × 10 ^{−8}	0.300	4.9 × 10 ^{−9}	2.6 × 10 ^{−9}	1.8 × 10 ^{−9}	1.3 × 10 ^{−9}	1.8 × 10 ^{−9}	1.3 × 10 ^{−9}	1.3 × 10 ^{−9}	7.1 × 10 ^{−10}	7.1 × 10 ^{−10}	
Ca-47	4.53 d	0.600	1.3 × 10 ^{−8}	0.300	9.3 × 10 ^{−9}	4.9 × 10 ^{−9}	3.0 × 10 ^{−9}	1.8 × 10 ^{−9}	3.0 × 10 ^{−9}	1.8 × 10 ^{−9}	1.8 × 10 ^{−9}	1.6 × 10 ^{−9}	1.6 × 10 ^{−9}	
Scandium														
Sc-43	3.89 h	0.001	1.8 × 10 ^{−9}	1.0 × 10 ^{−4}	1.2 × 10 ^{−9}	6.1 × 10 ^{−10}	3.7 × 10 ^{−10}	2.3 × 10 ^{−10}	3.7 × 10 ^{−10}	2.3 × 10 ^{−10}	2.3 × 10 ^{−10}	1.9 × 10 ^{−10}	1.9 × 10 ^{−10}	
Sc-44	3.93 h	0.001	3.5 × 10 ^{−9}	1.0 × 10 ^{−4}	2.2 × 10 ^{−9}	1.2 × 10 ^{−9}	7.1 × 10 ^{−10}	4.4 × 10 ^{−10}	7.1 × 10 ^{−10}	4.4 × 10 ^{−10}	4.4 × 10 ^{−10}	3.5 × 10 ^{−10}	3.5 × 10 ^{−10}	
Sc-44m	2.44 d	0.001	2.4 × 10 ^{−8}	1.0 × 10 ^{−4}	1.6 × 10 ^{−8}	8.3 × 10 ^{−9}	5.1 × 10 ^{−9}	3.1 × 10 ^{−9}	5.1 × 10 ^{−9}	3.1 × 10 ^{−9}	3.1 × 10 ^{−9}	2.4 × 10 ^{−9}	2.4 × 10 ^{−9}	

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Sc-46	83.8 d	0.001	1.1 × 10 ^{−8}	1.0 × 10 ^{−4}	7.9 × 10 ^{−9}	4.4 × 10 ^{−9}	2.9 × 10 ^{−9}	1.8 × 10 ^{−9}	1.5 × 10 ^{−9}				
Sc-47	3.35 d	0.001	6.1 × 10 ^{−9}	1.0 × 10 ^{−4}	3.9 × 10 ^{−9}	2.0 × 10 ^{−9}	1.2 × 10 ^{−9}	6.8 × 10 ^{−10}	5.4 × 10 ^{−10}				
Sc-48	1.82 d	0.001	1.3 × 10 ^{−8}	1.0 × 10 ^{−4}	9.3 × 10 ^{−9}	5.1 × 10 ^{−9}	3.3 × 10 ^{−9}	2.1 × 10 ^{−9}	1.7 × 10 ^{−9}				
Sc-49	0.956 h	0.001	1.0 × 10 ^{−9}	1.0 × 10 ^{−4}	5.7 × 10 ^{−10}	2.8 × 10 ^{−10}	1.6 × 10 ^{−10}	1.0 × 10 ^{−10}	8.2 × 10 ^{−11}				
Titanium													
Ti-44	47.3 a	0.020	5.5 × 10 ^{−8}	0.010	3.1 × 10 ^{−8}	1.7 × 10 ^{−8}	1.1 × 10 ^{−8}	6.9 × 10 ^{−9}	5.8 × 10 ^{−9}				
Ti-45	3.08 h	0.020	1.6 × 10 ^{−9}	0.010	9.8 × 10 ^{−10}	5.0 × 10 ^{−10}	3.1 × 10 ^{−10}	1.9 × 10 ^{−10}	1.5 × 10 ^{−10}				
Vanadium													
V-47	0.543 h	0.020	7.3 × 10 ^{−10}	0.010	4.1 × 10 ^{−10}	2.0 × 10 ^{−10}	1.2 × 10 ^{−10}	8.0 × 10 ^{−11}	6.3 × 10 ^{−11}				
V-48	16.2 d	0.020	1.5 × 10 ^{−8}	0.010	1.1 × 10 ^{−8}	5.9 × 10 ^{−9}	3.9 × 10 ^{−9}	2.5 × 10 ^{−9}	2.0 × 10 ^{−9}				
V-49	330 d	0.020	2.2 × 10 ^{−10}	0.010	1.4 × 10 ^{−10}	6.9 × 10 ^{−11}	4.0 × 10 ^{−11}	2.3 × 10 ^{−11}	1.8 × 10 ^{−11}				
Chromium													
Cr-48	23.0 h	0.200	1.4 × 10 ^{−9}	0.100	9.9 × 10 ^{−10}	5.7 × 10 ^{−10}	3.8 × 10 ^{−10}	2.5 × 10 ^{−10}	2.0 × 10 ^{−10}				
		0.020	1.4 × 10 ^{−9}	0.010	9.9 × 10 ^{−10}	5.7 × 10 ^{−10}	3.8 × 10 ^{−10}	2.5 × 10 ^{−10}	2.0 × 10 ^{−10}				
Cr-49	0.702 h	0.200	6.8 × 10 ^{−10}	0.100	3.9 × 10 ^{−10}	2.0 × 10 ^{−10}	1.1 × 10 ^{−10}	7.7 × 10 ^{−11}	6.1 × 10 ^{−11}				
		0.020	6.8 × 10 ^{−10}	0.010	3.9 × 10 ^{−10}	2.0 × 10 ^{−10}	1.1 × 10 ^{−10}	7.7 × 10 ^{−11}	6.1 × 10 ^{−11}				
Cr-51	27.7 d	0.200	3.5 × 10 ^{−10}	0.100	2.3 × 10 ^{−10}	1.2 × 10 ^{−10}	7.8 × 10 ^{−11}	4.8 × 10 ^{−11}	3.8 × 10 ^{−11}				
		0.020	3.3 × 10 ^{−10}	0.010	2.2 × 10 ^{−10}	1.2 × 10 ^{−10}	7.5 × 10 ^{−11}	4.6 × 10 ^{−11}	3.7 × 10 ^{−11}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Manganese														
Mn-51	0.770 h	0.200	1.1 × 10 ^{−9}	0.100	6.1 × 10 ^{−10}	3.0 × 10 ^{−10}	1.8 × 10 ^{−10}	1.2 × 10 ^{−10}	9.3 × 10 ^{−11}					
Mn-52	5.59 d	0.200	1.2 × 10 ^{−8}	0.100	8.8 × 10 ^{−9}	5.1 × 10 ^{−9}	3.4 × 10 ^{−9}	2.2 × 10 ^{−9}	1.8 × 10 ^{−9}					
Mn-52m	0.352 h	0.200	7.8 × 10 ^{−10}	0.100	4.4 × 10 ^{−10}	2.2 × 10 ^{−10}	1.3 × 10 ^{−10}	8.8 × 10 ^{−11}	6.9 × 10 ^{−11}					
Mn-53	3.70 × 10 ⁶ a	0.200	4.1 × 10 ^{−10}	0.100	2.2 × 10 ^{−10}	1.1 × 10 ^{−10}	6.5 × 10 ^{−11}	3.7 × 10 ^{−11}	3.0 × 10 ^{−11}					
Mn-54	312 d	0.200	5.4 × 10 ^{−9}	0.100	3.1 × 10 ^{−9}	1.9 × 10 ^{−9}	1.3 × 10 ^{−9}	8.7 × 10 ^{−10}	7.1 × 10 ^{−10}					
Mn-56	2.58 h	0.200	2.7 × 10 ^{−9}	0.100	1.7 × 10 ^{−9}	8.5 × 10 ^{−10}	5.1 × 10 ^{−10}	3.2 × 10 ^{−10}	2.5 × 10 ^{−10}					
Iron^c														
Fe-52	8.28 h	0.600	1.3 × 10 ^{−8}	0.100	9.1 × 10 ^{−9}	4.6 × 10 ^{−9}	2.8 × 10 ^{−9}	1.7 × 10 ^{−9}	1.4 × 10 ^{−9}					
Fe-55	2.70 a	0.600	7.6 × 10 ^{−9}	0.100	2.4 × 10 ^{−9}	1.7 × 10 ^{−9}	1.1 × 10 ^{−9}	7.7 × 10 ^{−10}	3.3 × 10 ^{−10}					
Fe-59	44.5 d	0.600	3.9 × 10 ^{−8}	0.100	1.3 × 10 ^{−8}	7.5 × 10 ^{−9}	4.7 × 10 ^{−9}	3.1 × 10 ^{−9}	1.8 × 10 ^{−9}					
Fe-60	1.00 × 10 ⁵ a	0.600	7.9 × 10 ^{−7}	0.100	2.7 × 10 ^{−7}	2.7 × 10 ^{−7}	2.5 × 10 ^{−7}	2.3 × 10 ^{−7}	1.1 × 10 ^{−7}					
Cobalt^d														
Co-55	17.5 h	0.600	6.0 × 10 ^{−9}	0.100	5.5 × 10 ^{−9}	2.9 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	1.0 × 10 ^{−9}					
Co-56	78.7 d	0.600	2.5 × 10 ^{−8}	0.100	1.5 × 10 ^{−8}	8.8 × 10 ^{−9}	5.8 × 10 ^{−9}	3.8 × 10 ^{−9}	2.5 × 10 ^{−9}					
Co-57	271 d	0.600	2.9 × 10 ^{−9}	0.100	1.6 × 10 ^{−9}	8.9 × 10 ^{−10}	5.8 × 10 ^{−10}	3.7 × 10 ^{−10}	2.1 × 10 ^{−10}					
Co-58	70.8 d	0.600	7.3 × 10 ^{−9}	0.100	4.4 × 10 ^{−9}	2.6 × 10 ^{−9}	1.7 × 10 ^{−9}	1.1 × 10 ^{−9}	7.4 × 10 ^{−10}					
Co-58m	9.15 h	0.600	2.0 × 10 ^{−10}	0.100	1.5 × 10 ^{−10}	7.8 × 10 ^{−11}	4.7 × 10 ^{−11}	2.8 × 10 ^{−11}	2.4 × 10 ^{−11}					
Co-60	5.27 a	0.600	5.4 × 10 ^{−8}	0.100	2.7 × 10 ^{−8}	1.7 × 10 ^{−8}	1.1 × 10 ^{−8}	7.9 × 10 ^{−9}	3.4 × 10 ^{−9}					

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Co-60m	0.174 h	0.600	2.2 × 10 ^{−11}	0.100	1.2 × 10 ^{−11}	5.7 × 10 ^{−12}	3.2 × 10 ^{−12}	2.2 × 10 ^{−12}	1.7 × 10 ^{−12}				
Co-61	1.65 h	0.600	8.2 × 10 ^{−10}	0.100	5.1 × 10 ^{−10}	2.5 × 10 ^{−10}	1.4 × 10 ^{−10}	9.2 × 10 ^{−11}	7.4 × 10 ^{−11}				
Co-62m	0.232 h	0.600	5.3 × 10 ^{−10}	0.100	3.0 × 10 ^{−10}	1.5 × 10 ^{−10}	8.7 × 10 ^{−11}	6.0 × 10 ^{−11}	4.7 × 10 ^{−11}				
Nickel													
Ni-56	6.10 d	0.100	5.3 × 10 ^{−9}	0.050	4.0 × 10 ^{−9}	2.3 × 10 ^{−9}	1.6 × 10 ^{−9}	1.1 × 10 ^{−9}	8.6 × 10 ^{−10}				
Ni-57	1.50 d	0.100	6.8 × 10 ^{−9}	0.050	4.9 × 10 ^{−9}	2.7 × 10 ^{−9}	1.7 × 10 ^{−9}	1.1 × 10 ^{−9}	8.7 × 10 ^{−10}				
Ni-59	7.50 × 10 ⁴ a	0.100	6.4 × 10 ^{−10}	0.050	3.4 × 10 ^{−10}	1.9 × 10 ^{−10}	1.1 × 10 ^{−10}	7.3 × 10 ^{−11}	6.3 × 10 ^{−11}				
Ni-63	96.0 a	0.100	1.6 × 10 ^{−9}	0.050	8.4 × 10 ^{−10}	4.6 × 10 ^{−10}	2.8 × 10 ^{−10}	1.8 × 10 ^{−10}	1.5 × 10 ^{−10}				
Ni-65	2.52 h	0.100	2.1 × 10 ^{−9}	0.050	1.3 × 10 ^{−9}	6.3 × 10 ^{−10}	3.8 × 10 ^{−10}	2.3 × 10 ^{−10}	1.8 × 10 ^{−10}				
Ni-66	2.27 d	0.100	3.3 × 10 ^{−8}	0.050	2.2 × 10 ^{−8}	1.1 × 10 ^{−8}	6.6 × 10 ^{−9}	3.7 × 10 ^{−9}	3.0 × 10 ^{−9}				
Copper													
Cu-60	0.387 h	1.000	7.0 × 10 ^{−10}	0.500	4.2 × 10 ^{−10}	2.2 × 10 ^{−10}	1.3 × 10 ^{−10}	8.9 × 10 ^{−11}	7.0 × 10 ^{−11}				
Cu-61	3.41 h	1.000	7.1 × 10 ^{−10}	0.500	7.5 × 10 ^{−10}	3.9 × 10 ^{−10}	2.3 × 10 ^{−10}	1.5 × 10 ^{−10}	1.2 × 10 ^{−10}				
Cu-64	12.7 h	1.000	5.2 × 10 ^{−10}	0.500	8.3 × 10 ^{−10}	4.2 × 10 ^{−10}	2.5 × 10 ^{−10}	1.5 × 10 ^{−10}	1.2 × 10 ^{−10}				
Cu-67	2.58 d	1.000	2.1 × 10 ^{−9}	0.500	2.4 × 10 ^{−9}	1.2 × 10 ^{−9}	7.2 × 10 ^{−10}	4.2 × 10 ^{−10}	3.4 × 10 ^{−10}				
Zinc													
Zn-62	9.26 h	1.000	4.2 × 10 ^{−9}	0.500	6.5 × 10 ^{−9}	3.3 × 10 ^{−9}	2.0 × 10 ^{−9}	1.2 × 10 ^{−9}	9.4 × 10 ^{−10}				
Zn-63	0.635 h	1.000	8.7 × 10 ^{−10}	0.500	5.2 × 10 ^{−10}	2.6 × 10 ^{−10}	1.5 × 10 ^{−10}	1.0 × 10 ^{−10}	7.9 × 10 ^{−11}				
Zn-65	244 d	1.000	3.6 × 10 ^{−8}	0.500	1.6 × 10 ^{−8}	9.7 × 10 ^{−9}	6.4 × 10 ^{−9}	4.5 × 10 ^{−9}	3.9 × 10 ^{−9}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Zn-69	0.950 h	1.000	3.5 × 10 ^{−10}	0.500	2.2 × 10 ^{−10}	1.1 × 10 ^{−10}	6.0 × 10 ^{−11}	3.9 × 10 ^{−11}	3.1 × 10 ^{−11}					
Zn-69m	13.8 h	1.000	1.3 × 10 ^{−9}	0.500	2.3 × 10 ^{−9}	1.2 × 10 ^{−9}	7.0 × 10 ^{−10}	4.1 × 10 ^{−10}	3.3 × 10 ^{−10}					
Zn-71m	3.92 h	1.000	1.4 × 10 ^{−9}	0.500	1.5 × 10 ^{−9}	7.8 × 10 ^{−10}	4.8 × 10 ^{−10}	3.0 × 10 ^{−10}	2.4 × 10 ^{−10}					
Zn-72	1.94 d	1.000	8.7 × 10 ^{−9}	0.500	8.6 × 10 ^{−9}	4.5 × 10 ^{−9}	2.8 × 10 ^{−9}	1.7 × 10 ^{−9}	1.4 × 10 ^{−9}					
Gallium														
Ga-65	0.253 h	0.010	4.3 × 10 ^{−10}	0.001	2.4 × 10 ^{−10}	1.2 × 10 ^{−10}	6.9 × 10 ^{−11}	4.7 × 10 ^{−11}	3.7 × 10 ^{−11}					
Ga-66	9.40 h	0.010	1.2 × 10 ^{−8}	0.001	7.9 × 10 ^{−9}	4.0 × 10 ^{−9}	2.5 × 10 ^{−9}	1.5 × 10 ^{−9}	1.2 × 10 ^{−9}					
Ga-67	3.26 d	0.010	1.8 × 10 ^{−9}	0.001	1.2 × 10 ^{−9}	6.4 × 10 ^{−10}	4.0 × 10 ^{−10}	2.4 × 10 ^{−10}	1.9 × 10 ^{−10}					
Ga-68	1.13 h	0.010	1.2 × 10 ^{−9}	0.001	6.7 × 10 ^{−10}	3.4 × 10 ^{−10}	2.0 × 10 ^{−10}	1.3 × 10 ^{−10}	1.0 × 10 ^{−10}					
Ga-70	0.353 h	0.010	3.9 × 10 ^{−10}	0.001	2.2 × 10 ^{−10}	1.0 × 10 ^{−10}	5.9 × 10 ^{−11}	4.0 × 10 ^{−11}	3.1 × 10 ^{−11}					
Ga-72	14.1 h	0.010	1.0 × 10 ^{−8}	0.001	6.8 × 10 ^{−9}	3.6 × 10 ^{−9}	2.2 × 10 ^{−9}	1.4 × 10 ^{−9}	1.1 × 10 ^{−9}					
Ga-73	4.91 h	0.010	3.0 × 10 ^{−9}	0.001	1.9 × 10 ^{−9}	9.3 × 10 ^{−10}	5.5 × 10 ^{−10}	3.3 × 10 ^{−10}	2.6 × 10 ^{−10}					
Germanium														
Ge-66	2.27 h	1.000	8.3 × 10 ^{−10}	1.000	5.3 × 10 ^{−10}	2.9 × 10 ^{−10}	1.9 × 10 ^{−10}	1.3 × 10 ^{−10}	1.0 × 10 ^{−10}					
Ge-67	0.312 h	1.000	7.7 × 10 ^{−10}	1.000	4.2 × 10 ^{−10}	2.1 × 10 ^{−10}	1.2 × 10 ^{−10}	8.2 × 10 ^{−11}	6.5 × 10 ^{−11}					
Ge-68	288 d	1.000	1.2 × 10 ^{−8}	1.000	8.0 × 10 ^{−9}	4.2 × 10 ^{−9}	2.6 × 10 ^{−9}	1.6 × 10 ^{−9}	1.3 × 10 ^{−9}					
Ge-69	1.63 d	1.000	2.0 × 10 ^{−9}	1.000	1.3 × 10 ^{−9}	7.1 × 10 ^{−10}	4.6 × 10 ^{−10}	3.0 × 10 ^{−10}	2.4 × 10 ^{−10}					
Ge-71	11.8 d	1.000	1.2 × 10 ^{−10}	1.000	7.8 × 10 ^{−11}	4.0 × 10 ^{−11}	2.4 × 10 ^{−11}	1.5 × 10 ^{−11}	1.2 × 10 ^{−11}					
Ge-75	1.38 h	1.000	5.5 × 10 ^{−10}	1.000	3.1 × 10 ^{−10}	1.5 × 10 ^{−10}	8.7 × 10 ^{−11}	5.9 × 10 ^{−11}	4.6 × 10 ^{−11}					

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		f_1	$e(g)$		$e(g)$		$e(g)$		$e(g)$		$e(g)$		$e(g)$
Ge-77	11.3 h	1.000	3.0×10^{-9}	1.000	1.8×10^{-9}		9.9×10^{-10}		6.2×10^{-10}		4.1×10^{-10}		3.3×10^{-10}
Ge-78	1.45 h	1.000	1.2×10^{-9}	1.000	7.0×10^{-10}		3.6×10^{-10}		2.2×10^{-10}		1.5×10^{-10}		1.2×10^{-10}
Arsenic													
As-69	0.253 h	1.000	6.6×10^{-10}	0.500	3.7×10^{-10}		1.8×10^{-10}		1.1×10^{-10}		7.2×10^{-11}		5.7×10^{-11}
As-70	0.876 h	1.000	1.2×10^{-9}	0.500	7.8×10^{-10}		4.1×10^{-10}		2.5×10^{-10}		1.7×10^{-10}		1.3×10^{-10}
As-71	2.70 d	1.000	2.8×10^{-9}	0.500	2.8×10^{-9}		1.5×10^{-9}		9.3×10^{-10}		5.7×10^{-10}		4.6×10^{-10}
As-72	1.08 d	1.000	1.1×10^{-8}	0.500	1.2×10^{-8}		6.3×10^{-9}		3.8×10^{-9}		2.3×10^{-9}		1.8×10^{-9}
As-73	80.3 d	1.000	2.6×10^{-9}	0.500	1.9×10^{-9}		9.3×10^{-10}		5.6×10^{-10}		3.2×10^{-10}		2.6×10^{-10}
As-74	17.8 d	1.000	1.0×10^{-8}	0.500	8.2×10^{-9}		4.3×10^{-9}		2.6×10^{-9}		1.6×10^{-9}		1.3×10^{-9}
As-76	110 d	1.000	1.0×10^{-8}	0.500	1.1×10^{-8}		5.8×10^{-9}		3.4×10^{-9}		2.0×10^{-9}		1.6×10^{-9}
As-77	1.62 d	1.000	2.7×10^{-9}	0.500	2.9×10^{-9}		1.5×10^{-9}		8.7×10^{-10}		5.0×10^{-10}		4.0×10^{-10}
As-78	1.51 h	1.000	2.0×10^{-9}	0.500	1.4×10^{-9}		7.0×10^{-10}		4.1×10^{-10}		2.7×10^{-10}		2.1×10^{-10}
Selenium													
Se-70	0.683 h	1.000	1.0×10^{-9}	0.800	7.1×10^{-10}		3.6×10^{-10}		2.2×10^{-10}		1.5×10^{-10}		1.2×10^{-10}
Se-73	7.15 h	1.000	1.6×10^{-9}	0.800	1.4×10^{-9}		7.4×10^{-10}		4.8×10^{-10}		2.5×10^{-10}		2.1×10^{-10}
Se-73m	0.650 h	1.000	2.6×10^{-10}	0.800	1.8×10^{-10}		9.5×10^{-11}		5.9×10^{-11}		3.5×10^{-11}		2.8×10^{-11}
Se-75	120 d	1.000	2.0×10^{-8}	0.800	1.3×10^{-8}		8.3×10^{-9}		6.0×10^{-9}		3.1×10^{-9}		2.6×10^{-9}
Se-79	6.50×10^4 a	1.000	4.1×10^{-8}	0.800	2.8×10^{-8}		1.9×10^{-8}		1.4×10^{-8}		4.1×10^{-9}		2.9×10^{-9}
Se-81	0.308 h	1.000	3.4×10^{-10}	0.800	1.9×10^{-10}		9.0×10^{-11}		5.1×10^{-11}		3.4×10^{-11}		2.7×10^{-11}

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		$e(g)$
		f_1	$e(g)$		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	
Se-81m	0.954 h	1.000	6.0×10^{-10}	0.800	3.7×10^{-10}	1.8×10^{-10}	1.1×10^{-10}	6.7×10^{-11}	5.3×10^{-11}				
Se-83	0.375 h	1.000	4.6×10^{-10}	0.800	2.9×10^{-10}	1.5×10^{-10}	8.7×10^{-11}	5.9×10^{-11}	4.7×10^{-11}				
Bromine													
Br-74	0.422 h	1.000	9.0×10^{-10}	1.000	5.2×10^{-10}	2.6×10^{-10}	1.5×10^{-10}	1.1×10^{-10}	8.4×10^{-11}				
Br-74m	0.691 h	1.000	1.5×10^{-9}	1.000	8.5×10^{-10}	4.3×10^{-10}	2.5×10^{-10}	1.7×10^{-10}	1.4×10^{-10}				
Br-75	1.63 h	1.000	8.5×10^{-10}	1.000	4.9×10^{-10}	2.5×10^{-10}	1.5×10^{-10}	9.9×10^{-11}	7.9×10^{-11}				
Br-76	16.2 h	1.000	4.2×10^{-9}	1.000	2.7×10^{-9}	1.4×10^{-9}	8.7×10^{-10}	5.6×10^{-10}	4.6×10^{-10}				
Br-77	2.33 d	1.000	6.3×10^{-10}	1.000	4.4×10^{-10}	2.5×10^{-10}	1.7×10^{-10}	1.1×10^{-10}	9.6×10^{-11}				
Br-80	0.290 h	1.000	3.9×10^{-10}	1.000	2.1×10^{-10}	1.0×10^{-10}	5.8×10^{-11}	3.9×10^{-11}	3.1×10^{-11}				
Br-80m	4.42 h	1.000	1.4×10^{-9}	1.000	8.0×10^{-10}	3.9×10^{-10}	2.3×10^{-10}	1.4×10^{-10}	1.1×10^{-10}				
Br-82	1.47 d	1.000	3.7×10^{-9}	1.000	2.6×10^{-9}	1.5×10^{-9}	9.5×10^{-10}	6.4×10^{-10}	5.4×10^{-10}				
Br-83	2.39 h	1.000	5.3×10^{-10}	1.000	3.0×10^{-10}	1.4×10^{-10}	8.3×10^{-11}	5.5×10^{-11}	4.3×10^{-11}				
Br-84	0.530 h	1.000	1.0×10^{-9}	1.000	5.8×10^{-10}	2.8×10^{-10}	1.6×10^{-10}	1.1×10^{-10}	8.8×10^{-11}				
Rubidium													
Rb-79	0.382 h	1.000	5.7×10^{-10}	1.000	3.2×10^{-10}	1.6×10^{-10}	9.2×10^{-11}	6.3×10^{-11}	5.0×10^{-11}				
Rb-81	4.58 h	1.000	5.4×10^{-10}	1.000	3.2×10^{-10}	1.6×10^{-10}	1.0×10^{-10}	6.7×10^{-11}	5.4×10^{-11}				
Rb-81m	0.533 h	1.000	1.1×10^{-10}	1.000	6.2×10^{-11}	3.1×10^{-11}	1.8×10^{-11}	1.2×10^{-11}	9.7×10^{-12}				
Rb-82m	6.20 h	1.000	8.7×10^{-10}	1.000	5.9×10^{-10}	3.4×10^{-10}	2.2×10^{-10}	1.5×10^{-10}	1.3×10^{-10}				
Rb-83	86.2 d	1.000	1.1×10^{-8}	1.000	8.4×10^{-9}	4.9×10^{-9}	3.2×10^{-9}	2.2×10^{-9}	1.9×10^{-9}				
Rb-84	32.8 d	1.000	2.0×10^{-8}	1.000	1.4×10^{-8}	7.9×10^{-9}	5.0×10^{-9}	3.3×10^{-9}	2.8×10^{-9}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)				
Rb-86	18.7 d	1.000	3.1 × 10 ^{−8}	1.000	2.0 × 10 ^{−8}	9.9 × 10 ^{−9}	5.9 × 10 ^{−9}	3.5 × 10 ^{−9}	2.8 × 10 ^{−9}				
Rb-87	4.70 × 10 ¹⁰ a	1.000	1.5 × 10 ^{−8}	1.000	1.0 × 10 ^{−8}	5.2 × 10 ^{−9}	3.1 × 10 ^{−9}	1.8 × 10 ^{−9}	1.5 × 10 ^{−9}				
Rb-88	0.297 h	1.000	1.1 × 10 ^{−9}	1.000	6.2 × 10 ^{−10}	3.0 × 10 ^{−10}	1.7 × 10 ^{−10}	1.2 × 10 ^{−10}	9.0 × 10 ^{−11}				
Rb-89	0.253 h	1.000	5.4 × 10 ^{−10}	1.000	3.0 × 10 ^{−10}	1.5 × 10 ^{−10}	8.6 × 10 ^{−11}	5.9 × 10 ^{−11}	4.7 × 10 ^{−11}				
Strontium ^e													
Sr-80	1.67 h	0.600	3.7 × 10 ^{−9}	0.300	2.3 × 10 ^{−9}	1.1 × 10 ^{−9}	6.5 × 10 ^{−10}	4.2 × 10 ^{−10}	3.4 × 10 ^{−10}				
Sr-81	0.425 h	0.600	8.4 × 10 ^{−10}	0.300	4.9 × 10 ^{−10}	2.4 × 10 ^{−10}	1.4 × 10 ^{−10}	9.6 × 10 ^{−11}	7.7 × 10 ^{−11}				
Sr-82	25.0 d	0.600	7.2 × 10 ^{−8}	0.300	4.1 × 10 ^{−8}	2.1 × 10 ^{−8}	1.3 × 10 ^{−8}	8.7 × 10 ^{−9}	6.1 × 10 ^{−9}				
Sr-83	1.35 d	0.600	3.4 × 10 ^{−9}	0.300	2.7 × 10 ^{−9}	1.4 × 10 ^{−9}	9.1 × 10 ^{−10}	5.7 × 10 ^{−10}	4.9 × 10 ^{−10}				
Sr-85	64.8 d	0.600	7.7 × 10 ^{−9}	0.300	3.1 × 10 ^{−9}	1.7 × 10 ^{−9}	1.5 × 10 ^{−9}	1.3 × 10 ^{−9}	5.6 × 10 ^{−10}				
Sr-85m	1.16 h	0.600	4.5 × 10 ^{−11}	0.300	3.0 × 10 ^{−11}	1.7 × 10 ^{−11}	1.1 × 10 ^{−11}	7.8 × 10 ^{−12}	6.1 × 10 ^{−12}				
Sr-87m	2.80 h	0.600	2.4 × 10 ^{−10}	0.300	1.7 × 10 ^{−10}	9.0 × 10 ^{−11}	5.6 × 10 ^{−11}	3.6 × 10 ^{−11}	3.0 × 10 ^{−11}				
Sr-89	50.5 d	0.600	3.6 × 10 ^{−8}	0.300	1.8 × 10 ^{−8}	8.9 × 10 ^{−9}	5.8 × 10 ^{−9}	4.0 × 10 ^{−9}	2.6 × 10 ^{−9}				
Sr-90	29.1 a	0.600	2.3 × 10 ^{−7}	0.300	7.3 × 10 ^{−8}	4.7 × 10 ^{−8}	6.0 × 10 ^{−8}	8.0 × 10 ^{−8}	2.8 × 10 ^{−8}				
Sr-91	9.50 h	0.600	5.2 × 10 ^{−9}	0.300	4.0 × 10 ^{−9}	2.1 × 10 ^{−9}	1.2 × 10 ^{−9}	7.4 × 10 ^{−10}	6.5 × 10 ^{−10}				
Sr-92	2.71 h	0.600	3.4 × 10 ^{−9}	0.300	2.7 × 10 ^{−9}	1.4 × 10 ^{−9}	8.2 × 10 ^{−10}	4.8 × 10 ^{−10}	4.3 × 10 ^{−10}				
Yttrium													
Y-86	14.7 h	0.001	7.6 × 10 ^{−9}	1.0 × 10 ^{−4}	5.2 × 10 ^{−9}	2.9 × 10 ^{−9}	1.9 × 10 ^{−9}	1.2 × 10 ^{−9}	9.6 × 10 ^{−10}				
Y-86m	0.800 h	0.001	4.5 × 10 ^{−10}	1.0 × 10 ^{−4}	3.1 × 10 ^{−10}	1.7 × 10 ^{−10}	1.1 × 10 ^{−10}	7.1 × 10 ^{−11}	5.6 × 10 ^{−11}				
Y-87	3.35 d	0.001	4.6 × 10 ^{−9}	1.0 × 10 ^{−4}	3.2 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	7.0 × 10 ^{−10}	5.5 × 10 ^{−10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)				
Y-88	107 d	0.001	8.1 × 10 ^{−9}	1.0 × 10 ^{−4}	6.0 × 10 ^{−9}	3.5 × 10 ^{−9}	2.4 × 10 ^{−9}	1.6 × 10 ^{−9}	1.3 × 10 ^{−9}				
Y-90	2.67 d	0.001	3.1 × 10 ^{−8}	1.0 × 10 ^{−4}	2.0 × 10 ^{−8}	1.0 × 10 ^{−8}	5.9 × 10 ^{−9}	3.3 × 10 ^{−9}	2.7 × 10 ^{−9}				
Y-90m	3.19 h	0.001	1.8 × 10 ^{−9}	1.0 × 10 ^{−4}	1.2 × 10 ^{−9}	6.1 × 10 ^{−10}	3.7 × 10 ^{−10}	2.2 × 10 ^{−10}	1.7 × 10 ^{−10}				
Y-91	58.5 d	0.001	2.8 × 10 ^{−8}	1.0 × 10 ^{−4}	1.8 × 10 ^{−8}	8.8 × 10 ^{−9}	5.2 × 10 ^{−9}	2.9 × 10 ^{−9}	2.4 × 10 ^{−9}				
Y-91m	0.828 h	0.001	9.2 × 10 ^{−11}	1.0 × 10 ^{−4}	6.0 × 10 ^{−11}	3.3 × 10 ^{−11}	2.1 × 10 ^{−11}	1.4 × 10 ^{−11}	1.1 × 10 ^{−11}				
Y-92	3.54 h	0.001	5.9 × 10 ^{−9}	1.0 × 10 ^{−4}	3.6 × 10 ^{−9}	1.8 × 10 ^{−9}	1.0 × 10 ^{−9}	6.2 × 10 ^{−10}	4.9 × 10 ^{−10}				
Y-93	10.1 h	0.001	1.4 × 10 ^{−8}	1.0 × 10 ^{−4}	8.5 × 10 ^{−9}	4.3 × 10 ^{−9}	2.5 × 10 ^{−9}	1.4 × 10 ^{−9}	1.2 × 10 ^{−9}				
Y-94	0.318 h	0.001	9.9 × 10 ^{−10}	1.0 × 10 ^{−4}	5.5 × 10 ^{−10}	2.7 × 10 ^{−10}	1.5 × 10 ^{−10}	1.0 × 10 ^{−10}	8.1 × 10 ^{−11}				
Y-95	0.178 h	0.001	5.7 × 10 ^{−10}	1.0 × 10 ^{−4}	3.1 × 10 ^{−10}	1.5 × 10 ^{−10}	8.7 × 10 ^{−11}	5.9 × 10 ^{−11}	4.6 × 10 ^{−11}				
Zirconium													
Zr-86	16.5 h	0.020	6.9 × 10 ^{−9}	0.010	4.8 × 10 ^{−9}	2.7 × 10 ^{−9}	1.7 × 10 ^{−9}	1.1 × 10 ^{−9}	8.6 × 10 ^{−10}				
Zr-88	83.4 d	0.020	2.8 × 10 ^{−9}	0.010	2.0 × 10 ^{−9}	1.2 × 10 ^{−9}	8.0 × 10 ^{−10}	5.4 × 10 ^{−10}	4.5 × 10 ^{−10}				
Zr-89	3.27 d	0.020	6.5 × 10 ^{−9}	0.010	4.5 × 10 ^{−9}	2.5 × 10 ^{−9}	1.6 × 10 ^{−9}	9.9 × 10 ^{−10}	7.9 × 10 ^{−10}				
Zr-93	1.53 × 10 ⁶ a	0.020	1.2 × 10 ^{−9}	0.010	7.6 × 10 ^{−10}	5.1 × 10 ^{−10}	5.8 × 10 ^{−10}	8.6 × 10 ^{−10}	1.1 × 10 ^{−9}				
Zr-95	64.0 d	0.020	8.5 × 10 ^{−9}	0.010	5.6 × 10 ^{−9}	3.0 × 10 ^{−9}	1.9 × 10 ^{−9}	1.2 × 10 ^{−9}	9.5 × 10 ^{−10}				
Zr-97	16.9 h	0.020	2.2 × 10 ^{−8}	0.010	1.4 × 10 ^{−8}	7.3 × 10 ^{−9}	4.4 × 10 ^{−9}	2.6 × 10 ^{−9}	2.1 × 10 ^{−9}				
Niobium													
Nb-88	0.238 h	0.020	6.7 × 10 ^{−10}	0.010	3.8 × 10 ^{−10}	1.9 × 10 ^{−10}	1.1 × 10 ^{−10}	7.9 × 10 ^{−11}	6.3 × 10 ^{−11}				
Nb-89	2.03 h	0.020	3.0 × 10 ^{−9}	0.010	2.0 × 10 ^{−9}	1.0 × 10 ^{−9}	6.0 × 10 ^{−10}	3.4 × 10 ^{−10}	2.7 × 10 ^{−10}				
Nb-89m	1.10 h	0.020	1.5 × 10 ^{−9}	0.010	8.7 × 10 ^{−10}	4.4 × 10 ^{−10}	2.7 × 10 ^{−10}	1.8 × 10 ^{−10}	1.4 × 10 ^{−10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Nb-90	14.6 h	0.020	1.1 × 10 ^{−8}	0.010	7.2 × 10 ^{−9}	3.9 × 10 ^{−9}	2.5 × 10 ^{−9}	1.6 × 10 ^{−9}	1.2 × 10 ^{−9}	1.2 × 10 ^{−9}	1.2 × 10 ^{−9}	1.2 × 10 ^{−9}	
Nb-93m	13.6 a	0.020	1.5 × 10 ^{−9}	0.010	9.1 × 10 ^{−10}	4.6 × 10 ^{−10}	2.7 × 10 ^{−10}	1.5 × 10 ^{−10}	1.2 × 10 ^{−10}	1.2 × 10 ^{−10}	1.2 × 10 ^{−10}	1.2 × 10 ^{−10}	
Nb-94	2.03 × 10 ⁴ a	0.020	1.5 × 10 ^{−8}	0.010	9.7 × 10 ^{−9}	5.3 × 10 ^{−9}	3.4 × 10 ^{−9}	2.1 × 10 ^{−9}	1.7 × 10 ^{−9}	1.7 × 10 ^{−9}	1.7 × 10 ^{−9}	1.7 × 10 ^{−9}	
Nb-95	35.1 d	0.020	4.6 × 10 ^{−9}	0.010	3.2 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	7.4 × 10 ^{−10}	5.8 × 10 ^{−10}	5.8 × 10 ^{−10}	5.8 × 10 ^{−10}	5.8 × 10 ^{−10}	
Nb-95m	3.61 d	0.020	6.4 × 10 ^{−9}	0.010	4.1 × 10 ^{−9}	2.1 × 10 ^{−9}	1.2 × 10 ^{−9}	7.1 × 10 ^{−10}	5.6 × 10 ^{−10}	5.6 × 10 ^{−10}	5.6 × 10 ^{−10}	5.6 × 10 ^{−10}	
Nb-96	23.3 h	0.020	9.2 × 10 ^{−9}	0.010	6.3 × 10 ^{−9}	3.4 × 10 ^{−9}	2.2 × 10 ^{−9}	1.4 × 10 ^{−9}	1.1 × 10 ^{−9}	1.1 × 10 ^{−9}	1.1 × 10 ^{−9}	1.1 × 10 ^{−9}	
Nb-97	1.20 h	0.020	7.7 × 10 ^{−10}	0.010	4.5 × 10 ^{−10}	2.3 × 10 ^{−10}	1.3 × 10 ^{−10}	8.7 × 10 ^{−11}	6.8 × 10 ^{−11}	6.8 × 10 ^{−11}	6.8 × 10 ^{−11}	6.8 × 10 ^{−11}	
Nb-98	0.858 h	0.020	1.2 × 10 ^{−9}	0.010	7.1 × 10 ^{−10}	3.6 × 10 ^{−10}	2.2 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}	1.1 × 10 ^{−10}	1.1 × 10 ^{−10}	1.1 × 10 ^{−10}	
Molybdenum													
Mo-90	5.67 h	1.000	1.7 × 10 ^{−9}	1.000	1.2 × 10 ^{−9}	6.3 × 10 ^{−10}	4.0 × 10 ^{−10}	2.7 × 10 ^{−10}	2.2 × 10 ^{−10}	2.2 × 10 ^{−10}	2.2 × 10 ^{−10}	2.2 × 10 ^{−10}	
Mo-93	3.50 × 10 ³ a	1.000	7.9 × 10 ^{−9}	1.000	6.9 × 10 ^{−9}	5.0 × 10 ^{−9}	4.0 × 10 ^{−9}	3.4 × 10 ^{−9}	3.1 × 10 ^{−9}	3.1 × 10 ^{−9}	3.1 × 10 ^{−9}	3.1 × 10 ^{−9}	
Mo-93m	6.85 h	1.000	8.0 × 10 ^{−10}	1.000	5.4 × 10 ^{−10}	3.1 × 10 ^{−10}	2.0 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}	1.1 × 10 ^{−10}	1.1 × 10 ^{−10}	1.1 × 10 ^{−10}	
Mo-99	2.75 d	1.000	5.5 × 10 ^{−9}	1.000	3.5 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	7.6 × 10 ^{−10}	6.0 × 10 ^{−10}	6.0 × 10 ^{−10}	6.0 × 10 ^{−10}	6.0 × 10 ^{−10}	
Mo-101	0.244 h	1.000	4.8 × 10 ^{−10}	1.000	2.7 × 10 ^{−10}	1.3 × 10 ^{−10}	7.6 × 10 ^{−11}	5.2 × 10 ^{−11}	4.1 × 10 ^{−11}	4.1 × 10 ^{−11}	4.1 × 10 ^{−11}	4.1 × 10 ^{−11}	
Technetium													
Tc-93	2.75 h	1.000	2.7 × 10 ^{−10}	0.500	2.5 × 10 ^{−10}	1.5 × 10 ^{−10}	9.8 × 10 ^{−11}	6.8 × 10 ^{−11}	5.5 × 10 ^{−11}	5.5 × 10 ^{−11}	5.5 × 10 ^{−11}	5.5 × 10 ^{−11}	
Tc-93m	0.725 h	1.000	2.0 × 10 ^{−10}	0.500	1.3 × 10 ^{−10}	7.3 × 10 ^{−11}	4.6 × 10 ^{−11}	3.2 × 10 ^{−11}	2.5 × 10 ^{−11}	2.5 × 10 ^{−11}	2.5 × 10 ^{−11}	2.5 × 10 ^{−11}	
Tc-94	4.88 h	1.000	1.2 × 10 ^{−9}	0.500	1.0 × 10 ^{−9}	5.8 × 10 ^{−10}	3.7 × 10 ^{−10}	2.5 × 10 ^{−10}	2.0 × 10 ^{−10}	2.0 × 10 ^{−10}	2.0 × 10 ^{−10}	2.0 × 10 ^{−10}	
Tc-94m	0.867 h	1.000	1.3 × 10 ^{−9}	0.500	6.5 × 10 ^{−10}	3.3 × 10 ^{−10}	1.9 × 10 ^{−10}	1.3 × 10 ^{−10}	1.0 × 10 ^{−10}	1.0 × 10 ^{−10}	1.0 × 10 ^{−10}	1.0 × 10 ^{−10}	
Tc-95	20.0 h	1.000	9.9 × 10 ^{−10}	0.500	8.7 × 10 ^{−10}	5.0 × 10 ^{−10}	3.3 × 10 ^{−10}	2.3 × 10 ^{−10}	1.8 × 10 ^{−10}	1.8 × 10 ^{−10}	1.8 × 10 ^{−10}	1.8 × 10 ^{−10}	

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Tc-95m	61.0 d	1.000	4.7 × 10 ^{−9}	0.500	2.8 × 10 ^{−9}	1.6 × 10 ^{−9}	1.0 × 10 ^{−9}	7.0 × 10 ^{−10}	5.6 × 10 ^{−10}				
Tc-96	4.28 d	1.000	6.7 × 10 ^{−9}	0.500	5.1 × 10 ^{−9}	3.0 × 10 ^{−9}	2.0 × 10 ^{−9}	1.4 × 10 ^{−9}	1.1 × 10 ^{−9}				
Tc-96m	0.858 h	1.000	1.0 × 10 ^{−10}	0.500	6.5 × 10 ^{−11}	3.6 × 10 ^{−11}	2.3 × 10 ^{−11}	1.6 × 10 ^{−11}	1.2 × 10 ^{−11}				
Tc-97	2.60 × 10 ⁶ a	1.000	9.9 × 10 ^{−10}	0.500	4.9 × 10 ^{−10}	2.4 × 10 ^{−10}	1.4 × 10 ^{−10}	8.8 × 10 ^{−11}	6.8 × 10 ^{−11}				
Tc-97m	87.0 d	1.000	8.7 × 10 ^{−9}	0.500	4.1 × 10 ^{−9}	2.0 × 10 ^{−9}	1.1 × 10 ^{−9}	7.0 × 10 ^{−10}	5.5 × 10 ^{−10}				
Tc-98	4.20 × 10 ⁶ a	1.000	2.3 × 10 ^{−8}	0.500	1.2 × 10 ^{−8}	6.1 × 10 ^{−9}	3.7 × 10 ^{−9}	2.5 × 10 ^{−9}	2.0 × 10 ^{−9}				
Tc-99	2.13 × 10 ⁵ a	1.000	1.0 × 10 ^{−8}	0.500	4.8 × 10 ^{−9}	2.3 × 10 ^{−9}	1.3 × 10 ^{−9}	8.2 × 10 ^{−10}	6.4 × 10 ^{−10}				
Tc-99m	6.02 h	1.000	2.0 × 10 ^{−10}	0.500	1.3 × 10 ^{−10}	7.2 × 10 ^{−11}	4.3 × 10 ^{−11}	2.8 × 10 ^{−11}	2.2 × 10 ^{−11}				
Tc-101	0.237 h	1.000	2.4 × 10 ^{−10}	0.500	1.3 × 10 ^{−10}	6.1 × 10 ^{−11}	3.5 × 10 ^{−11}	2.4 × 10 ^{−11}	1.9 × 10 ^{−11}				
Tc-104	0.303 h	1.000	1.0 × 10 ^{−9}	0.500	5.3 × 10 ^{−10}	2.6 × 10 ^{−10}	1.5 × 10 ^{−10}	1.0 × 10 ^{−10}	8.0 × 10 ^{−11}				
Ruthenium													
Ru-94	0.863 h	0.100	9.3 × 10 ^{−10}	0.050	5.9 × 10 ^{−10}	3.1 × 10 ^{−10}	1.9 × 10 ^{−10}	1.2 × 10 ^{−10}	9.4 × 10 ^{−11}				
Ru-97	2.90 d	0.100	1.2 × 10 ^{−9}	0.050	8.5 × 10 ^{−10}	4.7 × 10 ^{−10}	3.0 × 10 ^{−10}	1.9 × 10 ^{−10}	1.5 × 10 ^{−10}				
Ru-103	39.3 d	0.100	7.1 × 10 ^{−9}	0.050	4.6 × 10 ^{−9}	2.4 × 10 ^{−9}	1.5 × 10 ^{−9}	9.2 × 10 ^{−10}	7.3 × 10 ^{−10}				
Ru-105	4.44 h	0.100	2.7 × 10 ^{−9}	0.050	1.8 × 10 ^{−9}	9.1 × 10 ^{−10}	5.5 × 10 ^{−10}	3.3 × 10 ^{−10}	2.6 × 10 ^{−10}				
Ru-106	1.01 a	0.100	8.4 × 10 ^{−8}	0.050	4.9 × 10 ^{−8}	2.5 × 10 ^{−8}	1.5 × 10 ^{−8}	8.6 × 10 ^{−9}	7.0 × 10 ^{−9}				
Rhodium													
Rh-99	16.0 d	0.100	4.2 × 10 ^{−9}	0.050	2.9 × 10 ^{−9}	1.6 × 10 ^{−9}	1.0 × 10 ^{−9}	6.5 × 10 ^{−10}	5.1 × 10 ^{−10}				
Rh-99m	4.70 h	0.100	4.9 × 10 ^{−10}	0.050	3.5 × 10 ^{−10}	2.0 × 10 ^{−10}	1.3 × 10 ^{−10}	8.3 × 10 ^{−11}	6.6 × 10 ^{−11}				
Rh-100	20.8 h	0.100	4.9 × 10 ^{−9}	0.050	3.6 × 10 ^{−9}	2.0 × 10 ^{−9}	1.4 × 10 ^{−9}	8.8 × 10 ^{−10}	7.1 × 10 ^{−10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Rh-101	3.20 a	0.100	4.9 × 10 ^{−9}	0.050	2.8 × 10 ^{−9}	1.6 × 10 ^{−9}	1.0 × 10 ^{−9}	6.7 × 10 ^{−10}	5.5 × 10 ^{−10}				
Rh-101m	4.34 d	0.100	1.7 × 10 ^{−9}	0.050	1.2 × 10 ^{−9}	6.8 × 10 ^{−10}	4.4 × 10 ^{−10}	2.8 × 10 ^{−10}	2.2 × 10 ^{−10}				
Rh-102	2.90 a	0.100	1.9 × 10 ^{−8}	0.050	1.0 × 10 ^{−8}	6.4 × 10 ^{−9}	4.3 × 10 ^{−9}	3.0 × 10 ^{−9}	2.6 × 10 ^{−9}				
Rh-102m	207 d	0.100	1.2 × 10 ^{−8}	0.050	7.4 × 10 ^{−9}	3.9 × 10 ^{−9}	2.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.2 × 10 ^{−9}				
Rh-103m	0.935 h	0.100	4.7 × 10 ^{−11}	0.050	2.7 × 10 ^{−11}	1.3 × 10 ^{−11}	7.4 × 10 ^{−12}	4.8 × 10 ^{−12}	3.8 × 10 ^{−12}				
Rh-105	1.47 d	0.100	4.0 × 10 ^{−9}	0.050	2.7 × 10 ^{−9}	1.3 × 10 ^{−9}	8.0 × 10 ^{−10}	4.6 × 10 ^{−10}	3.7 × 10 ^{−10}				
Rh-106m	2.20 h	0.100	1.4 × 10 ^{−9}	0.050	9.7 × 10 ^{−10}	5.3 × 10 ^{−10}	3.3 × 10 ^{−10}	2.0 × 10 ^{−10}	1.6 × 10 ^{−10}				
Rh-107	0.362 h	0.100	2.9 × 10 ^{−10}	0.050	1.6 × 10 ^{−10}	7.9 × 10 ^{−11}	4.5 × 10 ^{−11}	3.1 × 10 ^{−11}	2.4 × 10 ^{−11}				
Palladium													
Pd-100	3.63 d	0.050	7.4 × 10 ^{−9}	0.005	5.2 × 10 ^{−9}	2.9 × 10 ^{−9}	1.9 × 10 ^{−9}	1.2 × 10 ^{−9}	9.4 × 10 ^{−10}				
Pd-101	8.27 h	0.050	8.2 × 10 ^{−10}	0.005	5.7 × 10 ^{−10}	3.1 × 10 ^{−10}	1.9 × 10 ^{−10}	1.2 × 10 ^{−10}	9.4 × 10 ^{−11}				
Pd-103	17.0 d	0.050	2.2 × 10 ^{−9}	0.005	1.4 × 10 ^{−9}	7.2 × 10 ^{−10}	4.3 × 10 ^{−10}	2.4 × 10 ^{−10}	1.9 × 10 ^{−10}				
Pd-107	6.50 × 10 ⁶ a	0.050	4.4 × 10 ^{−10}	0.005	2.8 × 10 ^{−10}	1.4 × 10 ^{−10}	8.1 × 10 ^{−11}	4.6 × 10 ^{−11}	3.7 × 10 ^{−11}				
Pd-109	13.4 h	0.050	6.3 × 10 ^{−9}	0.005	4.1 × 10 ^{−9}	2.0 × 10 ^{−9}	1.2 × 10 ^{−9}	6.8 × 10 ^{−10}	5.5 × 10 ^{−10}				
Silver													
Ag-102	0.215 h	0.100	4.2 × 10 ^{−10}	0.050	2.4 × 10 ^{−10}	1.2 × 10 ^{−10}	7.3 × 10 ^{−11}	5.0 × 10 ^{−11}	4.0 × 10 ^{−11}				
Ag-103	1.09 h	0.100	4.5 × 10 ^{−10}	0.050	2.7 × 10 ^{−10}	1.4 × 10 ^{−10}	8.3 × 10 ^{−11}	5.5 × 10 ^{−11}	4.3 × 10 ^{−11}				
Ag-104	1.15 h	0.100	4.3 × 10 ^{−10}	0.050	2.9 × 10 ^{−10}	1.7 × 10 ^{−10}	1.1 × 10 ^{−10}	7.5 × 10 ^{−11}	6.0 × 10 ^{−11}				
Ag-104m	0.558 h	0.100	5.6 × 10 ^{−10}	0.050	3.3 × 10 ^{−10}	1.7 × 10 ^{−10}	1.0 × 10 ^{−10}	6.8 × 10 ^{−11}	5.4 × 10 ^{−11}				
Ag-105	41.0 d	0.100	3.9 × 10 ^{−9}	0.050	2.5 × 10 ^{−9}	1.4 × 10 ^{−9}	9.1 × 10 ^{−10}	5.9 × 10 ^{−10}	4.7 × 10 ^{−10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Ag-106	0.399 h	0.100	3.7 × 10 ^{−10}	0.050	2.1 × 10 ^{−10}	1.0 × 10 ^{−10}	6.0 × 10 ^{−11}	4.1 × 10 ^{−11}	3.2 × 10 ^{−11}				
Ag-106m	8.41 d	0.100	9.7 × 10 ^{−9}	0.050	6.9 × 10 ^{−9}	4.1 × 10 ^{−9}	2.8 × 10 ^{−9}	1.8 × 10 ^{−9}	1.5 × 10 ^{−9}				
Ag-108m	1.27 × 10 ² a	0.100	2.1 × 10 ^{−8}	0.050	1.1 × 10 ^{−8}	6.5 × 10 ^{−9}	4.3 × 10 ^{−9}	2.8 × 10 ^{−9}	2.3 × 10 ^{−9}				
Ag-110m	250 d	0.100	2.4 × 10 ^{−8}	0.050	1.4 × 10 ^{−8}	7.8 × 10 ^{−9}	5.2 × 10 ^{−9}	3.4 × 10 ^{−9}	2.8 × 10 ^{−9}				
Ag-111	7.45 d	0.100	1.4 × 10 ^{−8}	0.050	9.3 × 10 ^{−9}	4.6 × 10 ^{−9}	2.7 × 10 ^{−9}	1.6 × 10 ^{−9}	1.3 × 10 ^{−9}				
Ag-112	3.12 h	0.100	4.9 × 10 ^{−9}	0.050	3.0 × 10 ^{−9}	1.5 × 10 ^{−9}	8.9 × 10 ^{−10}	5.4 × 10 ^{−10}	4.3 × 10 ^{−10}				
Ag-115	0.333 h	0.100	7.2 × 10 ^{−10}	0.050	4.1 × 10 ^{−10}	2.0 × 10 ^{−10}	1.2 × 10 ^{−10}	7.7 × 10 ^{−11}	6.0 × 10 ^{−11}				
Cadmium													
Cd-104	0.961 h	0.100	4.2 × 10 ^{−10}	0.050	2.9 × 10 ^{−10}	1.7 × 10 ^{−10}	1.1 × 10 ^{−10}	7.2 × 10 ^{−11}	5.4 × 10 ^{−11}				
Cd-107	6.49 h	0.100	7.1 × 10 ^{−10}	0.050	4.6 × 10 ^{−10}	2.3 × 10 ^{−10}	1.3 × 10 ^{−10}	7.8 × 10 ^{−11}	6.2 × 10 ^{−11}				
Cd-109	1.27 a	0.100	2.1 × 10 ^{−8}	0.050	9.5 × 10 ^{−9}	5.5 × 10 ^{−9}	3.5 × 10 ^{−9}	2.4 × 10 ^{−9}	2.0 × 10 ^{−9}				
Cd-113	9.30 × 10 ¹⁵ a	0.100	1.0 × 10 ^{−7}	0.050	4.8 × 10 ^{−8}	3.7 × 10 ^{−8}	3.0 × 10 ^{−8}	2.6 × 10 ^{−8}	2.5 × 10 ^{−8}				
Cd-113m	13.6 a	0.100	1.2 × 10 ^{−7}	0.050	5.6 × 10 ^{−8}	3.9 × 10 ^{−8}	2.9 × 10 ^{−8}	2.4 × 10 ^{−8}	2.3 × 10 ^{−8}				
Cd-115	2.23 d	0.100	1.4 × 10 ^{−8}	0.050	9.7 × 10 ^{−9}	4.9 × 10 ^{−9}	2.9 × 10 ^{−9}	1.7 × 10 ^{−9}	1.4 × 10 ^{−9}				
Cd-115m	44.6 d	0.100	4.1 × 10 ^{−8}	0.050	1.9 × 10 ^{−8}	9.7 × 10 ^{−9}	6.9 × 10 ^{−9}	4.1 × 10 ^{−9}	3.3 × 10 ^{−9}				
Cd-117	2.49 h	0.100	2.9 × 10 ^{−9}	0.050	1.9 × 10 ^{−9}	9.5 × 10 ^{−10}	5.7 × 10 ^{−10}	3.5 × 10 ^{−10}	2.8 × 10 ^{−10}				
Cd-117m	3.36 h	0.100	2.6 × 10 ^{−9}	0.050	1.7 × 10 ^{−9}	9.0 × 10 ^{−10}	5.6 × 10 ^{−10}	3.5 × 10 ^{−10}	2.8 × 10 ^{−10}				
Indium													
In-109	4.20 h	0.040	5.2 × 10 ^{−10}	0.020	3.6 × 10 ^{−10}	2.0 × 10 ^{−10}	1.3 × 10 ^{−10}	8.2 × 10 ^{−11}	6.6 × 10 ^{−11}				
In-110	4.90 h	0.040	1.5 × 10 ^{−9}	0.020	1.1 × 10 ^{−9}	6.5 × 10 ^{−10}	4.4 × 10 ^{−10}	3.0 × 10 ^{−10}	2.4 × 10 ^{−10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		f_1	$e(g)$		$e(g)$		$e(g)$		$e(g)$		$e(g)$		$e(g)$	
In-110m	1.15 h	0.040	1.1×10^{-9}	0.020	6.4×10^{-10}		3.2×10^{-10}		1.9×10^{-10}		1.3×10^{-10}		1.0×10^{-10}	
In-111	2.83 d	0.040	2.4×10^{-9}	0.020	1.7×10^{-9}		9.1×10^{-10}		5.9×10^{-10}		3.7×10^{-10}		2.9×10^{-10}	
In-112	0.240 h	0.040	1.2×10^{-10}	0.020	6.7×10^{-11}		3.3×10^{-11}		1.9×10^{-11}		1.3×10^{-11}		1.0×10^{-11}	
In-113m	1.66 h	0.040	3.0×10^{-10}	0.020	1.8×10^{-10}		9.3×10^{-11}		6.2×10^{-11}		3.6×10^{-11}		2.8×10^{-11}	
In-114m	49.5 d	0.040	5.6×10^{-8}	0.020	3.1×10^{-8}		1.5×10^{-8}		9.0×10^{-9}		5.2×10^{-9}		4.1×10^{-9}	
In-115	5.10×10^{15} a	0.040	1.3×10^{-7}	0.020	6.4×10^{-8}		4.8×10^{-8}		4.3×10^{-8}		3.6×10^{-8}		3.2×10^{-8}	
In-115m	4.49 h	0.040	9.6×10^{-10}	0.020	6.0×10^{-10}		3.0×10^{-10}		1.8×10^{-10}		1.1×10^{-10}		8.6×10^{-11}	
In-116m	0.902 h	0.040	5.8×10^{-10}	0.020	3.6×10^{-10}		1.9×10^{-10}		1.2×10^{-10}		8.0×10^{-11}		6.4×10^{-11}	
In-117	0.730 h	0.040	3.3×10^{-10}	0.020	1.9×10^{-10}		9.7×10^{-11}		5.8×10^{-11}		3.9×10^{-11}		3.1×10^{-11}	
In-117m	1.94 h	0.040	1.4×10^{-9}	0.020	8.6×10^{-10}		4.3×10^{-10}		2.5×10^{-10}		1.6×10^{-10}		1.2×10^{-10}	
In-119m	0.300 h	0.040	5.9×10^{-10}	0.020	3.2×10^{-10}		1.6×10^{-10}		8.8×10^{-11}		6.0×10^{-11}		4.7×10^{-11}	
Tin														
Sn-110	4.00 h	0.040	3.5×10^{-9}	0.020	2.3×10^{-9}		1.2×10^{-9}		7.4×10^{-10}		4.4×10^{-10}		3.5×10^{-10}	
Sn-111	0.588 h	0.040	2.5×10^{-10}	0.020	1.5×10^{-10}		7.4×10^{-11}		4.4×10^{-11}		3.0×10^{-11}		2.3×10^{-11}	
Sn-113	115 d	0.040	7.8×10^{-9}	0.020	5.0×10^{-9}		2.6×10^{-9}		1.6×10^{-9}		9.2×10^{-10}		7.3×10^{-10}	
Sn-117m	13.6 d	0.040	7.7×10^{-9}	0.020	5.0×10^{-9}		2.5×10^{-9}		1.5×10^{-9}		8.8×10^{-10}		7.1×10^{-10}	
Sn-119m	293 d	0.040	4.1×10^{-9}	0.020	2.5×10^{-9}		1.3×10^{-9}		7.5×10^{-10}		4.3×10^{-10}		3.4×10^{-10}	
Sn-121	1.13 d	0.040	2.6×10^{-9}	0.020	1.7×10^{-9}		8.4×10^{-10}		5.0×10^{-10}		2.8×10^{-10}		2.3×10^{-10}	
Sn-121m	55.0 a	0.040	4.6×10^{-9}	0.020	2.7×10^{-9}		1.4×10^{-9}		8.2×10^{-10}		4.7×10^{-10}		3.8×10^{-10}	
Sn-123	129 d	0.040	2.5×10^{-8}	0.020	1.6×10^{-8}		7.8×10^{-9}		4.6×10^{-9}		2.6×10^{-9}		2.1×10^{-9}	
Sn-123m	0.668 h	0.040	4.7×10^{-10}	0.020	2.6×10^{-10}		1.3×10^{-10}		7.3×10^{-11}		4.9×10^{-11}		3.8×10^{-11}	

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Sn-125	9.64 d	0.040	3.5 × 10 ^{−8}	0.020	2.2 × 10 ^{−8}	1.1 × 10 ^{−8}	6.7 × 10 ^{−9}	3.8 × 10 ^{−9}	3.1 × 10 ^{−9}				
Sn-126	1.00 × 10 ⁵ a	0.040	5.0 × 10 ^{−8}	0.020	3.0 × 10 ^{−8}	1.6 × 10 ^{−8}	9.8 × 10 ^{−9}	5.9 × 10 ^{−9}	4.7 × 10 ^{−9}				
Sn-127	2.10 h	0.040	2.0 × 10 ^{−9}	0.020	1.3 × 10 ^{−9}	6.6 × 10 ^{−10}	4.0 × 10 ^{−10}	2.5 × 10 ^{−10}	2.0 × 10 ^{−10}				
Sn-128	0.985 h	0.040	1.6 × 10 ^{−9}	0.020	9.7 × 10 ^{−10}	4.9 × 10 ^{−10}	3.0 × 10 ^{−10}	1.9 × 10 ^{−10}	1.5 × 10 ^{−10}				
Antimony													
Sb-115	0.530 h	0.200	2.5 × 10 ^{−10}	0.100	1.5 × 10 ^{−10}	7.5 × 10 ^{−11}	4.5 × 10 ^{−11}	3.1 × 10 ^{−11}	2.4 × 10 ^{−11}				
Sb-116	0.263 h	0.200	2.7 × 10 ^{−10}	0.100	1.6 × 10 ^{−10}	8.0 × 10 ^{−11}	4.8 × 10 ^{−11}	3.3 × 10 ^{−11}	2.6 × 10 ^{−11}				
Sb-116m	1.00 h	0.200	5.0 × 10 ^{−10}	0.100	3.3 × 10 ^{−10}	1.9 × 10 ^{−10}	1.2 × 10 ^{−10}	8.3 × 10 ^{−11}	6.7 × 10 ^{−11}				
Sb-117	2.80 h	0.200	1.6 × 10 ^{−10}	0.100	1.0 × 10 ^{−10}	5.6 × 10 ^{−11}	3.5 × 10 ^{−11}	2.2 × 10 ^{−11}	1.8 × 10 ^{−11}				
Sb-118m	5.00 h	0.200	1.3 × 10 ^{−9}	0.100	1.0 × 10 ^{−9}	5.8 × 10 ^{−10}	3.9 × 10 ^{−10}	2.6 × 10 ^{−10}	2.1 × 10 ^{−10}				
Sb-119	1.59 d	0.200	8.4 × 10 ^{−10}	0.100	5.8 × 10 ^{−10}	3.0 × 10 ^{−10}	1.8 × 10 ^{−10}	1.0 × 10 ^{−10}	8.0 × 10 ^{−11}				
Sb-120	0.265 h	0.200	1.7 × 10 ^{−10}	0.100	9.4 × 10 ^{−11}	4.6 × 10 ^{−11}	2.7 × 10 ^{−11}	1.8 × 10 ^{−11}	1.4 × 10 ^{−11}				
Sb-120m	5.76 d	0.200	8.1 × 10 ^{−9}	0.100	6.0 × 10 ^{−9}	3.5 × 10 ^{−9}	2.3 × 10 ^{−9}	1.6 × 10 ^{−9}	1.2 × 10 ^{−9}				
Sb-122	2.70 d	0.200	1.8 × 10 ^{−8}	0.100	1.2 × 10 ^{−8}	6.1 × 10 ^{−9}	3.7 × 10 ^{−9}	2.1 × 10 ^{−9}	1.7 × 10 ^{−9}				
Sb-124	60.2 d	0.200	2.5 × 10 ^{−8}	0.100	1.6 × 10 ^{−8}	8.4 × 10 ^{−9}	5.2 × 10 ^{−9}	3.2 × 10 ^{−9}	2.5 × 10 ^{−9}				
Sb-124m	0.337 h	0.200	8.5 × 10 ^{−11}	0.100	4.9 × 10 ^{−11}	2.5 × 10 ^{−11}	1.5 × 10 ^{−11}	1.0 × 10 ^{−11}	8.0 × 10 ^{−12}				
Sb-125	2.77 a	0.200	1.1 × 10 ^{−8}	0.100	6.1 × 10 ^{−9}	3.4 × 10 ^{−9}	2.1 × 10 ^{−9}	1.4 × 10 ^{−9}	1.1 × 10 ^{−9}				
Sb-126	12.4 d	0.200	2.0 × 10 ^{−8}	0.100	1.4 × 10 ^{−8}	7.6 × 10 ^{−9}	4.9 × 10 ^{−9}	3.1 × 10 ^{−9}	2.4 × 10 ^{−9}				
Sb-126m	0.317 h	0.200	3.9 × 10 ^{−10}	0.100	2.2 × 10 ^{−10}	1.1 × 10 ^{−10}	6.6 × 10 ^{−11}	4.5 × 10 ^{−11}	3.6 × 10 ^{−11}				
Sb-127	3.85 d	0.200	1.7 × 10 ^{−8}	0.100	1.2 × 10 ^{−8}	5.9 × 10 ^{−9}	3.6 × 10 ^{−9}	2.1 × 10 ^{−9}	1.7 × 10 ^{−9}				
Sb-128	9.01 h	0.200	6.3 × 10 ^{−9}	0.100	4.5 × 10 ^{−9}	2.4 × 10 ^{−9}	1.5 × 10 ^{−9}	9.5 × 10 ^{−10}	7.6 × 10 ^{−10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Tellurium													
Sb-128	0.173 h	0.200	3.7 × 10 ^{−10}	0.100	2.1 × 10 ^{−10}	1.0 × 10 ^{−10}	6.0 × 10 ^{−11}	4.1 × 10 ^{−11}	3.3 × 10 ^{−11}				
Sb-129	4.32 h	0.200	4.3 × 10 ^{−9}	0.100	2.8 × 10 ^{−9}	1.5 × 10 ^{−9}	8.8 × 10 ^{−10}	5.3 × 10 ^{−10}	4.2 × 10 ^{−10}				
Sb-130	0.667 h	0.200	9.1 × 10 ^{−10}	0.100	5.4 × 10 ^{−10}	2.8 × 10 ^{−10}	1.7 × 10 ^{−10}	1.2 × 10 ^{−10}	9.1 × 10 ^{−11}				
Sb-131	0.383 h	0.200	1.1 × 10 ^{−9}	0.100	7.3 × 10 ^{−10}	3.9 × 10 ^{−10}	2.1 × 10 ^{−10}	1.4 × 10 ^{−10}	1.0 × 10 ^{−10}				
Tellurium													
Te-116	2.49 h	0.600	1.4 × 10 ^{−9}	0.300	1.0 × 10 ^{−9}	5.5 × 10 ^{−10}	3.4 × 10 ^{−10}	2.1 × 10 ^{−10}	1.7 × 10 ^{−1}				
Te-121	17.0 d	0.600	3.1 × 10 ^{−9}	0.300	2.0 × 10 ^{−9}	1.2 × 10 ^{−9}	8.0 × 10 ^{−10}	5.4 × 10 ^{−10}	4.3 × 10 ^{−1}				
Te-121m	154 d	0.600	2.7 × 10 ^{−8}	0.300	1.2 × 10 ^{−8}	6.9 × 10 ^{−9}	4.2 × 10 ^{−9}	2.8 × 10 ^{−9}	2.3 × 10 ^{−9}				
Te-123	1.00 × 10 ¹³ a	0.600	2.0 × 10 ^{−8}	0.300	9.3 × 10 ^{−9}	6.9 × 10 ^{−9}	5.4 × 10 ^{−9}	4.7 × 10 ^{−9}	4.4 × 10 ^{−9}				
Te-123m	120 d	0.600	1.9 × 10 ^{−8}	0.300	8.8 × 10 ^{−9}	4.9 × 10 ^{−9}	2.8 × 10 ^{−9}	1.7 × 10 ^{−9}	1.4 × 10 ^{−9}				
Te-125m	58.0 d	0.600	1.3 × 10 ^{−8}	0.300	6.3 × 10 ^{−9}	3.3 × 10 ^{−9}	1.9 × 10 ^{−9}	1.1 × 10 ^{−9}	8.7 × 10 ^{−10}				
Te-127	9.35 h	0.600	1.5 × 10 ^{−9}	0.300	1.2 × 10 ^{−9}	6.2 × 10 ^{−10}	3.6 × 10 ^{−10}	2.1 × 10 ^{−10}	1.7 × 10 ^{−10}				
Te-127m	109 d	0.600	4.1 × 10 ^{−8}	0.300	1.8 × 10 ^{−8}	9.5 × 10 ^{−9}	5.2 × 10 ^{−9}	3.0 × 10 ^{−9}	2.3 × 10 ^{−9}				
Te-129	1.16 h	0.600	7.5 × 10 ^{−10}	0.300	4.4 × 10 ^{−10}	2.1 × 10 ^{−10}	1.2 × 10 ^{−10}	8.0 × 10 ^{−11}	6.3 × 10 ^{−11}				
Te-129m	33.6 d	0.600	4.4 × 10 ^{−8}	0.300	2.4 × 10 ^{−8}	1.2 × 10 ^{−8}	6.6 × 10 ^{−9}	3.9 × 10 ^{−9}	3.0 × 10 ^{−9}				
Te-131	0.417 h	0.600	9.0 × 10 ^{−10}	0.300	6.6 × 10 ^{−10}	3.5 × 10 ^{−10}	1.9 × 10 ^{−10}	1.2 × 10 ^{−10}	8.7 × 10 ^{−11}				
Te-131m	1.25 d	0.600	2.0 × 10 ^{−8}	0.300	1.4 × 10 ^{−8}	7.8 × 10 ^{−9}	4.3 × 10 ^{−9}	2.7 × 10 ^{−9}	1.9 × 10 ^{−9}				
Te-132	3.26 d	0.600	4.8 × 10 ^{−8}	0.300	3.0 × 10 ^{−8}	1.6 × 10 ^{−8}	8.3 × 10 ^{−9}	5.3 × 10 ^{−9}	3.8 × 10 ^{−9}				
Te-133	0.207 h	0.600	8.4 × 10 ^{−10}	0.300	6.3 × 10 ^{−10}	3.3 × 10 ^{−10}	1.6 × 10 ^{−10}	1.1 × 10 ^{−10}	7.2 × 10 ^{−11}				
Te-133m	0.923 h	0.600	3.1 × 10 ^{−9}	0.300	2.4 × 10 ^{−9}	1.3 × 10 ^{−9}	6.3 × 10 ^{−10}	4.1 × 10 ^{−10}	2.8 × 10 ^{−10}				
Te-134	0.696 h	0.600	1.1 × 10 ^{−9}	0.300	7.5 × 10 ^{−10}	3.9 × 10 ^{−10}	2.2 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Iodine														
I-120	1.35 h	1.000	3.9 × 10 ^{−9}	1.000	2.8 × 10 ^{−9}	1.4 × 10 ^{−9}	7.2 × 10 ^{−10}	4.8 × 10 ^{−10}	3.4 × 10 ^{−10}					
I-120m	0.883 h	1.000	2.3 × 10 ^{−9}	1.000	1.5 × 10 ^{−9}	7.8 × 10 ^{−10}	4.2 × 10 ^{−10}	2.9 × 10 ^{−10}	2.1 × 10 ^{−10}					
I-121	2.12 h	1.000	6.2 × 10 ^{−10}	1.000	5.3 × 10 ^{−10}	3.1 × 10 ^{−10}	1.7 × 10 ^{−10}	1.2 × 10 ^{−10}	8.2 × 10 ^{−11}					
I-123	13.2 h	1.000	2.2 × 10 ^{−9}	1.000	1.9 × 10 ^{−9}	1.1 × 10 ^{−9}	4.9 × 10 ^{−10}	3.3 × 10 ^{−10}	2.1 × 10 ^{−10}					
I-124	4.18 d	1.000	1.2 × 10 ^{−7}	1.000	1.1 × 10 ^{−7}	6.3 × 10 ^{−8}	3.1 × 10 ^{−8}	2.0 × 10 ^{−8}	1.3 × 10 ^{−8}					
I-125	60.1 d	1.000	5.2 × 10 ^{−8}	1.000	5.7 × 10 ^{−8}	4.1 × 10 ^{−8}	3.1 × 10 ^{−8}	2.2 × 10 ^{−8}	1.5 × 10 ^{−8}					
I-126	13.0 d	1.000	2.1 × 10 ^{−7}	1.000	2.1 × 10 ^{−7}	1.3 × 10 ^{−7}	6.8 × 10 ^{−8}	4.5 × 10 ^{−8}	2.9 × 10 ^{−8}					
I-128	0.416 h	1.000	5.7 × 10 ^{−10}	1.000	3.3 × 10 ^{−10}	1.6 × 10 ^{−10}	8.9 × 10 ^{−11}	6.0 × 10 ^{−11}	4.6 × 10 ^{−11}					
I-129	1.57 × 10 ⁷ a	1.000	1.8 × 10 ^{−7}	1.000	2.2 × 10 ^{−7}	1.7 × 10 ^{−7}	1.9 × 10 ^{−7}	1.4 × 10 ^{−7}	1.1 × 10 ^{−7}					
I-130	12.4 h	1.000	2.1 × 10 ^{−8}	1.000	1.8 × 10 ^{−8}	9.8 × 10 ^{−9}	4.6 × 10 ^{−9}	3.0 × 10 ^{−9}	2.0 × 10 ^{−9}					
I-131	8.04 d	1.000	1.8 × 10 ^{−7}	1.000	1.8 × 10 ^{−7}	1.0 × 10 ^{−7}	5.2 × 10 ^{−8}	3.4 × 10 ^{−8}	2.2 × 10 ^{−8}					
I-132	2.30 h	1.000	3.0 × 10 ^{−9}	1.000	2.4 × 10 ^{−9}	1.3 × 10 ^{−9}	6.2 × 10 ^{−10}	4.1 × 10 ^{−10}	2.9 × 10 ^{−10}					
I-132m	1.39 h	1.000	2.4 × 10 ^{−9}	1.000	2.0 × 10 ^{−9}	1.1 × 10 ^{−9}	5.0 × 10 ^{−10}	3.3 × 10 ^{−10}	2.2 × 10 ^{−10}					
I-133	20.8 h	1.000	4.9 × 10 ^{−8}	1.000	4.4 × 10 ^{−8}	2.3 × 10 ^{−8}	1.0 × 10 ^{−8}	6.8 × 10 ^{−9}	4.3 × 10 ^{−9}					
I-134	0.876 h	1.000	1.1 × 10 ^{−9}	1.000	7.5 × 10 ^{−10}	3.9 × 10 ^{−10}	2.1 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}					
I-135	6.61 h	1.000	1.0 × 10 ^{−8}	1.000	8.9 × 10 ^{−9}	4.7 × 10 ^{−9}	2.2 × 10 ^{−9}	1.4 × 10 ^{−9}	9.3 × 10 ^{−10}					
Caesium														
Cs-125	0.750 h	1.000	3.9 × 10 ^{−10}	1.000	2.2 × 10 ^{−10}	1.1 × 10 ^{−10}	6.5 × 10 ^{−11}	4.4 × 10 ^{−11}	3.5 × 10 ^{−11}					
Cs-127	6.25 h	1.000	1.8 × 10 ^{−10}	1.000	1.2 × 10 ^{−10}	6.6 × 10 ^{−11}	4.2 × 10 ^{−11}	2.9 × 10 ^{−11}	2.4 × 10 ^{−11}					

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Cs-129	1.34 d	1.000	4.4 × 10 ^{−10}	1.000	3.0 × 10 ^{−10}	1.7 × 10 ^{−10}	1.1 × 10 ^{−10}	7.2 × 10 ^{−11}	6.0 × 10 ^{−11}				
Cs-130	0.498 h	1.000	3.3 × 10 ^{−10}	1.000	1.8 × 10 ^{−10}	9.0 × 10 ^{−11}	5.2 × 10 ^{−11}	3.6 × 10 ^{−11}	2.8 × 10 ^{−11}				
Cs-131	9.69 d	1.000	4.6 × 10 ^{−10}	1.000	2.9 × 10 ^{−10}	1.6 × 10 ^{−10}	1.0 × 10 ^{−10}	6.9 × 10 ^{−11}	5.8 × 10 ^{−11}				
Cs-132	6.48 d	1.000	2.7 × 10 ^{−9}	1.000	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	7.7 × 10 ^{−10}	5.7 × 10 ^{−10}	5.0 × 10 ^{−10}				
Cs-134	2.06 a	1.000	2.6 × 10 ^{−8}	1.000	1.6 × 10 ^{−8}	1.3 × 10 ^{−8}	1.4 × 10 ^{−8}	1.9 × 10 ^{−8}	1.9 × 10 ^{−8}				
Cs-134m	2.90 h	1.000	2.1 × 10 ^{−10}	1.000	1.2 × 10 ^{−10}	5.9 × 10 ^{−11}	3.5 × 10 ^{−11}	2.5 × 10 ^{−11}	2.0 × 10 ^{−11}				
Cs-135	2.30 × 10 ⁶ a	1.000	4.1 × 10 ^{−9}	1.000	2.3 × 10 ^{−9}	1.7 × 10 ^{−9}	1.7 × 10 ^{−9}	2.0 × 10 ^{−9}	2.0 × 10 ^{−9}				
Cs-135m	0.883 h	1.000	1.3 × 10 ^{−10}	1.000	8.6 × 10 ^{−11}	4.9 × 10 ^{−11}	3.2 × 10 ^{−11}	2.3 × 10 ^{−11}	1.9 × 10 ^{−11}				
Cs-136	13.1 d	1.000	1.5 × 10 ^{−8}	1.000	9.5 × 10 ^{−9}	6.1 × 10 ^{−9}	4.4 × 10 ^{−9}	3.4 × 10 ^{−9}	3.0 × 10 ^{−9}				
Cs-137	30.0 a	1.000	2.1 × 10 ^{−8}	1.000	1.2 × 10 ^{−8}	9.6 × 10 ^{−9}	1.0 × 10 ^{−8}	1.3 × 10 ^{−8}	1.3 × 10 ^{−8}				
Cs-138	0.536 h	1.000	1.1 × 10 ^{−9}	1.000	5.9 × 10 ^{−10}	2.9 × 10 ^{−10}	1.7 × 10 ^{−10}	1.2 × 10 ^{−10}	9.2 × 10 ^{−11}				
Barium^f													
Ba-126	1.61 h	0.600	2.7 × 10 ^{−9}	0.200	1.7 × 10 ^{−9}	8.5 × 10 ^{−10}	5.0 × 10 ^{−10}	3.1 × 10 ^{−10}	2.6 × 10 ^{−10}				
Ba-128	2.43 d	0.600	2.0 × 10 ^{−8}	0.200	1.7 × 10 ^{−8}	9.0 × 10 ^{−9}	5.2 × 10 ^{−9}	3.0 × 10 ^{−9}	2.7 × 10 ^{−9}				
Ba-131	11.8 d	0.600	4.2 × 10 ^{−9}	0.200	2.6 × 10 ^{−9}	1.4 × 10 ^{−9}	9.4 × 10 ^{−10}	6.2 × 10 ^{−10}	4.5 × 10 ^{−10}				
Ba-131m	0.243 h	0.600	5.8 × 10 ^{−11}	0.200	3.2 × 10 ^{−11}	1.6 × 10 ^{−11}	9.3 × 10 ^{−12}	6.3 × 10 ^{−12}	4.9 × 10 ^{−12}				
Ba-133	10.7 a	0.600	2.2 × 10 ^{−8}	0.200	6.2 × 10 ^{−9}	3.9 × 10 ^{−9}	4.6 × 10 ^{−9}	7.3 × 10 ^{−9}	1.5 × 10 ^{−9}				
Ba-133m	1.62 d	0.600	4.2 × 10 ^{−9}	0.200	3.6 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	5.9 × 10 ^{−10}	5.4 × 10 ^{−10}				
Ba-135m	1.20 d	0.600	3.3 × 10 ^{−9}	0.200	2.9 × 10 ^{−9}	1.5 × 10 ^{−9}	8.5 × 10 ^{−10}	4.7 × 10 ^{−10}	4.3 × 10 ^{−10}				
Ba-139	1.38 h	0.600	1.4 × 10 ^{−9}	0.200	8.4 × 10 ^{−10}	4.1 × 10 ^{−10}	2.4 × 10 ^{−10}	1.5 × 10 ^{−10}	1.2 × 10 ^{−10}				
Ba-140	12.7 d	0.600	3.2 × 10 ^{−8}	0.200	1.8 × 10 ^{−8}	9.2 × 10 ^{−9}	5.8 × 10 ^{−9}	3.7 × 10 ^{−9}	2.6 × 10 ^{−9}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)		<i>e</i> (g)		<i>e</i> (g)				
Lanthanum													
Ba-141	0.305 h	0.600	7.6 × 10 ^{−10}	0.200	4.7 × 10 ^{−10}		2.3 × 10 ^{−10}		1.3 × 10 ^{−10}		8.6 × 10 ^{−11}		7.0 × 10 ^{−11}
Ba-142	0.177 h	0.600	3.6 × 10 ^{−10}	0.200	2.2 × 10 ^{−10}		1.1 × 10 ^{−10}		6.6 × 10 ^{−11}		4.3 × 10 ^{−11}		3.5 × 10 ^{−11}
Lanthanum													
La-131	0.983 h	0.005	3.5 × 10 ^{−10}	5.0 × 10 ^{−4}	2.1 × 10 ^{−10}		1.1 × 10 ^{−10}		6.6 × 10 ^{−11}		4.4 × 10 ^{−11}		3.5 × 10 ^{−11}
La-132	4.80 h	0.005	3.8 × 10 ^{−9}	5.0 × 10 ^{−4}	2.4 × 10 ^{−9}		1.3 × 10 ^{−9}		7.8 × 10 ^{−10}		4.8 × 10 ^{−10}		3.9 × 10 ^{−10}
La-135	19.5 h	0.005	2.8 × 10 ^{−10}	5.0 × 10 ^{−4}	1.9 × 10 ^{−10}		1.0 × 10 ^{−10}		6.4 × 10 ^{−11}		3.9 × 10 ^{−11}		3.0 × 10 ^{−11}
La-137	6.00 × 10 ⁴ a	0.005	1.1 × 10 ^{−9}	5.0 × 10 ^{−4}	4.5 × 10 ^{−10}		2.5 × 10 ^{−10}		1.6 × 10 ^{−10}		1.0 × 10 ^{−10}		8.1 × 10 ^{−11}
La-138	1.35 × 10 ¹¹ a	0.005	1.3 × 10 ^{−8}	5.0 × 10 ^{−4}	4.6 × 10 ^{−9}		2.7 × 10 ^{−9}		1.9 × 10 ^{−9}		1.3 × 10 ^{−9}		1.1 × 10 ^{−9}
La-140	1.68 d	0.005	2.0 × 10 ^{−8}	5.0 × 10 ^{−4}	1.3 × 10 ^{−8}		6.8 × 10 ^{−9}		4.2 × 10 ^{−9}		2.5 × 10 ^{−9}		2.0 × 10 ^{−9}
La-141	3.93 h	0.005	4.3 × 10 ^{−9}	5.0 × 10 ^{−4}	2.6 × 10 ^{−9}		1.3 × 10 ^{−9}		7.6 × 10 ^{−10}		4.5 × 10 ^{−10}		3.6 × 10 ^{−10}
La-142	1.54 h	0.005	1.9 × 10 ^{−9}	5.0 × 10 ^{−4}	1.1 × 10 ^{−9}		5.8 × 10 ^{−10}		3.5 × 10 ^{−10}		2.3 × 10 ^{−10}		1.8 × 10 ^{−10}
La-143	0.237 h	0.005	6.9 × 10 ^{−10}	5.0 × 10 ^{−4}	3.9 × 10 ^{−10}		1.9 × 10 ^{−10}		1.1 × 10 ^{−10}		7.1 × 10 ^{−11}		5.6 × 10 ^{−11}
Cerium													
Ce-134	3.00 d	0.005	2.8 × 10 ^{−8}	5.0 × 10 ^{−4}	1.8 × 10 ^{−8}		9.1 × 10 ^{−9}		5.5 × 10 ^{−9}		3.2 × 10 ^{−9}		2.5 × 10 ^{−9}
Ce-135	17.6 h	0.005	7.0 × 10 ^{−9}	5.0 × 10 ^{−4}	4.7 × 10 ^{−9}		2.6 × 10 ^{−9}		1.6 × 10 ^{−9}		1.0 × 10 ^{−9}		7.9 × 10 ^{−10}
Ce-137	9.00 h	0.005	2.6 × 10 ^{−10}	5.0 × 10 ^{−4}	1.7 × 10 ^{−10}		8.8 × 10 ^{−11}		5.4 × 10 ^{−11}		3.2 × 10 ^{−11}		2.5 × 10 ^{−11}
Ce-137m	1.43 d	0.005	6.1 × 10 ^{−9}	5.0 × 10 ^{−4}	3.9 × 10 ^{−9}		2.0 × 10 ^{−9}		1.2 × 10 ^{−9}		6.8 × 10 ^{−10}		5.4 × 10 ^{−10}
Ce-139	138 d	0.005	2.6 × 10 ^{−9}	5.0 × 10 ^{−4}	1.6 × 10 ^{−9}		8.6 × 10 ^{−10}		5.4 × 10 ^{−10}		3.3 × 10 ^{−10}		2.6 × 10 ^{−10}
Ce-141	32.5 d	0.005	8.1 × 10 ^{−9}	5.0 × 10 ^{−4}	5.1 × 10 ^{−9}		2.6 × 10 ^{−9}		1.5 × 10 ^{−9}		8.8 × 10 ^{−10}		7.1 × 10 ^{−10}

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		f_1	$e(g)$		$e(g)$		$e(g)$		$e(g)$		$e(g)$		$e(g)$
Ce-143	1.38 d	0.005	1.2×10^{-8}	5.0×10^{-4}	8.0×10^{-9}		4.1×10^{-9}		2.4×10^{-9}		1.4×10^{-9}		1.1×10^{-9}
Ce-144	284 d	0.005	6.6×10^{-8}	5.0×10^{-4}	3.9×10^{-8}		1.9×10^{-8}		1.1×10^{-8}		6.5×10^{-9}		5.2×10^{-9}
Praseodymium													
Pr-136	0.218 h	0.005	3.7×10^{-10}	5.0×10^{-4}	2.1×10^{-10}		1.0×10^{-10}		6.1×10^{-11}		4.2×10^{-11}		3.3×10^{-11}
Pr-137	1.28 h	0.005	4.1×10^{-10}	5.0×10^{-4}	2.5×10^{-10}		1.3×10^{-10}		7.7×10^{-11}		5.0×10^{-11}		4.0×10^{-11}
Pr-138m	2.10 h	0.005	1.0×10^{-9}	5.0×10^{-4}	7.4×10^{-10}		4.1×10^{-10}		2.6×10^{-10}		1.6×10^{-10}		1.3×10^{-10}
Pr-139	4.51 h	0.005	3.2×10^{-10}	5.0×10^{-4}	2.0×10^{-10}		1.1×10^{-10}		6.5×10^{-11}		4.0×10^{-11}		3.1×10^{-11}
Pr-142	19.1 h	0.005	1.5×10^{-8}	5.0×10^{-4}	9.8×10^{-9}		4.9×10^{-9}		2.9×10^{-9}		1.6×10^{-9}		1.3×10^{-9}
Pr-142m	0.243 h	0.005	2.0×10^{-10}	5.0×10^{-4}	1.2×10^{-10}		6.2×10^{-11}		3.7×10^{-11}		2.1×10^{-11}		1.7×10^{-11}
Pr-143	13.6 d	0.005	1.4×10^{-8}	5.0×10^{-4}	8.7×10^{-9}		4.3×10^{-9}		2.6×10^{-9}		1.5×10^{-9}		1.2×10^{-9}
Pr-144	0.288 h	0.005	6.4×10^{-10}	5.0×10^{-4}	3.5×10^{-10}		1.7×10^{-10}		9.5×10^{-11}		6.5×10^{-11}		5.0×10^{-11}
Pr-145	5.98 h	0.005	4.7×10^{-9}	5.0×10^{-4}	2.9×10^{-9}		1.4×10^{-9}		8.5×10^{-10}		4.9×10^{-10}		3.9×10^{-10}
Pr-147	0.227 h	0.005	3.9×10^{-10}	5.0×10^{-4}	2.2×10^{-10}		1.1×10^{-10}		6.1×10^{-11}		4.2×10^{-11}		3.3×10^{-11}
Neodymium													
Nd-136	0.844 h	0.005	1.0×10^{-9}	5.0×10^{-4}	6.1×10^{-10}		3.1×10^{-10}		1.9×10^{-10}		1.2×10^{-10}		9.9×10^{-11}
Nd-138	5.04 h	0.005	7.2×10^{-9}	5.0×10^{-4}	4.5×10^{-9}		2.3×10^{-9}		1.3×10^{-9}		8.0×10^{-10}		6.4×10^{-10}
Nd-139	0.495 h	0.005	2.1×10^{-10}	5.0×10^{-4}	1.2×10^{-10}		6.3×10^{-11}		3.7×10^{-11}		2.5×10^{-11}		2.0×10^{-11}
Nd-139m	5.50 h	0.005	2.1×10^{-9}	5.0×10^{-4}	1.4×10^{-9}		7.8×10^{-10}		5.0×10^{-10}		3.1×10^{-10}		2.5×10^{-10}
Nd-141	2.49 h	0.005	7.8×10^{-11}	5.0×10^{-4}	5.0×10^{-11}		2.7×10^{-11}		1.6×10^{-11}		1.0×10^{-11}		8.3×10^{-12}
Nd-147	11.0 d	0.005	1.2×10^{-8}	5.0×10^{-4}	7.8×10^{-9}		3.9×10^{-9}		2.3×10^{-9}		1.3×10^{-9}		1.1×10^{-9}

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Promethium													
Nd-149	1.73 h	0.005	1.4 × 10 ^{−9}	5.0 × 10 ^{−4}	8.7 × 10 ^{−10}	4.3 × 10 ^{−10}	2.6 × 10 ^{−10}	1.6 × 10 ^{−10}	1.2 × 10 ^{−10}				
Nd-151	0.207 h	0.005	3.4 × 10 ^{−10}	5.0 × 10 ^{−4}	2.0 × 10 ^{−10}	9.7 × 10 ^{−11}	5.7 × 10 ^{−11}	3.8 × 10 ^{−11}	3.0 × 10 ^{−11}				
Promethium													
Pm-141	0.348 h	0.005	4.2 × 10 ^{−10}	5.0 × 10 ^{−4}	2.4 × 10 ^{−10}	1.2 × 10 ^{−10}	6.8 × 10 ^{−11}	4.6 × 10 ^{−11}	3.6 × 10 ^{−11}				
Pm-143	265 d	0.005	1.9 × 10 ^{−9}	5.0 × 10 ^{−4}	1.2 × 10 ^{−9}	6.7 × 10 ^{−10}	4.4 × 10 ^{−10}	2.9 × 10 ^{−10}	2.3 × 10 ^{−10}				
Pm-144	363 d	0.005	7.6 × 10 ^{−9}	5.0 × 10 ^{−4}	4.7 × 10 ^{−9}	2.7 × 10 ^{−9}	1.8 × 10 ^{−9}	1.2 × 10 ^{−9}	9.7 × 10 ^{−10}				
Pm-145	17.7 a	0.005	1.5 × 10 ^{−9}	5.0 × 10 ^{−4}	6.8 × 10 ^{−10}	3.7 × 10 ^{−10}	2.3 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}				
Pm-146	5.53 a	0.005	1.0 × 10 ^{−8}	5.0 × 10 ^{−4}	5.1 × 10 ^{−9}	2.8 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	9.0 × 10 ^{−10}				
Pm-147	2.62 a	0.005	3.6 × 10 ^{−9}	5.0 × 10 ^{−4}	1.9 × 10 ^{−9}	9.6 × 10 ^{−10}	5.7 × 10 ^{−10}	3.2 × 10 ^{−10}	2.6 × 10 ^{−10}				
Pm-148	5.37 d	0.005	3.0 × 10 ^{−8}	5.0 × 10 ^{−4}	1.9 × 10 ^{−8}	9.7 × 10 ^{−9}	5.8 × 10 ^{−9}	3.3 × 10 ^{−9}	2.7 × 10 ^{−9}				
Pm-148m	41.3 d	0.005	1.5 × 10 ^{−8}	5.0 × 10 ^{−4}	1.0 × 10 ^{−8}	5.5 × 10 ^{−9}	3.5 × 10 ^{−9}	2.2 × 10 ^{−9}	1.7 × 10 ^{−9}				
Pm-149	2.21 d	0.005	1.2 × 10 ^{−8}	5.0 × 10 ^{−4}	7.4 × 10 ^{−9}	3.7 × 10 ^{−9}	2.2 × 10 ^{−9}	1.2 × 10 ^{−9}	9.9 × 10 ^{−10}				
Pm-150	2.68 h	0.005	2.8 × 10 ^{−9}	5.0 × 10 ^{−4}	1.7 × 10 ^{−9}	8.7 × 10 ^{−10}	5.2 × 10 ^{−10}	3.2 × 10 ^{−10}	2.6 × 10 ^{−10}				
Pm-151	1.18 d	0.005	8.0 × 10 ^{−9}	5.0 × 10 ^{−4}	5.1 × 10 ^{−9}	2.6 × 10 ^{−9}	1.6 × 10 ^{−9}	9.1 × 10 ^{−10}	7.3 × 10 ^{−10}				
Samarium													
Sm-141	0.170 h	0.005	4.5 × 10 ^{−10}	5.0 × 10 ^{−4}	2.5 × 10 ^{−10}	1.3 × 10 ^{−10}	7.3 × 10 ^{−11}	5.0 × 10 ^{−11}	3.9 × 10 ^{−11}				
Sm-141m	0.377 h	0.005	7.0 × 10 ^{−10}	5.0 × 10 ^{−4}	4.0 × 10 ^{−10}	2.0 × 10 ^{−10}	1.2 × 10 ^{−10}	8.2 × 10 ^{−11}	6.5 × 10 ^{−11}				
Sm-142	1.21 h	0.005	2.2 × 10 ^{−9}	5.0 × 10 ^{−4}	1.3 × 10 ^{−9}	6.2 × 10 ^{−10}	3.6 × 10 ^{−10}	2.4 × 10 ^{−10}	1.9 × 10 ^{−10}				
Sm-145	340 d	0.005	2.4 × 10 ^{−9}	5.0 × 10 ^{−4}	1.4 × 10 ^{−9}	7.3 × 10 ^{−10}	4.5 × 10 ^{−10}	2.7 × 10 ^{−10}	2.1 × 10 ^{−10}				
Sm-146	1.03 × 10 ⁸ a	0.005	1.5 × 10 ^{−6}	5.0 × 10 ^{−4}	1.5 × 10 ^{−7}	1.0 × 10 ^{−7}	7.0 × 10 ^{−8}	5.8 × 10 ^{−8}	5.4 × 10 ^{−8}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a					2–7 a					7–12 a		12–17 a		>17 a	
		<i>f</i> ₁	<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)								<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)	
Sm-147	1.06 × 10 ¹¹ a	0.005	1.4 × 10 ^{−6}	5.0 × 10 ^{−4}	1.4 × 10 ^{−7}	9.2 × 10 ^{−8}	6.4 × 10 ^{−8}	5.2 × 10 ^{−8}	4.9 × 10 ^{−8}											
Sm-151	90.0 a	0.005	1.5 × 10 ^{−9}	5.0 × 10 ^{−4}	6.4 × 10 ^{−10}	3.3 × 10 ^{−10}	2.0 × 10 ^{−10}	1.2 × 10 ^{−10}	9.8 × 10 ^{−11}											
Sm-153	1.95 d	0.005	8.4 × 10 ^{−9}	5.0 × 10 ^{−4}	5.4 × 10 ^{−9}	2.7 × 10 ^{−9}	1.6 × 10 ^{−9}	9.2 × 10 ^{−10}	7.4 × 10 ^{−10}											
Sm-155	0.368 h	0.005	3.6 × 10 ^{−10}	5.0 × 10 ^{−4}	2.0 × 10 ^{−10}	9.7 × 10 ^{−11}	5.5 × 10 ^{−11}	3.7 × 10 ^{−11}	2.9 × 10 ^{−11}											
Sm-156	9.40 h	0.005	2.8 × 10 ^{−9}	5.0 × 10 ^{−4}	1.8 × 10 ^{−9}	9.0 × 10 ^{−10}	5.4 × 10 ^{−10}	3.1 × 10 ^{−10}	2.5 × 10 ^{−10}											
Europium																				
Eu-145	5.94 d	0.005	5.1 × 10 ^{−9}	5.0 × 10 ^{−4}	3.7 × 10 ^{−9}	2.1 × 10 ^{−9}	1.4 × 10 ^{−9}	9.4 × 10 ^{−10}	7.5 × 10 ^{−10}											
Eu-146	4.61 d	0.005	8.5 × 10 ^{−9}	5.0 × 10 ^{−4}	6.2 × 10 ^{−9}	3.6 × 10 ^{−9}	2.4 × 10 ^{−9}	1.6 × 10 ^{−9}	1.3 × 10 ^{−9}											
Eu-147	24.0 d	0.005	3.7 × 10 ^{−9}	5.0 × 10 ^{−4}	2.5 × 10 ^{−9}	1.4 × 10 ^{−9}	8.9 × 10 ^{−10}	5.6 × 10 ^{−10}	4.4 × 10 ^{−10}											
Eu-148	54.5 d	0.005	8.5 × 10 ^{−9}	5.0 × 10 ^{−4}	6.0 × 10 ^{−9}	3.5 × 10 ^{−9}	2.4 × 10 ^{−9}	1.6 × 10 ^{−9}	1.3 × 10 ^{−9}											
Eu-149	93.1 d	0.005	9.7 × 10 ^{−10}	5.0 × 10 ^{−4}	6.3 × 10 ^{−10}	3.4 × 10 ^{−10}	2.1 × 10 ^{−10}	1.3 × 10 ^{−10}	1.0 × 10 ^{−10}											
Eu-150	34.2 a	0.005	1.3 × 10 ^{−8}	5.0 × 10 ^{−4}	5.7 × 10 ^{−9}	3.4 × 10 ^{−9}	2.3 × 10 ^{−9}	1.5 × 10 ^{−9}	1.3 × 10 ^{−9}											
Eu-150m	12.6 h	0.005	4.4 × 10 ^{−9}	5.0 × 10 ^{−4}	2.8 × 10 ^{−9}	1.4 × 10 ^{−9}	8.2 × 10 ^{−10}	4.7 × 10 ^{−10}	3.8 × 10 ^{−10}											
Eu-152	13.3 a	0.005	1.6 × 10 ^{−8}	5.0 × 10 ^{−4}	7.4 × 10 ^{−9}	4.1 × 10 ^{−9}	2.6 × 10 ^{−9}	1.7 × 10 ^{−9}	1.4 × 10 ^{−9}											
Eu-152m	9.32 h	0.005	5.7 × 10 ^{−9}	5.0 × 10 ^{−4}	3.6 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	6.2 × 10 ^{−10}	5.0 × 10 ^{−10}											
Eu-154	8.80 a	0.005	2.5 × 10 ^{−8}	5.0 × 10 ^{−4}	1.2 × 10 ^{−8}	6.5 × 10 ^{−9}	4.1 × 10 ^{−9}	2.5 × 10 ^{−9}	2.0 × 10 ^{−9}											
Eu-155	4.96 a	0.005	4.3 × 10 ^{−9}	5.0 × 10 ^{−4}	2.2 × 10 ^{−9}	1.1 × 10 ^{−9}	6.8 × 10 ^{−10}	4.0 × 10 ^{−10}	3.2 × 10 ^{−10}											
Eu-156	15.2 d	0.005	2.2 × 10 ^{−8}	5.0 × 10 ^{−4}	1.5 × 10 ^{−8}	7.5 × 10 ^{−9}	4.6 × 10 ^{−9}	2.7 × 10 ^{−9}	2.2 × 10 ^{−9}											
Eu-157	15.1 h	0.005	6.7 × 10 ^{−9}	5.0 × 10 ^{−4}	4.3 × 10 ^{−9}	2.2 × 10 ^{−9}	1.3 × 10 ^{−9}	7.5 × 10 ^{−10}	6.0 × 10 ^{−10}											
Eu-158	0.765 h	0.005	1.1 × 10 ^{−9}	5.0 × 10 ^{−4}	6.2 × 10 ^{−10}	3.1 × 10 ^{−10}	1.8 × 10 ^{−10}	1.2 × 10 ^{−10}	9.4 × 10 ^{−11}											

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Gadolinium													
Gd-145	0.382 h	0.005	4.5 × 10 ^{−10}	5.0 × 10 ^{−4}	2.6 × 10 ^{−10}	1.3 × 10 ^{−10}	8.1 × 10 ^{−11}	5.6 × 10 ^{−11}	4.4 × 10 ^{−11}				
Gd-146	48.3 d	0.005	9.4 × 10 ^{−9}	5.0 × 10 ^{−4}	6.0 × 10 ^{−9}	3.2 × 10 ^{−9}	2.0 × 10 ^{−9}	1.2 × 10 ^{−9}	9.6 × 10 ^{−10}				
Gd-147	1.59 d	0.005	4.5 × 10 ^{−9}	5.0 × 10 ^{−4}	3.2 × 10 ^{−9}	1.8 × 10 ^{−9}	1.2 × 10 ^{−9}	7.7 × 10 ^{−10}	6.1 × 10 ^{−10}				
Gd-148	93.0 a	0.005	1.7 × 10 ^{−6}	5.0 × 10 ^{−4}	1.6 × 10 ^{−7}	1.1 × 10 ^{−7}	7.3 × 10 ^{−8}	5.9 × 10 ^{−8}	5.6 × 10 ^{−8}				
Gd-149	9.40 d	0.005	4.0 × 10 ^{−9}	5.0 × 10 ^{−4}	2.7 × 10 ^{−9}	1.5 × 10 ^{−9}	9.3 × 10 ^{−10}	5.7 × 10 ^{−10}	4.5 × 10 ^{−10}				
Gd-151	120 d	0.005	2.1 × 10 ^{−9}	5.0 × 10 ^{−4}	1.3 × 10 ^{−9}	6.8 × 10 ^{−10}	4.2 × 10 ^{−10}	2.4 × 10 ^{−10}	2.0 × 10 ^{−10}				
Gd-152	1.08 × 10 ¹⁴ a	0.005	1.2 × 10 ^{−6}	5.0 × 10 ^{−4}	1.2 × 10 ^{−7}	7.7 × 10 ^{−8}	5.3 × 10 ^{−8}	4.3 × 10 ^{−8}	4.1 × 10 ^{−8}				
Gd-153	242 d	0.005	2.9 × 10 ^{−9}	5.0 × 10 ^{−4}	1.8 × 10 ^{−9}	9.4 × 10 ^{−10}	5.8 × 10 ^{−10}	3.4 × 10 ^{−10}	2.7 × 10 ^{−10}				
Gd-159	18.6 h	0.005	5.7 × 10 ^{−9}	5.0 × 10 ^{−4}	3.6 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	6.2 × 10 ^{−10}	4.9 × 10 ^{−10}				
Terbium													
Tb-147	1.65 h	0.005	1.5 × 10 ^{−9}	5.0 × 10 ^{−4}	1.0 × 10 ^{−9}	5.4 × 10 ^{−10}	3.3 × 10 ^{−10}	2.0 × 10 ^{−10}	1.6 × 10 ^{−10}				
Tb-149	4.15 h	0.005	2.4 × 10 ^{−9}	5.0 × 10 ^{−4}	1.5 × 10 ^{−9}	8.0 × 10 ^{−10}	5.0 × 10 ^{−10}	3.1 × 10 ^{−10}	2.5 × 10 ^{−10}				
Tb-150	3.27 h	0.005	2.5 × 10 ^{−9}	5.0 × 10 ^{−4}	1.6 × 10 ^{−9}	8.3 × 10 ^{−10}	5.1 × 10 ^{−10}	3.2 × 10 ^{−10}	2.5 × 10 ^{−10}				
Tb-151	17.6 h	0.005	2.7 × 10 ^{−9}	5.0 × 10 ^{−4}	1.9 × 10 ^{−9}	1.0 × 10 ^{−9}	6.7 × 10 ^{−10}	4.2 × 10 ^{−10}	3.4 × 10 ^{−10}				
Tb-153	2.34 d	0.005	2.3 × 10 ^{−9}	5.0 × 10 ^{−4}	1.5 × 10 ^{−9}	8.2 × 10 ^{−10}	5.1 × 10 ^{−10}	3.1 × 10 ^{−10}	2.5 × 10 ^{−10}				
Tb-154	21.4 h	0.005	4.7 × 10 ^{−9}	5.0 × 10 ^{−4}	3.4 × 10 ^{−9}	1.9 × 10 ^{−9}	1.3 × 10 ^{−9}	8.1 × 10 ^{−10}	6.5 × 10 ^{−10}				
Tb-155	5.32 d	0.005	1.9 × 10 ^{−9}	5.0 × 10 ^{−4}	1.3 × 10 ^{−9}	6.8 × 10 ^{−10}	4.3 × 10 ^{−10}	2.6 × 10 ^{−10}	2.1 × 10 ^{−10}				
Tb-156	5.34 d	0.005	9.0 × 10 ^{−9}	5.0 × 10 ^{−4}	6.3 × 10 ^{−9}	3.5 × 10 ^{−9}	2.3 × 10 ^{−9}	1.5 × 10 ^{−9}	1.2 × 10 ^{−9}				
Tb-156m	1.02 d	0.005	1.5 × 10 ^{−9}	5.0 × 10 ^{−4}	1.0 × 10 ^{−9}	5.6 × 10 ^{−10}	3.5 × 10 ^{−10}	2.2 × 10 ^{−10}	1.7 × 10 ^{−10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)				
Dysprosium													
Tb-156m'	5.00 h	0.005	8.0 × 10 ^{−10}	5.0 × 10 ^{−4}	5.2 × 10 ^{−10}	2.7 × 10 ^{−10}	1.7 × 10 ^{−10}	1.0 × 10 ^{−10}	8.1 × 10 ^{−11}				
Tb-157	1.50 × 10 ² a	0.005	4.9 × 10 ^{−10}	5.0 × 10 ^{−4}	2.2 × 10 ^{−10}	1.1 × 10 ^{−10}	6.8 × 10 ^{−11}	4.1 × 10 ^{−11}	3.4 × 10 ^{−11}				
Tb-158	1.50 × 10 ² a	0.005	1.3 × 10 ^{−8}	5.0 × 10 ^{−4}	5.9 × 10 ^{−9}	3.3 × 10 ^{−9}	2.1 × 10 ^{−9}	1.4 × 10 ^{−9}	1.1 × 10 ^{−9}				
Tb-160	72.3 d	0.005	1.6 × 10 ^{−8}	5.0 × 10 ^{−4}	1.0 × 10 ^{−8}	5.4 × 10 ^{−9}	3.3 × 10 ^{−9}	2.0 × 10 ^{−9}	1.6 × 10 ^{−9}				
Tb-161	6.91 d	0.005	8.3 × 10 ^{−9}	5.0 × 10 ^{−4}	5.3 × 10 ^{−9}	2.7 × 10 ^{−9}	1.6 × 10 ^{−9}	9.0 × 10 ^{−10}	7.2 × 10 ^{−10}				
Dysprosium													
Dy-155	10.0 h	0.005	9.7 × 10 ^{−10}	5.0 × 10 ^{−4}	6.8 × 10 ^{−10}	3.8 × 10 ^{−10}	2.5 × 10 ^{−10}	1.6 × 10 ^{−10}	1.3 × 10 ^{−10}				
Dy-157	8.10 h	0.005	4.4 × 10 ^{−10}	5.0 × 10 ^{−4}	3.1 × 10 ^{−10}	1.8 × 10 ^{−10}	1.2 × 10 ^{−10}	7.7 × 10 ^{−11}	6.1 × 10 ^{−11}				
Dy-159	144 d	0.005	1.0 × 10 ^{−9}	5.0 × 10 ^{−4}	6.4 × 10 ^{−10}	3.4 × 10 ^{−10}	2.1 × 10 ^{−10}	1.3 × 10 ^{−10}	1.0 × 10 ^{−10}				
Dy-165	2.33 h	0.005	1.3 × 10 ^{−9}	5.0 × 10 ^{−4}	7.9 × 10 ^{−10}	3.9 × 10 ^{−10}	2.3 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}				
Dy-166	3.40 d	0.005	1.9 × 10 ^{−8}	5.0 × 10 ^{−4}	1.2 × 10 ^{−8}	6.0 × 10 ^{−9}	3.6 × 10 ^{−9}	2.0 × 10 ^{−9}	1.6 × 10 ^{−9}				
Holmium													
Ho-155	0.800 h	0.005	3.8 × 10 ^{−10}	5.0 × 10 ^{−4}	2.3 × 10 ^{−10}	1.2 × 10 ^{−10}	7.1 × 10 ^{−11}	4.7 × 10 ^{−11}	3.7 × 10 ^{−11}				
Ho-157	0.210 h	0.005	5.8 × 10 ^{−11}	5.0 × 10 ^{−4}	3.6 × 10 ^{−11}	1.9 × 10 ^{−11}	1.2 × 10 ^{−11}	8.1 × 10 ^{−12}	6.5 × 10 ^{−12}				
Ho-159	0.550 h	0.005	7.1 × 10 ^{−11}	5.0 × 10 ^{−4}	4.3 × 10 ^{−11}	2.3 × 10 ^{−11}	1.4 × 10 ^{−11}	9.9 × 10 ^{−12}	7.9 × 10 ^{−12}				
Ho-161	2.50 h	0.005	1.4 × 10 ^{−10}	5.0 × 10 ^{−4}	8.1 × 10 ^{−11}	4.2 × 10 ^{−11}	2.5 × 10 ^{−11}	1.6 × 10 ^{−11}	1.3 × 10 ^{−11}				
Ho-162	0.250 h	0.005	3.5 × 10 ^{−11}	5.0 × 10 ^{−4}	2.0 × 10 ^{−11}	1.0 × 10 ^{−11}	6.0 × 10 ^{−12}	4.2 × 10 ^{−12}	3.3 × 10 ^{−12}				
Ho-162m	1.13 h	0.005	2.4 × 10 ^{−10}	5.0 × 10 ^{−4}	1.5 × 10 ^{−10}	7.9 × 10 ^{−11}	4.9 × 10 ^{−11}	3.3 × 10 ^{−11}	2.6 × 10 ^{−11}				
Ho-164	0.483 h	0.005	1.2 × 10 ^{−10}	5.0 × 10 ^{−4}	6.5 × 10 ^{−11}	3.2 × 10 ^{−11}	1.8 × 10 ^{−11}	1.2 × 10 ^{−11}	9.5 × 10 ^{−12}				
Ho-164m	0.625 h	0.005	2.0 × 10 ^{−10}	5.0 × 10 ^{−4}	1.1 × 10 ^{−10}	5.5 × 10 ^{−11}	3.2 × 10 ^{−11}	2.1 × 10 ^{−11}	1.6 × 10 ^{−11}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)				
Ho-166	1.12 d	0.005	1.6 × 10 ^{−8}	5.0 × 10 ^{−4}	1.0 × 10 ^{−8}	5.2 × 10 ^{−9}	3.1 × 10 ^{−9}	1.7 × 10 ^{−9}	1.4 × 10 ^{−9}				
Ho-166m	1.20 × 10 ³ a	0.005	2.6 × 10 ^{−8}	5.0 × 10 ^{−4}	9.3 × 10 ^{−9}	5.3 × 10 ^{−9}	3.5 × 10 ^{−9}	2.4 × 10 ^{−9}	2.0 × 10 ^{−9}				
Ho-167	3.10 h	0.005	8.8 × 10 ^{−10}	5.0 × 10 ^{−4}	5.5 × 10 ^{−10}	2.8 × 10 ^{−10}	1.7 × 10 ^{−10}	1.0 × 10 ^{−10}	8.3 × 10 ^{−11}				
Erbium													
Er-161	3.24 h	0.005	6.5 × 10 ^{−10}	5.0 × 10 ^{−4}	4.4 × 10 ^{−10}	2.4 × 10 ^{−10}	1.6 × 10 ^{−10}	1.0 × 10 ^{−10}	8.0 × 10 ^{−11}				
Er-165	10.4 h	0.005	1.7 × 10 ^{−10}	5.0 × 10 ^{−4}	1.1 × 10 ^{−10}	6.2 × 10 ^{−11}	3.9 × 10 ^{−11}	2.4 × 10 ^{−11}	1.9 × 10 ^{−11}				
Er-169	9.30 d	0.005	4.4 × 10 ^{−9}	5.0 × 10 ^{−4}	2.8 × 10 ^{−9}	1.4 × 10 ^{−9}	8.2 × 10 ^{−10}	4.7 × 10 ^{−10}	3.7 × 10 ^{−10}				
Er-171	7.52 h	0.005	4.0 × 10 ^{−9}	5.0 × 10 ^{−4}	2.5 × 10 ^{−9}	1.3 × 10 ^{−9}	7.6 × 10 ^{−10}	4.5 × 10 ^{−10}	3.6 × 10 ^{−10}				
Er-172	2.05 d	0.005	1.0 × 10 ^{−8}	5.0 × 10 ^{−4}	6.8 × 10 ^{−9}	3.5 × 10 ^{−9}	2.1 × 10 ^{−9}	1.3 × 10 ^{−9}	1.0 × 10 ^{−9}				
Thulium													
Tm-162	0.362 h	0.005	2.9 × 10 ^{−10}	5.0 × 10 ^{−4}	1.7 × 10 ^{−10}	8.7 × 10 ^{−11}	5.2 × 10 ^{−11}	3.6 × 10 ^{−11}	2.9 × 10 ^{−11}				
Tm-166	7.70 h	0.005	2.1 × 10 ^{−9}	5.0 × 10 ^{−4}	1.5 × 10 ^{−9}	8.3 × 10 ^{−10}	5.5 × 10 ^{−10}	3.5 × 10 ^{−10}	2.8 × 10 ^{−10}				
Tm-167	9.24 d	0.005	6.0 × 10 ^{−9}	5.0 × 10 ^{−4}	3.9 × 10 ^{−9}	2.0 × 10 ^{−9}	1.2 × 10 ^{−9}	7.0 × 10 ^{−10}	5.6 × 10 ^{−10}				
Tm-170	129 d	0.005	1.6 × 10 ^{−8}	5.0 × 10 ^{−4}	9.8 × 10 ^{−9}	4.9 × 10 ^{−9}	2.9 × 10 ^{−9}	1.6 × 10 ^{−9}	1.3 × 10 ^{−9}				
Tm-171	1.92 a	0.005	1.5 × 10 ^{−9}	5.0 × 10 ^{−4}	7.8 × 10 ^{−10}	3.9 × 10 ^{−10}	2.3 × 10 ^{−10}	1.3 × 10 ^{−10}	1.1 × 10 ^{−10}				
Tm-172	2.65 d	0.005	1.9 × 10 ^{−8}	5.0 × 10 ^{−4}	1.2 × 10 ^{−8}	6.1 × 10 ^{−9}	3.7 × 10 ^{−9}	2.1 × 10 ^{−9}	1.7 × 10 ^{−9}				
Tm-173	8.24 h	0.005	3.3 × 10 ^{−9}	5.0 × 10 ^{−4}	2.1 × 10 ^{−9}	1.1 × 10 ^{−9}	6.5 × 10 ^{−10}	3.8 × 10 ^{−10}	3.1 × 10 ^{−10}				
Tm-175	0.253 h	0.005	3.1 × 10 ^{−10}	5.0 × 10 ^{−4}	1.7 × 10 ^{−10}	8.6 × 10 ^{−11}	5.0 × 10 ^{−11}	3.4 × 10 ^{−11}	2.7 × 10 ^{−11}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Ytterbium														
Yb-162	0.315 h	0.005	2.2 × 10 ^{−10}	5.0 × 10 ^{−4}	1.3 × 10 ^{−10}	6.9 × 10 ^{−11}	4.2 × 10 ^{−11}	2.9 × 10 ^{−11}	2.3 × 10 ^{−11}					
Yb-166	2.36 d	0.005	7.7 × 10 ^{−9}	5.0 × 10 ^{−4}	5.4 × 10 ^{−9}	2.9 × 10 ^{−9}	1.9 × 10 ^{−9}	1.2 × 10 ^{−9}	9.5 × 10 ^{−10}					
Yb-167	0.292 h	0.005	7.0 × 10 ^{−11}	5.0 × 10 ^{−4}	4.1 × 10 ^{−11}	2.1 × 10 ^{−11}	1.2 × 10 ^{−11}	8.4 × 10 ^{−12}	6.7 × 10 ^{−12}					
Yb-169	32.0 d	0.005	7.1 × 10 ^{−9}	5.0 × 10 ^{−4}	4.6 × 10 ^{−9}	2.4 × 10 ^{−9}	1.5 × 10 ^{−9}	8.8 × 10 ^{−10}	7.1 × 10 ^{−10}					
Yb-175	4.19 d	0.005	5.0 × 10 ^{−9}	5.0 × 10 ^{−4}	3.2 × 10 ^{−9}	1.6 × 10 ^{−9}	9.5 × 10 ^{−10}	5.4 × 10 ^{−10}	4.4 × 10 ^{−10}					
Yb-177	1.90 h	0.005	1.0 × 10 ^{−9}	5.0 × 10 ^{−4}	6.8 × 10 ^{−10}	3.4 × 10 ^{−10}	2.0 × 10 ^{−10}	1.1 × 10 ^{−10}	8.8 × 10 ^{−11}					
Yb-178	1.23 h	0.005	1.4 × 10 ^{−9}	5.0 × 10 ^{−4}	8.4 × 10 ^{−10}	4.2 × 10 ^{−10}	2.4 × 10 ^{−10}	1.5 × 10 ^{−10}	1.2 × 10 ^{−10}					
Lutetium														
Lu-169	1.42 d	0.005	3.5 × 10 ^{−9}	5.0 × 10 ^{−4}	2.4 × 10 ^{−9}	1.4 × 10 ^{−9}	8.9 × 10 ^{−10}	5.7 × 10 ^{−10}	4.6 × 10 ^{−10}					
Lu-170	2.00 d	0.005	7.4 × 10 ^{−9}	5.0 × 10 ^{−4}	5.2 × 10 ^{−9}	2.9 × 10 ^{−9}	1.9 × 10 ^{−9}	1.2 × 10 ^{−9}	9.9 × 10 ^{−10}					
Lu-171	8.22 d	0.005	5.9 × 10 ^{−9}	5.0 × 10 ^{−4}	4.0 × 10 ^{−9}	2.2 × 10 ^{−9}	1.4 × 10 ^{−9}	8.5 × 10 ^{−10}	6.7 × 10 ^{−10}					
Lu-172	6.70 d	0.005	1.0 × 10 ^{−8}	5.0 × 10 ^{−4}	7.0 × 10 ^{−9}	3.9 × 10 ^{−9}	2.5 × 10 ^{−9}	1.6 × 10 ^{−9}	1.3 × 10 ^{−9}					
Lu-173	1.37 a	0.005	2.7 × 10 ^{−9}	5.0 × 10 ^{−4}	1.6 × 10 ^{−9}	8.6 × 10 ^{−10}	5.3 × 10 ^{−10}	3.2 × 10 ^{−10}	2.6 × 10 ^{−1}					
Lu-174	3.31 a	0.005	3.2 × 10 ^{−9}	5.0 × 10 ^{−4}	1.7 × 10 ^{−9}	9.1 × 10 ^{−10}	5.6 × 10 ^{−10}	3.3 × 10 ^{−10}	2.7 × 10 ^{−10}					
Lu-174m	142 d	0.005	6.2 × 10 ^{−9}	5.0 × 10 ^{−4}	3.8 × 10 ^{−9}	1.9 × 10 ^{−9}	1.1 × 10 ^{−9}	6.6 × 10 ^{−10}	5.3 × 10 ^{−10}					
Lu-176	3.60 × 10 ¹⁰ a	0.005	2.4 × 10 ^{−8}	5.0 × 10 ^{−4}	1.1 × 10 ^{−8}	5.7 × 10 ^{−9}	3.5 × 10 ^{−9}	2.2 × 10 ^{−9}	1.8 × 10 ^{−9}					
Lu-176m	3.68 h	0.005	2.0 × 10 ^{−9}	5.0 × 10 ^{−4}	1.2 × 10 ^{−9}	6.0 × 10 ^{−10}	3.5 × 10 ^{−10}	2.1 × 10 ^{−10}	1.7 × 10 ^{−10}					
Lu-177	6.71 d	0.005	6.1 × 10 ^{−9}	5.0 × 10 ^{−4}	3.9 × 10 ^{−9}	2.0 × 10 ^{−9}	1.2 × 10 ^{−9}	6.6 × 10 ^{−10}	5.3 × 10 ^{−10}					
Lu-177m	161 d	0.005	1.7 × 10 ^{−8}	5.0 × 10 ^{−4}	1.1 × 10 ^{−8}	5.8 × 10 ^{−9}	3.6 × 10 ^{−9}	2.1 × 10 ^{−9}	1.7 × 10 ^{−9}					

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)				
Lu-178	0.473 h	0.005	5.9 × 10 ^{−10}	5.0 × 10 ^{−4}	3.3 × 10 ^{−10}	1.6 × 10 ^{−10}	9.0 × 10 ^{−11}	6.1 × 10 ^{−11}	4.7 × 10 ^{−11}				
Lu-178m	0.378 h	0.005	4.3 × 10 ^{−10}	5.0 × 10 ^{−4}	2.4 × 10 ^{−10}	1.2 × 10 ^{−10}	7.1 × 10 ^{−11}	4.9 × 10 ^{−11}	3.8 × 10 ^{−11}				
Lu-179	4.59 h	0.005	2.4 × 10 ^{−9}	5.0 × 10 ^{−4}	1.5 × 10 ^{−9}	7.5 × 10 ^{−10}	4.4 × 10 ^{−10}	2.6 × 10 ^{−10}	2.1 × 10 ^{−10}				
Hafnium													
Hf-170	16.0 h	0.020	3.9 × 10 ^{−9}	0.002	2.7 × 10 ^{−9}	1.5 × 10 ^{−9}	9.5 × 10 ^{−10}	6.0 × 10 ^{−10}	4.8 × 10 ^{−10}				
Hf-172	1.87 a	0.020	1.9 × 10 ^{−8}	0.002	6.1 × 10 ^{−9}	3.3 × 10 ^{−9}	2.0 × 10 ^{−9}	1.3 × 10 ^{−9}	1.0 × 10 ^{−9}				
Hf-173	24.0 h	0.020	1.9 × 10 ^{−9}	0.002	1.3 × 10 ^{−9}	7.2 × 10 ^{−10}	4.6 × 10 ^{−10}	2.8 × 10 ^{−10}	2.3 × 10 ^{−10}				
Hf-175	70.0 d	0.020	3.8 × 10 ^{−9}	0.002	2.4 × 10 ^{−9}	1.3 × 10 ^{−9}	8.4 × 10 ^{−10}	5.2 × 10 ^{−10}	4.1 × 10 ^{−10}				
Hf-177m	0.856 h	0.020	7.8 × 10 ^{−10}	0.002	4.7 × 10 ^{−10}	2.5 × 10 ^{−10}	1.5 × 10 ^{−10}	1.0 × 10 ^{−10}	8.1 × 10 ^{−11}				
Hf-178m	31.0 a	0.020	7.0 × 10 ^{−8}	0.002	1.9 × 10 ^{−8}	1.1 × 10 ^{−8}	7.8 × 10 ^{−9}	5.5 × 10 ^{−9}	4.7 × 10 ^{−9}				
Hf-179m	25.1 d	0.020	1.2 × 10 ^{−8}	0.002	7.8 × 10 ^{−9}	4.1 × 10 ^{−9}	2.6 × 10 ^{−9}	1.6 × 10 ^{−9}	1.2 × 10 ^{−9}				
Hf-180m	5.50 h	0.020	1.4 × 10 ^{−9}	0.002	9.7 × 10 ^{−10}	5.3 × 10 ^{−10}	3.3 × 10 ^{−10}	2.1 × 10 ^{−10}	1.7 × 10 ^{−10}				
Hf-181	42.4 d	0.020	1.2 × 10 ^{−8}	0.002	7.4 × 10 ^{−9}	3.8 × 10 ^{−9}	2.3 × 10 ^{−9}	1.4 × 10 ^{−9}	1.1 × 10 ^{−9}				
Hf-182	9.00 × 10 ⁶ a	0.020	5.6 × 10 ^{−8}	0.002	7.9 × 10 ^{−9}	5.4 × 10 ^{−9}	4.0 × 10 ^{−9}	3.3 × 10 ^{−9}	3.0 × 10 ^{−9}				
Hf-182m	1.02 h	0.020	4.1 × 10 ^{−10}	0.002	2.5 × 10 ^{−10}	1.3 × 10 ^{−10}	7.8 × 10 ^{−11}	5.2 × 10 ^{−11}	4.2 × 10 ^{−11}				
Hf-183	1.07 h	0.020	8.1 × 10 ^{−10}	0.002	4.8 × 10 ^{−10}	2.4 × 10 ^{−10}	1.4 × 10 ^{−10}	9.3 × 10 ^{−11}	7.3 × 10 ^{−11}				
Hf-184	4.12 h	0.020	5.5 × 10 ^{−9}	0.002	3.6 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	6.6 × 10 ^{−10}	5.2 × 10 ^{−10}				
Tantalum													
Ta-172	0.613 h	0.010	5.5 × 10 ^{−10}	0.001	3.2 × 10 ^{−10}	1.6 × 10 ^{−10}	9.8 × 10 ^{−11}	6.6 × 10 ^{−11}	5.3 × 10 ^{−11}				
Ta-173	3.65 h	0.010	2.0 × 10 ^{−9}	0.001	1.3 × 10 ^{−9}	6.5 × 10 ^{−10}	3.9 × 10 ^{−10}	2.4 × 10 ^{−10}	1.9 × 10 ^{−10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		f_1	$e(g)$		$e(g)$		$e(g)$		$e(g)$		$e(g)$		$e(g)$	
Ta-174	1.20 h	0.010	6.2×10^{-10}	0.001	3.7×10^{-10}		1.9×10^{-10}		1.1×10^{-10}		7.2×10^{-11}		5.7×10^{-11}	
Ta-175	10.5 h	0.010	1.6×10^{-9}	0.001	1.1×10^{-9}		6.2×10^{-10}		4.0×10^{-10}		2.6×10^{-10}		2.1×10^{-10}	
Ta-176	8.08 h	0.010	2.4×10^{-9}	0.001	1.7×10^{-9}		9.2×10^{-10}		6.1×10^{-10}		3.9×10^{-10}		3.1×10^{-10}	
Ta-177	2.36 d	0.010	1.0×10^{-9}	0.001	6.9×10^{-10}		3.6×10^{-10}		2.2×10^{-10}		1.3×10^{-10}		1.1×10^{-10}	
Ta-178	2.20 h	0.010	6.3×10^{-10}	0.001	4.5×10^{-10}		2.4×10^{-10}		1.5×10^{-10}		9.1×10^{-11}		7.2×10^{-11}	
Ta-179	1.82 a	0.010	6.2×10^{-10}	0.001	4.1×10^{-10}		2.2×10^{-10}		1.3×10^{-10}		8.1×10^{-11}		6.5×10^{-11}	
Ta-180	1.00×10^{13} a	0.010	8.1×10^{-9}	0.001	5.3×10^{-9}		2.8×10^{-9}		1.7×10^{-9}		1.1×10^{-9}		8.4×10^{-10}	
Ta-180m	8.10 h	0.010	5.8×10^{-10}	0.001	3.7×10^{-10}		1.9×10^{-10}		1.1×10^{-10}		6.7×10^{-11}		5.4×10^{-11}	
Ta-182	115 d	0.010	1.4×10^{-8}	0.001	9.4×10^{-9}		5.0×10^{-9}		3.1×10^{-9}		1.9×10^{-9}		1.5×10^{-9}	
Ta-182m	0.264 h	0.010	1.4×10^{-10}	0.001	7.5×10^{-11}		3.7×10^{-11}		2.1×10^{-11}		1.5×10^{-11}		1.2×10^{-11}	
Ta-183	5.10 d	0.010	1.4×10^{-8}	0.001	9.3×10^{-9}		4.7×10^{-9}		2.8×10^{-9}		1.6×10^{-9}		1.3×10^{-9}	
Ta-184	8.70 h	0.010	6.7×10^{-9}	0.001	4.4×10^{-9}		2.3×10^{-9}		1.4×10^{-9}		8.5×10^{-10}		6.8×10^{-10}	
Ta-185	0.816 h	0.010	8.3×10^{-10}	0.001	4.6×10^{-10}		2.3×10^{-10}		1.3×10^{-10}		8.6×10^{-11}		6.8×10^{-11}	
Ta-186	0.175 h	0.010	3.8×10^{-10}	0.001	2.1×10^{-10}		1.1×10^{-10}		6.1×10^{-11}		4.2×10^{-11}		3.3×10^{-11}	
Tungsten														
W-176	2.30 h	0.600	6.8×10^{-10}	0.300	5.5×10^{-10}		3.0×10^{-10}		2.0×10^{-10}		1.3×10^{-10}		1.0×10^{-10}	
W-177	2.25 h	0.600	4.4×10^{-10}	0.300	3.2×10^{-10}		1.7×10^{-10}		1.1×10^{-10}		7.2×10^{-11}		5.8×10^{-11}	
W-178	21.7 d	0.600	1.8×10^{-9}	0.300	1.4×10^{-9}		7.3×10^{-10}		4.5×10^{-10}		2.7×10^{-10}		2.2×10^{-10}	
W-179	0.625 h	0.600	3.4×10^{-11}	0.300	2.0×10^{-11}		1.0×10^{-11}		6.2×10^{-12}		4.2×10^{-12}		3.3×10^{-12}	
W-181	121 d	0.600	6.3×10^{-10}	0.300	4.7×10^{-10}		2.5×10^{-10}		1.6×10^{-10}		9.5×10^{-11}		7.6×10^{-11}	
W-185	75.1 d	0.600	4.4×10^{-9}	0.300	3.3×10^{-9}		1.6×10^{-9}		9.7×10^{-10}		5.5×10^{-10}		4.4×10^{-10}	

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1-2 a		2-7 a		7-12 a		12-17 a		$e(g)$
		f_1	$e(g)$		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	
W-187	23.9 h	0.600	5.5×10^{-9}	0.300	4.3×10^{-9}	2.2×10^{-9}	1.3×10^{-9}	7.8×10^{-10}	6.3×10^{-10}				
W-188	69.4 d	0.600	2.1×10^{-8}	0.300	1.5×10^{-8}	7.7×10^{-9}	4.6×10^{-9}	2.6×10^{-9}	2.1×10^{-9}				
Rhenium													
Re-177	0.233 h	1.000	2.5×10^{-10}	0.800	1.4×10^{-10}	7.2×10^{-11}	4.1×10^{-11}	2.8×10^{-11}	2.2×10^{-11}				
Re-178	0.220 h	1.000	2.9×10^{-10}	0.800	1.6×10^{-10}	7.9×10^{-11}	4.6×10^{-11}	3.1×10^{-11}	2.5×10^{-11}				
Re-181	20.0 h	1.000	4.2×10^{-9}	0.800	2.8×10^{-9}	1.4×10^{-9}	8.2×10^{-10}	5.4×10^{-10}	4.2×10^{-10}				
Re-182	2.67 d	1.000	1.4×10^{-8}	0.800	8.9×10^{-9}	4.7×10^{-9}	2.8×10^{-9}	1.8×10^{-9}	1.4×10^{-9}				
Re-182	12.7 h	1.000	2.4×10^{-9}	0.800	1.7×10^{-9}	8.9×10^{-10}	5.2×10^{-10}	3.5×10^{-10}	2.7×10^{-10}				
Re-184	38.0 d	1.000	8.9×10^{-9}	0.800	5.6×10^{-9}	3.0×10^{-9}	1.8×10^{-9}	1.3×10^{-9}	1.0×10^{-9}				
Re-184m	165 d	1.000	1.7×10^{-8}	0.800	9.8×10^{-9}	4.9×10^{-9}	2.8×10^{-9}	1.9×10^{-9}	1.5×10^{-9}				
Re-186	3.78 d	1.000	1.9×10^{-8}	0.800	1.1×10^{-8}	5.5×10^{-9}	3.0×10^{-9}	1.9×10^{-9}	1.5×10^{-9}				
Re-186m	2.00×10^5 a	1.000	3.0×10^{-8}	0.800	1.6×10^{-8}	7.6×10^{-9}	4.4×10^{-9}	2.8×10^{-9}	2.2×10^{-9}				
Re-187	5.00×10^{10} a	1.000	6.8×10^{-11}	0.800	3.8×10^{-11}	1.8×10^{-11}	1.0×10^{-11}	6.6×10^{-12}	5.1×10^{-12}				
Re-188	17.0 h	1.000	1.7×10^{-8}	0.800	1.1×10^{-8}	5.4×10^{-9}	2.9×10^{-9}	1.8×10^{-9}	1.4×10^{-9}				
Re-188m	0.310 h	1.000	3.8×10^{-10}	0.800	2.3×10^{-10}	1.1×10^{-10}	6.1×10^{-11}	4.0×10^{-11}	3.0×10^{-11}				
Re-189	1.01 d	1.000	9.8×10^{-9}	0.800	6.2×10^{-9}	3.0×10^{-9}	1.6×10^{-9}	1.0×10^{-9}	7.8×10^{-10}				
Osmium													
Os-180	0.366 h	0.020	1.6×10^{-10}	0.010	9.8×10^{-11}	5.1×10^{-11}	3.2×10^{-11}	2.2×10^{-11}	1.7×10^{-11}				
Os-181	1.75 h	0.020	7.6×10^{-10}	0.010	5.0×10^{-10}	2.7×10^{-10}	1.7×10^{-10}	1.1×10^{-10}	8.9×10^{-11}				
Os-182	22.0 h	0.020	4.6×10^{-9}	0.010	3.2×10^{-9}	1.7×10^{-9}	1.1×10^{-9}	7.0×10^{-10}	5.6×10^{-10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Os-185	94.0 d	0.020	3.8 × 10 ^{−9}	0.010	2.6 × 10 ^{−9}	1.5 × 10 ^{−9}	9.8 × 10 ^{−10}	6.5 × 10 ^{−10}	5.1 × 10 ^{−10}				
Os-189m	6.00 h	0.020	2.1 × 10 ^{−10}	0.010	1.3 × 10 ^{−10}	6.5 × 10 ^{−11}	3.8 × 10 ^{−11}	2.2 × 10 ^{−11}	1.8 × 10 ^{−11}				
Os-191	15.4 d	0.020	6.3 × 10 ^{−9}	0.010	4.1 × 10 ^{−9}	2.1 × 10 ^{−9}	1.2 × 10 ^{−9}	7.0 × 10 ^{−10}	5.7 × 10 ^{−10}				
Os-191m	13.0 h	0.020	1.1 × 10 ^{−9}	0.010	7.1 × 10 ^{−10}	3.5 × 10 ^{−10}	2.1 × 10 ^{−10}	1.2 × 10 ^{−10}	9.6 × 10 ^{−11}				
Os-193	1.25 d	0.020	9.3 × 10 ^{−9}	0.010	6.0 × 10 ^{−9}	3.0 × 10 ^{−9}	1.8 × 10 ^{−9}	1.0 × 10 ^{−9}	8.1 × 10 ^{−10}				
Os-194	6.00 a	0.020	2.9 × 10 ^{−8}	0.010	1.7 × 10 ^{−8}	8.8 × 10 ^{−9}	5.2 × 10 ^{−9}	3.0 × 10 ^{−9}	2.4 × 10 ^{−9}				
Iridium													
Ir-182	0.250 h	0.020	5.3 × 10 ^{−10}	0.010	3.0 × 10 ^{−10}	1.5 × 10 ^{−10}	8.9 × 10 ^{−11}	6.0 × 10 ^{−11}	4.8 × 10 ^{−11}				
Ir-184	3.02 h	0.020	1.5 × 10 ^{−9}	0.010	9.7 × 10 ^{−10}	5.2 × 10 ^{−10}	3.3 × 10 ^{−10}	2.1 × 10 ^{−10}	1.7 × 10 ^{−10}				
Ir-185	14.0 h	0.020	2.4 × 10 ^{−9}	0.010	1.6 × 10 ^{−9}	8.6 × 10 ^{−10}	5.3 × 10 ^{−10}	3.3 × 10 ^{−10}	2.6 × 10 ^{−10}				
Ir-186	15.8 h	0.020	3.8 × 10 ^{−9}	0.010	2.7 × 10 ^{−9}	1.5 × 10 ^{−9}	9.6 × 10 ^{−10}	6.1 × 10 ^{−10}	4.9 × 10 ^{−10}				
Ir-186m	1.75 h	0.020	5.8 × 10 ^{−10}	0.010	3.6 × 10 ^{−10}	2.1 × 10 ^{−10}	1.3 × 10 ^{−10}	7.7 × 10 ^{−11}	6.1 × 10 ^{−11}				
Ir-187	10.5 h	0.020	1.1 × 10 ^{−9}	0.010	7.3 × 10 ^{−10}	3.9 × 10 ^{−10}	2.5 × 10 ^{−10}	1.5 × 10 ^{−10}	1.2 × 10 ^{−10}				
Ir-188	1.73 d	0.020	4.6 × 10 ^{−9}	0.010	3.3 × 10 ^{−9}	1.8 × 10 ^{−9}	1.2 × 10 ^{−9}	7.9 × 10 ^{−10}	6.3 × 10 ^{−10}				
Ir-189	13.3 d	0.020	2.5 × 10 ^{−9}	0.010	1.7 × 10 ^{−9}	8.6 × 10 ^{−10}	5.2 × 10 ^{−10}	3.0 × 10 ^{−10}	2.4 × 10 ^{−10}				
Ir-190	12.1 d	0.020	1.0 × 10 ^{−8}	0.010	7.1 × 10 ^{−9}	3.9 × 10 ^{−9}	2.5 × 10 ^{−9}	1.6 × 10 ^{−9}	1.2 × 10 ^{−9}				
Ir-190m	3.10 h	0.020	9.4 × 10 ^{−10}	0.010	6.4 × 10 ^{−10}	3.5 × 10 ^{−10}	2.3 × 10 ^{−10}	1.5 × 10 ^{−10}	1.2 × 10 ^{−10}				
Ir-190m′	1.20 h	0.020	7.9 × 10 ^{−11}	0.010	5.0 × 10 ^{−11}	2.6 × 10 ^{−11}	1.6 × 10 ^{−11}	1.0 × 10 ^{−11}	8.0 × 10 ^{−12}				
Ir-192	74.0 d	0.020	1.3 × 10 ^{−8}	0.010	8.7 × 10 ^{−9}	4.6 × 10 ^{−9}	2.8 × 10 ^{−9}	1.7 × 10 ^{−9}	1.4 × 10 ^{−9}				
Ir-192m	2.41 × 10 ² a	0.020	2.8 × 10 ^{−9}	0.010	1.4 × 10 ^{−9}	8.3 × 10 ^{−10}	5.5 × 10 ^{−10}	3.7 × 10 ^{−10}	3.1 × 10 ^{−10}				
Ir-193m	11.9 d	0.020	3.2 × 10 ^{−9}	0.010	2.0 × 10 ^{−9}	1.0 × 10 ^{−9}	6.0 × 10 ^{−10}	3.4 × 10 ^{−10}	2.7 × 10 ^{−10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Ir-194	19.1 h	0.020	1.5 × 10 ^{−8}	0.010	9.8 × 10 ^{−9}	4.9 × 10 ^{−9}	2.9 × 10 ^{−9}	1.7 × 10 ^{−9}	1.3 × 10 ^{−9}				
Ir-194m	171 d	0.020	1.7 × 10 ^{−8}	0.010	1.1 × 10 ^{−8}	6.4 × 10 ^{−9}	4.1 × 10 ^{−9}	2.6 × 10 ^{−9}	2.1 × 10 ^{−9}				
Ir-195	2.50 h	0.020	1.2 × 10 ^{−9}	0.010	7.3 × 10 ^{−10}	3.6 × 10 ^{−10}	2.1 × 10 ^{−10}	1.3 × 10 ^{−10}	1.0 × 10 ^{−10}				
Ir-195m	3.80 h	0.020	2.3 × 10 ^{−9}	0.010	1.5 × 10 ^{−9}	7.3 × 10 ^{−10}	4.3 × 10 ^{−10}	2.6 × 10 ^{−10}	2.1 × 10 ^{−10}				
Platinum													
Pt-186	2.00 h	0.020	7.8 × 10 ^{−10}	0.010	5.3 × 10 ^{−10}	2.9 × 10 ^{−10}	1.8 × 10 ^{−10}	1.2 × 10 ^{−10}	9.3 × 10 ^{−11}				
Pt-188	10.2 d	0.020	6.7 × 10 ^{−9}	0.010	4.5 × 10 ^{−9}	2.4 × 10 ^{−9}	1.5 × 10 ^{−9}	9.5 × 10 ^{−10}	7.6 × 10 ^{−10}				
Pt-189	10.9 h	0.020	1.1 × 10 ^{−9}	0.010	7.4 × 10 ^{−10}	3.9 × 10 ^{−10}	2.5 × 10 ^{−10}	1.5 × 10 ^{−10}	1.2 × 10 ^{−10}				
Pt-191	2.80 d	0.020	3.1 × 10 ^{−9}	0.010	2.1 × 10 ^{−9}	1.1 × 10 ^{−9}	6.9 × 10 ^{−10}	4.2 × 10 ^{−10}	3.4 × 10 ^{−10}				
Pt-193	50.0 a	0.020	3.7 × 10 ^{−10}	0.010	2.4 × 10 ^{−10}	1.2 × 10 ^{−10}	6.9 × 10 ^{−11}	3.9 × 10 ^{−11}	3.1 × 10 ^{−11}				
Pt-193m	4.33 d	0.020	5.2 × 10 ^{−9}	0.010	3.4 × 10 ^{−9}	1.7 × 10 ^{−9}	9.9 × 10 ^{−10}	5.6 × 10 ^{−10}	4.5 × 10 ^{−10}				
Pt-195m	4.02 d	0.020	7.1 × 10 ^{−9}	0.010	4.6 × 10 ^{−9}	2.3 × 10 ^{−9}	1.4 × 10 ^{−9}	7.9 × 10 ^{−10}	6.3 × 10 ^{−10}				
Pt-197	18.3 h	0.020	4.7 × 10 ^{−9}	0.010	3.0 × 10 ^{−9}	1.5 × 10 ^{−9}	8.8 × 10 ^{−10}	5.1 × 10 ^{−10}	4.0 × 10 ^{−10}				
Pt-197m	1.57 h	0.020	1.0 × 10 ^{−9}	0.010	6.1 × 10 ^{−10}	3.0 × 10 ^{−10}	1.8 × 10 ^{−10}	1.1 × 10 ^{−10}	8.4 × 10 ^{−11}				
Pt-199	0.513 h	0.020	4.7 × 10 ^{−10}	0.010	2.7 × 10 ^{−10}	1.3 × 10 ^{−10}	7.5 × 10 ^{−11}	5.0 × 10 ^{−11}	3.9 × 10 ^{−11}				
Pt-200	12.5 h	0.020	1.4 × 10 ^{−8}	0.010	8.8 × 10 ^{−9}	4.4 × 10 ^{−9}	2.6 × 10 ^{−9}	1.5 × 10 ^{−9}	1.2 × 10 ^{−9}				
Gold													
Au-193	17.6 h	0.200	1.2 × 10 ^{−9}	0.100	8.8 × 10 ^{−10}	4.6 × 10 ^{−10}	2.8 × 10 ^{−10}	1.7 × 10 ^{−10}	1.3 × 10 ^{−10}				
Au-194	1.65 d	0.200	2.9 × 10 ^{−9}	0.100	2.2 × 10 ^{−9}	1.2 × 10 ^{−9}	8.1 × 10 ^{−10}	5.3 × 10 ^{−10}	4.2 × 10 ^{−10}				
Au-195	183 d	0.200	2.4 × 10 ^{−9}	0.100	1.7 × 10 ^{−9}	8.9 × 10 ^{−10}	5.4 × 10 ^{−10}	3.2 × 10 ^{−10}	2.5 × 10 ^{−10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)				
Au-198	2.69 d	0.200	1.0 × 10 ^{−8}	0.100	7.2 × 10 ^{−9}	3.7 × 10 ^{−9}	2.2 × 10 ^{−9}	1.3 × 10 ^{−9}	1.0 × 10 ^{−9}				
Au-198m	2.30 d	0.200	1.2 × 10 ^{−8}	0.100	8.5 × 10 ^{−9}	4.4 × 10 ^{−9}	2.7 × 10 ^{−9}	1.6 × 10 ^{−9}	1.3 × 10 ^{−9}				
Au-199	3.14 d	0.200	4.5 × 10 ^{−9}	0.100	3.1 × 10 ^{−9}	1.6 × 10 ^{−9}	9.5 × 10 ^{−10}	5.5 × 10 ^{−10}	4.4 × 10 ^{−10}				
Au-200	0.807 h	0.200	8.3 × 10 ^{−10}	0.100	4.7 × 10 ^{−10}	2.3 × 10 ^{−10}	1.3 × 10 ^{−10}	8.7 × 10 ^{−11}	6.8 × 10 ^{−11}				
Au-200m	18.7 h	0.200	9.2 × 10 ^{−9}	0.100	6.6 × 10 ^{−9}	3.5 × 10 ^{−9}	2.2 × 10 ^{−9}	1.3 × 10 ^{−9}	1.1 × 10 ^{−9}				
Au-201	0.440 h	0.200	3.1 × 10 ^{−10}	0.100	1.7 × 10 ^{−10}	8.2 × 10 ^{−11}	4.6 × 10 ^{−11}	3.1 × 10 ^{−11}	2.4 × 10 ^{−11}				
Mercury													
Hg-193	3.50 h	1.000	3.3 × 10 ^{−10}	1.000	1.9 × 10 ^{−10}	9.8 × 10 ^{−11}	5.8 × 10 ^{−11}	3.9 × 10 ^{−11}	3.1 × 10 ^{−11}				
(organic)		0.800	4.7 × 10 ^{−10}	0.400	4.4 × 10 ^{−10}	2.2 × 10 ^{−10}	1.4 × 10 ^{−10}	8.3 × 10 ^{−11}	6.6 × 10 ^{−11}				
Hg-193	3.50 h	0.040	8.5 × 10 ^{−10}	0.020	5.5 × 10 ^{−10}	2.8 × 10 ^{−10}	1.7 × 10 ^{−10}	1.0 × 10 ^{−10}	8.2 × 10 ^{−11}				
(inorganic)													
Hg-193m	11.1 h	1.000	1.1 × 10 ^{−9}	1.000	6.8 × 10 ^{−10}	3.7 × 10 ^{−10}	2.3 × 10 ^{−10}	1.5 × 10 ^{−10}	1.3 × 10 ^{−10}				
(organic)		0.800	1.6 × 10 ^{−9}	0.400	1.8 × 10 ^{−9}	9.5 × 10 ^{−10}	6.0 × 10 ^{−10}	3.7 × 10 ^{−10}	3.0 × 10 ^{−10}				
Hg-193m	11.1 h	0.040	3.6 × 10 ^{−9}	0.020	2.4 × 10 ^{−9}	1.3 × 10 ^{−9}	8.1 × 10 ^{−10}	5.0 × 10 ^{−10}	4.0 × 10 ^{−10}				
(inorganic)													
Hg-194	2.60 × 10 ² a	1.000	1.3 × 10 ^{−7}	1.000	1.2 × 10 ^{−7}	8.4 × 10 ^{−8}	6.6 × 10 ^{−8}	5.5 × 10 ^{−8}	5.1 × 10 ^{−8}				
(organic)		0.800	1.1 × 10 ^{−7}	0.400	4.8 × 10 ^{−8}	3.5 × 10 ^{−8}	2.7 × 10 ^{−8}	2.3 × 10 ^{−8}	2.1 × 10 ^{−8}				
Hg-194	2.60 × 10 ² a	0.040	7.2 × 10 ^{−9}	0.020	3.6 × 10 ^{−9}	2.6 × 10 ^{−9}	1.9 × 10 ^{−9}	1.5 × 10 ^{−9}	1.4 × 10 ^{−9}				
(inorganic)													
Hg-195	9.90 h	1.000	3.0 × 10 ^{−10}	1.000	2.0 × 10 ^{−10}	1.0 × 10 ^{−10}	6.4 × 10 ^{−11}	4.2 × 10 ^{−11}	3.4 × 10 ^{−11}				
(organic)		0.800	4.6 × 10 ^{−10}	0.400	4.8 × 10 ^{−10}	2.5 × 10 ^{−10}	1.5 × 10 ^{−10}	9.3 × 10 ^{−11}	7.5 × 10 ^{−11}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		> 17 a	
		f_1	$e(g)$		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$
Hg-195 (inorganic)	9.90 h	0.040	9.5×10^{-10}	0.020	6.3×10^{-10}	3.3×10^{-10}	2.0×10^{-10}	1.2×10^{-10}	9.7×10^{-11}					
Hg-195m (organic)	1.73 d	1.000	2.1×10^{-9}	1.000	1.3×10^{-9}	6.8×10^{-10}	4.2×10^{-10}	2.7×10^{-10}	2.2×10^{-10}					
Hg-195m (inorganic)	1.73 d	0.800	2.6×10^{-9}	0.400	2.8×10^{-9}	1.4×10^{-9}	8.7×10^{-10}	5.1×10^{-10}	4.1×10^{-10}					
Hg-197 (organic)	2.67 d	0.040	5.8×10^{-9}	0.020	3.8×10^{-9}	2.0×10^{-9}	1.2×10^{-9}	7.0×10^{-10}	5.6×10^{-10}					
Hg-197 (inorganic)	2.67 d	1.000	9.7×10^{-10}	1.000	6.2×10^{-10}	3.1×10^{-10}	1.9×10^{-10}	1.2×10^{-10}	9.9×10^{-11}					
Hg-197m (organic)	2.67 d	0.800	1.3×10^{-9}	0.400	1.2×10^{-9}	6.1×10^{-10}	3.7×10^{-10}	2.2×10^{-10}	1.7×10^{-10}					
Hg-197m (inorganic)	23.8 h	0.040	2.5×10^{-9}	0.020	1.6×10^{-9}	8.3×10^{-10}	5.0×10^{-10}	2.9×10^{-10}	2.3×10^{-10}					
Hg-197m (organic)	23.8 h	1.000	1.5×10^{-9}	1.000	9.5×10^{-10}	4.8×10^{-10}	2.9×10^{-10}	1.8×10^{-10}	1.5×10^{-10}					
Hg-199m (organic)	0.710 h	0.800	2.2×10^{-9}	0.400	2.5×10^{-9}	1.2×10^{-9}	7.3×10^{-10}	4.2×10^{-10}	3.4×10^{-10}					
Hg-199m (inorganic)	0.710 h	0.040	5.2×10^{-9}	0.020	3.4×10^{-9}	1.7×10^{-9}	1.0×10^{-9}	5.9×10^{-10}	4.7×10^{-10}					
Hg-199m (organic)	0.710 h	1.000	3.4×10^{-10}	1.000	1.9×10^{-10}	9.3×10^{-11}	5.3×10^{-11}	3.6×10^{-11}	2.8×10^{-11}					
Hg-203 (organic)	46.6 d	0.800	3.6×10^{-10}	0.400	2.1×10^{-10}	1.0×10^{-10}	5.8×10^{-11}	3.9×10^{-11}	3.1×10^{-11}					
Hg-203 (inorganic)	46.6 d	0.040	3.7×10^{-10}	0.020	2.1×10^{-10}	1.0×10^{-10}	5.9×10^{-11}	3.9×10^{-11}	3.1×10^{-11}					
Hg-203 (organic)	46.6 d	1.000	1.5×10^{-8}	1.000	1.1×10^{-8}	5.7×10^{-9}	3.6×10^{-9}	2.3×10^{-9}	1.9×10^{-9}					
Hg-203 (inorganic)	46.6 d	0.800	1.3×10^{-8}	0.400	6.4×10^{-9}	3.4×10^{-9}	2.1×10^{-9}	1.3×10^{-9}	1.1×10^{-9}					
Hg-203 (inorganic)	46.6 d	0.040	5.5×10^{-9}	0.020	3.6×10^{-9}	1.8×10^{-9}	1.1×10^{-9}	6.7×10^{-10}	5.4×10^{-10}					

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)				
Thallium													
Tl-194	0.550 h	1.000	6.1 × 10 ^{−11}	1.000	3.9 × 10 ^{−11}	2.2 × 10 ^{−11}	1.4 × 10 ^{−11}	1.0 × 10 ^{−11}	8.1 × 10 ^{−12}				
Tl-194m	0.546 h	1.000	3.8 × 10 ^{−10}	1.000	2.2 × 10 ^{−10}	1.2 × 10 ^{−10}	7.0 × 10 ^{−11}	4.9 × 10 ^{−11}	4.0 × 10 ^{−11}				
Tl-195	1.16 h	1.000	2.3 × 10 ^{−10}	1.000	1.4 × 10 ^{−10}	7.5 × 10 ^{−11}	4.7 × 10 ^{−11}	3.3 × 10 ^{−11}	2.7 × 10 ^{−11}				
Tl-197	2.84 h	1.000	2.1 × 10 ^{−10}	1.000	1.3 × 10 ^{−10}	6.7 × 10 ^{−11}	4.2 × 10 ^{−11}	2.8 × 10 ^{−11}	2.3 × 10 ^{−11}				
Tl-198	5.30 h	1.000	4.7 × 10 ^{−10}	1.000	3.3 × 10 ^{−10}	1.9 × 10 ^{−10}	1.2 × 10 ^{−10}	8.7 × 10 ^{−11}	7.3 × 10 ^{−11}				
Tl-198m	1.87 h	1.000	4.8 × 10 ^{−10}	1.000	3.0 × 10 ^{−10}	1.6 × 10 ^{−10}	9.7 × 10 ^{−11}	6.7 × 10 ^{−11}	5.4 × 10 ^{−11}				
Tl-199	7.42 h	1.000	2.3 × 10 ^{−10}	1.000	1.5 × 10 ^{−10}	7.7 × 10 ^{−11}	4.8 × 10 ^{−11}	3.2 × 10 ^{−11}	2.6 × 10 ^{−11}				
Tl-200	1.09 d	1.000	1.3 × 10 ^{−9}	1.000	9.1 × 10 ^{−10}	5.3 × 10 ^{−10}	3.5 × 10 ^{−10}	2.4 × 10 ^{−10}	2.0 × 10 ^{−10}				
Tl-201	3.04 d	1.000	8.4 × 10 ^{−10}	1.000	5.5 × 10 ^{−10}	2.9 × 10 ^{−10}	1.8 × 10 ^{−10}	1.2 × 10 ^{−10}	9.5 × 10 ^{−11}				
Tl-202	12.2 d	1.000	2.9 × 10 ^{−9}	1.000	2.1 × 10 ^{−9}	1.2 × 10 ^{−9}	7.9 × 10 ^{−10}	5.4 × 10 ^{−10}	4.5 × 10 ^{−10}				
Tl-204	3.78 a	1.000	1.3 × 10 ^{−8}	1.000	8.5 × 10 ^{−9}	4.2 × 10 ^{−9}	2.5 × 10 ^{−9}	1.5 × 10 ^{−9}	1.2 × 10 ^{−9}				
Lead^g													
Pb-195m	0.263 h	0.600	2.6 × 10 ^{−10}	0.200	1.6 × 10 ^{−10}	8.4 × 10 ^{−11}	5.2 × 10 ^{−11}	3.5 × 10 ^{−11}	2.9 × 10 ^{−11}				
Pb-198	2.40 h	0.600	5.9 × 10 ^{−10}	0.200	4.8 × 10 ^{−10}	2.7 × 10 ^{−10}	1.7 × 10 ^{−10}	1.1 × 10 ^{−10}	1.0 × 10 ^{−10}				
Pb-199	1.50 h	0.600	3.5 × 10 ^{−10}	0.200	2.6 × 10 ^{−10}	1.5 × 10 ^{−10}	9.4 × 10 ^{−11}	6.3 × 10 ^{−11}	5.4 × 10 ^{−11}				
Pb-200	21.5 h	0.600	2.5 × 10 ^{−9}	0.200	2.0 × 10 ^{−9}	1.1 × 10 ^{−9}	7.0 × 10 ^{−10}	4.4 × 10 ^{−10}	4.0 × 10 ^{−10}				
Pb-201	9.40 h	0.600	9.4 × 10 ^{−10}	0.200	7.8 × 10 ^{−10}	4.3 × 10 ^{−10}	2.7 × 10 ^{−10}	1.8 × 10 ^{−10}	1.6 × 10 ^{−10}				
Pb-202	3.00 × 10 ⁵ a	0.600	3.4 × 10 ^{−8}	0.200	1.6 × 10 ^{−8}	1.3 × 10 ^{−8}	1.9 × 10 ^{−8}	2.7 × 10 ^{−8}	8.8 × 10 ^{−9}				
Pb-202m	3.62 h	0.600	7.6 × 10 ^{−10}	0.200	6.1 × 10 ^{−10}	3.5 × 10 ^{−10}	2.3 × 10 ^{−10}	1.5 × 10 ^{−10}	1.3 × 10 ^{−10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		> 17 a
		f_1	$e(g)$		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	
Pb-203	2.17 d	0.600	1.6×10^{-9}	0.200	1.3×10^{-9}	6.8×10^{-10}	4.3×10^{-10}	2.7×10^{-10}	2.4×10^{-10}	2.7×10^{-10}	2.4×10^{-10}	2.4×10^{-10}	
Pb-205	1.43×10^7 a	0.600	2.1×10^{-9}	0.200	9.9×10^{-10}	6.2×10^{-10}	6.1×10^{-10}	6.5×10^{-10}	2.8×10^{-10}	6.5×10^{-10}	2.8×10^{-10}	2.8×10^{-10}	
Pb-209	3.25 h	0.600	5.7×10^{-10}	0.200	3.8×10^{-10}	1.9×10^{-10}	1.1×10^{-10}	6.6×10^{-11}	5.7×10^{-11}	6.6×10^{-11}	5.7×10^{-11}	5.7×10^{-11}	
Pb-210	22.3 a	0.600	8.4×10^{-6}	0.200	3.6×10^{-6}	2.2×10^{-6}	1.9×10^{-6}	1.9×10^{-6}	6.9×10^{-7}	1.9×10^{-6}	6.9×10^{-7}	6.9×10^{-7}	
Pb-211	0.601 h	0.600	3.1×10^{-9}	0.200	1.4×10^{-9}	7.1×10^{-10}	4.1×10^{-10}	2.7×10^{-10}	1.8×10^{-10}	2.7×10^{-10}	1.8×10^{-10}	1.8×10^{-10}	
Pb-212	10.6 h	0.600	1.5×10^{-7}	0.200	6.3×10^{-8}	3.3×10^{-8}	2.0×10^{-8}	1.3×10^{-8}	6.0×10^{-9}	1.3×10^{-8}	6.0×10^{-9}	6.0×10^{-9}	
Pb-214	0.447 h	0.600	2.7×10^{-9}	0.200	1.0×10^{-9}	5.2×10^{-10}	3.1×10^{-10}	2.0×10^{-10}	1.4×10^{-10}	3.1×10^{-10}	2.0×10^{-10}	1.4×10^{-10}	
Bismuth													
Bi-200	0.606 h	0.100	4.2×10^{-10}	0.050	2.7×10^{-10}	1.5×10^{-10}	9.5×10^{-11}	6.4×10^{-11}	5.1×10^{-11}	9.5×10^{-11}	6.4×10^{-11}	5.1×10^{-11}	
Bi-201	1.80 h	0.100	1.0×10^{-9}	0.050	6.7×10^{-10}	3.6×10^{-10}	2.2×10^{-10}	1.4×10^{-10}	1.2×10^{-10}	2.2×10^{-10}	1.4×10^{-10}	1.2×10^{-10}	
Bi-202	1.67 h	0.100	6.4×10^{-10}	0.050	4.4×10^{-10}	2.5×10^{-10}	1.6×10^{-10}	1.1×10^{-10}	8.9×10^{-11}	1.6×10^{-10}	1.1×10^{-10}	8.9×10^{-11}	
Bi-203	11.8 h	0.100	3.5×10^{-9}	0.050	2.5×10^{-9}	1.4×10^{-9}	9.3×10^{-10}	6.0×10^{-10}	4.8×10^{-10}	9.3×10^{-10}	6.0×10^{-10}	4.8×10^{-10}	
Bi-205	15.3 d	0.100	6.1×10^{-9}	0.050	4.5×10^{-9}	2.6×10^{-9}	1.7×10^{-9}	1.1×10^{-9}	9.0×10^{-10}	1.7×10^{-9}	1.1×10^{-9}	9.0×10^{-10}	
Bi-206	6.24 d	0.100	1.4×10^{-8}	0.050	1.0×10^{-8}	5.7×10^{-9}	3.7×10^{-9}	2.4×10^{-9}	1.9×10^{-9}	3.7×10^{-9}	2.4×10^{-9}	1.9×10^{-9}	
Bi-207	38.0 a	0.100	1.0×10^{-8}	0.050	7.1×10^{-9}	3.9×10^{-9}	2.5×10^{-9}	1.6×10^{-9}	1.3×10^{-9}	2.5×10^{-9}	1.6×10^{-9}	1.3×10^{-9}	
Bi-210	5.01 d	0.100	1.5×10^{-8}	0.050	9.7×10^{-9}	4.8×10^{-9}	2.9×10^{-9}	1.6×10^{-9}	1.3×10^{-9}	2.9×10^{-9}	1.6×10^{-9}	1.3×10^{-9}	
Bi-210m	3.00×10^6 a	0.100	2.1×10^{-7}	0.050	9.1×10^{-8}	4.7×10^{-8}	3.0×10^{-8}	1.9×10^{-8}	1.5×10^{-8}	3.0×10^{-8}	1.9×10^{-8}	1.5×10^{-8}	
Bi-212	1.01 h	0.100	3.2×10^{-9}	0.050	1.8×10^{-9}	8.7×10^{-10}	5.0×10^{-10}	3.3×10^{-10}	2.6×10^{-10}	5.0×10^{-10}	3.3×10^{-10}	2.6×10^{-10}	
Bi-213	0.761 h	0.100	2.5×10^{-9}	0.050	1.4×10^{-9}	6.7×10^{-10}	3.9×10^{-10}	2.5×10^{-10}	2.0×10^{-10}	3.9×10^{-10}	2.5×10^{-10}	2.0×10^{-10}	
Bi-214	0.332 h	0.100	1.4×10^{-9}	0.050	7.4×10^{-10}	3.6×10^{-10}	2.1×10^{-10}	1.4×10^{-10}	1.1×10^{-10}	2.1×10^{-10}	1.4×10^{-10}	1.1×10^{-10}	

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Polonium													
Po-203	0.612 h	1.000	2.9 × 10 ^{−10}	0.500	2.4 × 10 ^{−10}		1.3 × 10 ^{−10}		8.5 × 10 ^{−11}		5.8 × 10 ^{−11}		4.6 × 10 ^{−11}
Po-205	1.80 h	1.000	3.5 × 10 ^{−10}	0.500	2.8 × 10 ^{−10}		1.6 × 10 ^{−10}		1.1 × 10 ^{−10}		7.2 × 10 ^{−11}		5.8 × 10 ^{−11}
Po-207	5.83 h	1.000	4.4 × 10 ^{−10}	0.500	5.7 × 10 ^{−10}		3.2 × 10 ^{−10}		2.1 × 10 ^{−10}		1.4 × 10 ^{−10}		1.1 × 10 ^{−1}
Po-210	138 d	1.000	2.6 × 10 ^{−5}	0.500	8.8 × 10 ^{−6}		4.4 × 10 ^{−6}		2.6 × 10 ^{−6}		1.6 × 10 ^{−6}		1.2 × 10 ^{−6}
Astatine													
At-207	1.80 h	1.000	2.5 × 10 ^{−9}	1.000	1.6 × 10 ^{−9}		8.0 × 10 ^{−10}		4.8 × 10 ^{−10}		2.9 × 10 ^{−10}		2.4 × 10 ^{−10}
At-211	7.21 h	1.000	1.2 × 10 ^{−7}	1.000	7.8 × 10 ^{−8}		3.8 × 10 ^{−8}		2.3 × 10 ^{−8}		1.3 × 10 ^{−8}		1.1 × 10 ^{−8}
Francium													
Fr-222	0.240 h	1.000	6.2 × 10 ^{−9}	1.000	3.9 × 10 ^{−9}		2.0 × 10 ^{−9}		1.3 × 10 ^{−9}		8.5 × 10 ^{−10}		7.2 × 10 ^{−10}
Fr-223	0.363 h	1.000	2.6 × 10 ^{−8}	1.000	1.7 × 10 ^{−8}		8.3 × 10 ^{−9}		5.0 × 10 ^{−9}		2.9 × 10 ^{−9}		2.4 × 10 ^{−9}
Radium^b													
Ra-223	11.4 d	0.600	5.3 × 10 ^{−6}	0.200	1.1 × 10 ^{−6}		5.7 × 10 ^{−7}		4.5 × 10 ^{−7}		3.7 × 10 ^{−7}		1.0 × 10 ^{−7}
Ra-224	3.66 d	0.600	2.7 × 10 ^{−6}	0.200	6.6 × 10 ^{−7}		3.5 × 10 ^{−7}		2.6 × 10 ^{−7}		2.0 × 10 ^{−7}		6.5 × 10 ^{−8}
Ra-225	14.8 d	0.600	7.1 × 10 ^{−6}	0.200	1.2 × 10 ^{−6}		6.1 × 10 ^{−7}		5.0 × 10 ^{−7}		4.4 × 10 ^{−7}		9.9 × 10 ^{−8}
Ra-226	1.60 × 10 ³ a	0.600	4.7 × 10 ^{−6}	0.200	9.6 × 10 ^{−7}		6.2 × 10 ^{−7}		8.0 × 10 ^{−7}		1.5 × 10 ^{−6}		2.8 × 10 ^{−7}
Ra-227	0.703 h	0.600	1.1 × 10 ^{−9}	0.200	4.3 × 10 ^{−10}		2.5 × 10 ^{−10}		1.7 × 10 ^{−10}		1.3 × 10 ^{−10}		8.1 × 10 ^{−11}
Ra-228	5.75 a	0.600	3.0 × 10 ^{−5}	0.200	5.7 × 10 ^{−6}		3.4 × 10 ^{−6}		3.9 × 10 ^{−6}		5.3 × 10 ^{−6}		6.9 × 10 ^{−7}

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)				
Actinium													
Ac-224	2.90 h	0.005	1.0 × 10 ^{−8}	5.0 × 10 ^{−4}	5.2 × 10 ^{−9}	2.6 × 10 ^{−9}	1.5 × 10 ^{−9}	8.8 × 10 ^{−10}	7.0 × 10 ^{−10}				
Ac-225	10.0 d	0.005	4.6 × 10 ^{−7}	5.0 × 10 ^{−4}	1.8 × 10 ^{−7}	9.1 × 10 ^{−8}	5.4 × 10 ^{−8}	3.0 × 10 ^{−8}	2.4 × 10 ^{−8}				
Ac-226	1.21 d	0.005	1.4 × 10 ^{−7}	5.0 × 10 ^{−4}	7.6 × 10 ^{−8}	3.8 × 10 ^{−8}	2.3 × 10 ^{−8}	1.3 × 10 ^{−8}	1.0 × 10 ^{−8}				
Ac-227	21.8 a	0.005	3.3 × 10 ^{−5}	5.0 × 10 ^{−4}	3.1 × 10 ^{−6}	2.2 × 10 ^{−6}	1.5 × 10 ^{−6}	1.2 × 10 ^{−6}	1.1 × 10 ^{−6}				
Ac-228	6.13 h	0.005	7.4 × 10 ^{−9}	5.0 × 10 ^{−4}	2.8 × 10 ^{−9}	1.4 × 10 ^{−9}	8.7 × 10 ^{−10}	5.3 × 10 ^{−10}	4.3 × 10 ^{−10}				
Thorium													
Th-226	0.515 h	0.005	4.4 × 10 ^{−9}	5.0 × 10 ^{−4}	2.4 × 10 ^{−9}	1.2 × 10 ^{−9}	6.7 × 10 ^{−10}	4.5 × 10 ^{−10}	3.5 × 10 ^{−10}				
Th-227	18.7 d	0.005	3.0 × 10 ^{−7}	5.0 × 10 ^{−4}	7.0 × 10 ^{−8}	3.6 × 10 ^{−8}	2.3 × 10 ^{−8}	1.5 × 10 ^{−8}	8.8 × 10 ^{−9}				
Th-228	1.91 a	0.005	3.7 × 10 ^{−6}	5.0 × 10 ^{−4}	3.7 × 10 ^{−7}	2.2 × 10 ^{−7}	1.5 × 10 ^{−7}	9.4 × 10 ^{−8}	7.2 × 10 ^{−8}				
Th-229	7.34 × 10 ³ a	0.005	1.1 × 10 ^{−5}	5.0 × 10 ^{−4}	1.0 × 10 ^{−6}	7.8 × 10 ^{−7}	6.2 × 10 ^{−7}	5.3 × 10 ^{−7}	4.9 × 10 ^{−7}				
Th-230	7.70 × 10 ⁴ a	0.005	4.1 × 10 ^{−6}	5.0 × 10 ^{−4}	4.1 × 10 ^{−7}	3.1 × 10 ^{−7}	2.4 × 10 ^{−7}	2.2 × 10 ^{−7}	2.1 × 10 ^{−7}				
Th-231	1.06 d	0.005	3.9 × 10 ^{−9}	5.0 × 10 ^{−4}	2.5 × 10 ^{−9}	1.2 × 10 ^{−9}	7.4 × 10 ^{−10}	4.2 × 10 ^{−10}	3.4 × 10 ^{−10}				
Th-232	1.40 × 10 ¹⁰ a	0.005	4.6 × 10 ^{−6}	5.0 × 10 ^{−4}	4.5 × 10 ^{−7}	3.5 × 10 ^{−7}	2.9 × 10 ^{−7}	2.5 × 10 ^{−7}	2.3 × 10 ^{−7}				
Th-234	24.1 d	0.005	4.0 × 10 ^{−8}	5.0 × 10 ^{−4}	2.5 × 10 ^{−8}	1.3 × 10 ^{−8}	7.4 × 10 ^{−9}	4.2 × 10 ^{−9}	3.4 × 10 ^{−9}				
Protactinium													
Pa-227	0.638 h	0.005	5.8 × 10 ^{−9}	5.0 × 10 ^{−4}	3.2 × 10 ^{−9}	1.5 × 10 ^{−9}	8.7 × 10 ^{−10}	5.8 × 10 ^{−10}	4.5 × 10 ^{−10}				
Pa-228	22.0 h	0.005	1.2 × 10 ^{−8}	5.0 × 10 ^{−4}	4.8 × 10 ^{−9}	2.6 × 10 ^{−9}	1.6 × 10 ^{−9}	9.7 × 10 ^{−10}	7.8 × 10 ^{−10}				
Pa-230	17.4 d	0.005	2.6 × 10 ^{−8}	5.0 × 10 ^{−4}	5.7 × 10 ^{−9}	3.1 × 10 ^{−9}	1.9 × 10 ^{−9}	1.1 × 10 ^{−9}	9.2 × 10 ^{−10}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a				7–12 a				12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)					
Pa-231	3.27 × 10 ⁴ a	0.005	1.3 × 10 ^{−5}	5.0 × 10 ^{−4}	1.3 × 10 ^{−6}	1.1 × 10 ^{−6}	9.2 × 10 ^{−7}	8.0 × 10 ^{−7}	8.0 × 10 ^{−7}	8.0 × 10 ^{−7}	8.0 × 10 ^{−7}	8.0 × 10 ^{−7}	8.0 × 10 ^{−7}	7.1 × 10 ^{−7}	
Pa-232	1.31 d	0.005	6.3 × 10 ^{−9}	5.0 × 10 ^{−4}	4.2 × 10 ^{−9}	2.2 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	
Pa-233	27.0 d	0.005	9.7 × 10 ^{−9}	5.0 × 10 ^{−4}	6.2 × 10 ^{−9}	3.2 × 10 ^{−9}	1.9 × 10 ^{−9}	1.9 × 10 ^{−9}	1.9 × 10 ^{−9}	1.9 × 10 ^{−9}	1.9 × 10 ^{−9}	1.9 × 10 ^{−9}	1.9 × 10 ^{−9}	1.9 × 10 ^{−9}	
Pa-234	6.70 h	0.005	5.0 × 10 ^{−9}	5.0 × 10 ^{−4}	3.2 × 10 ^{−9}	1.7 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	
Uranium															
U-230	20.8 d	0.040	7.9 × 10 ^{−7}	0.020	3.0 × 10 ^{−7}	1.5 × 10 ^{−7}	1.0 × 10 ^{−7}	6.6 × 10 ^{−8}	6.6 × 10 ^{−8}	6.6 × 10 ^{−8}	6.6 × 10 ^{−8}	6.6 × 10 ^{−8}	6.6 × 10 ^{−8}	6.6 × 10 ^{−8}	
U-231	4.20 d	0.040	3.1 × 10 ^{−9}	0.020	2.0 × 10 ^{−9}	1.0 × 10 ^{−9}	6.1 × 10 ^{−10}	3.5 × 10 ^{−10}	3.5 × 10 ^{−10}	3.5 × 10 ^{−10}	3.5 × 10 ^{−10}	3.5 × 10 ^{−10}	3.5 × 10 ^{−10}	3.5 × 10 ^{−10}	
U-232	72.0 a	0.040	2.5 × 10 ^{−6}	0.020	8.2 × 10 ^{−7}	5.8 × 10 ^{−7}	5.7 × 10 ^{−7}	6.4 × 10 ^{−7}	6.4 × 10 ^{−7}	6.4 × 10 ^{−7}	6.4 × 10 ^{−7}	6.4 × 10 ^{−7}	6.4 × 10 ^{−7}	6.4 × 10 ^{−7}	
U-233	1.58 × 10 ⁵ a	0.040	3.8 × 10 ^{−7}	0.020	1.4 × 10 ^{−7}	9.2 × 10 ^{−8}	7.8 × 10 ^{−8}	7.8 × 10 ^{−8}	7.8 × 10 ^{−8}	7.8 × 10 ^{−8}	7.8 × 10 ^{−8}	7.8 × 10 ^{−8}	7.8 × 10 ^{−8}	7.8 × 10 ^{−8}	
U-234	2.44 × 10 ⁵ a	0.040	3.7 × 10 ^{−7}	0.020	1.3 × 10 ^{−7}	8.8 × 10 ^{−8}	7.4 × 10 ^{−8}	7.4 × 10 ^{−8}	7.4 × 10 ^{−8}	7.4 × 10 ^{−8}	7.4 × 10 ^{−8}	7.4 × 10 ^{−8}	7.4 × 10 ^{−8}	7.4 × 10 ^{−8}	
U-235	7.04 × 10 ⁸ a	0.040	3.5 × 10 ^{−7}	0.020	1.3 × 10 ^{−7}	8.5 × 10 ^{−8}	7.1 × 10 ^{−8}	7.0 × 10 ^{−8}	7.0 × 10 ^{−8}	7.0 × 10 ^{−8}	7.0 × 10 ^{−8}	7.0 × 10 ^{−8}	7.0 × 10 ^{−8}	7.0 × 10 ^{−8}	
U-236	2.34 × 10 ⁷ a	0.040	3.5 × 10 ^{−7}	0.020	1.3 × 10 ^{−7}	8.4 × 10 ^{−8}	7.0 × 10 ^{−8}	7.0 × 10 ^{−8}	7.0 × 10 ^{−8}	7.0 × 10 ^{−8}	7.0 × 10 ^{−8}	7.0 × 10 ^{−8}	7.0 × 10 ^{−8}	7.0 × 10 ^{−8}	
U-237	6.75 d	0.040	8.3 × 10 ^{−9}	0.020	5.4 × 10 ^{−9}	2.8 × 10 ^{−9}	1.6 × 10 ^{−9}	1.6 × 10 ^{−9}	1.6 × 10 ^{−9}	1.6 × 10 ^{−9}	1.6 × 10 ^{−9}	1.6 × 10 ^{−9}	1.6 × 10 ^{−9}	1.6 × 10 ^{−9}	
U-238	4.47 × 10 ⁹ a	0.040	3.4 × 10 ^{−7}	0.020	1.2 × 10 ^{−7}	8.0 × 10 ^{−8}	6.8 × 10 ^{−8}	6.8 × 10 ^{−8}	6.8 × 10 ^{−8}	6.8 × 10 ^{−8}	6.8 × 10 ^{−8}	6.8 × 10 ^{−8}	6.8 × 10 ^{−8}	6.8 × 10 ^{−8}	
U-239	0.392 h	0.040	3.4 × 10 ^{−10}	0.020	1.9 × 10 ^{−10}	9.3 × 10 ^{−11}	5.4 × 10 ^{−11}	3.5 × 10 ^{−11}	3.5 × 10 ^{−11}	3.5 × 10 ^{−11}	3.5 × 10 ^{−11}	3.5 × 10 ^{−11}	3.5 × 10 ^{−11}	3.5 × 10 ^{−11}	
U-240	14.1 h	0.040	1.3 × 10 ^{−8}	0.020	8.1 × 10 ^{−9}	4.1 × 10 ^{−9}	2.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	1.4 × 10 ^{−9}	
Neptunium															
Np-232	0.245 h	0.005	8.7 × 10 ^{−11}	5.0 × 10 ^{−4}	5.1 × 10 ^{−11}	2.7 × 10 ^{−11}	1.7 × 10 ^{−11}	1.2 × 10 ^{−11}	1.2 × 10 ^{−11}	1.2 × 10 ^{−11}	1.2 × 10 ^{−11}	1.2 × 10 ^{−11}	1.2 × 10 ^{−11}	1.2 × 10 ^{−11}	
Np-233	0.603 h	0.005	2.1 × 10 ^{−11}	5.0 × 10 ^{−4}	1.3 × 10 ^{−11}	6.6 × 10 ^{−12}	4.0 × 10 ^{−12}	2.8 × 10 ^{−12}	2.8 × 10 ^{−12}	2.8 × 10 ^{−12}	2.8 × 10 ^{−12}	2.8 × 10 ^{−12}	2.8 × 10 ^{−12}	2.8 × 10 ^{−12}	
Np-234	4.40 d	0.005	6.2 × 10 ^{−9}	5.0 × 10 ^{−4}	4.4 × 10 ^{−9}	2.4 × 10 ^{−9}	1.6 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a				Age 2–7 a				7–12 a		12–17 a		>17 a	
		f_1	$e(g)$		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$
Np-235	1.08 a	0.005	7.1×10^{-10}	5.0×10^{-4}	4.1×10^{-10}	2.0×10^{-10}	1.2×10^{-10}	6.8×10^{-11}	5.3×10^{-11}									
Np-236	1.15×10^5 a	0.005	1.9×10^{-7}	5.0×10^{-4}	2.4×10^{-8}	1.8×10^{-8}	1.8×10^{-8}	1.8×10^{-8}	1.7×10^{-8}									
Np-236m	22.5 h	0.005	2.5×10^{-9}	5.0×10^{-4}	1.3×10^{-9}	6.6×10^{-10}	4.0×10^{-10}	2.4×10^{-10}	1.9×10^{-10}									
Np-237	2.14×10^6 a	0.005	2.0×10^{-6}	5.0×10^{-4}	2.1×10^{-7}	1.4×10^{-7}	1.1×10^{-7}	1.1×10^{-7}	1.1×10^{-7}									
Np-238	2.12 d	0.005	9.5×10^{-9}	5.0×10^{-4}	6.2×10^{-9}	3.2×10^{-9}	1.9×10^{-9}	1.1×10^{-9}	9.1×10^{-10}									
Np-239	2.36 d	0.005	8.9×10^{-9}	5.0×10^{-4}	5.7×10^{-9}	2.9×10^{-9}	1.7×10^{-9}	1.0×10^{-9}	8.0×10^{-10}									
Np-240	1.08 h	0.005	8.7×10^{-10}	5.0×10^{-4}	5.2×10^{-10}	2.6×10^{-10}	1.6×10^{-10}	1.0×10^{-10}	8.2×10^{-11}									
Plutonium																		
Pu-234	8.80 h	0.005	2.1×10^{-9}	5.0×10^{-4}	1.1×10^{-9}	5.5×10^{-10}	3.3×10^{-10}	2.0×10^{-10}	1.6×10^{-10}									
Pu-235	0.422 h	0.005	2.2×10^{-11}	5.0×10^{-4}	1.3×10^{-11}	6.5×10^{-12}	3.9×10^{-12}	2.7×10^{-12}	2.1×10^{-12}									
Pu-236	2.85 a	0.005	2.1×10^{-6}	5.0×10^{-4}	2.2×10^{-7}	1.4×10^{-7}	1.0×10^{-7}	8.5×10^{-8}	8.7×10^{-8}									
Pu-237	45.3 d	0.005	1.1×10^{-9}	5.0×10^{-4}	6.9×10^{-10}	3.6×10^{-10}	2.2×10^{-10}	1.3×10^{-10}	1.0×10^{-10}									
Pu-238	87.7 a	0.005	4.0×10^{-6}	5.0×10^{-4}	4.0×10^{-7}	3.1×10^{-7}	2.4×10^{-7}	2.2×10^{-7}	2.3×10^{-7}									
Pu-239	2.41×10^4 a	0.005	4.2×10^{-6}	5.0×10^{-4}	4.2×10^{-7}	3.3×10^{-7}	2.7×10^{-7}	2.4×10^{-7}	2.5×10^{-7}									
Pu-240	6.54×10^3 a	0.005	4.2×10^{-6}	5.0×10^{-4}	4.2×10^{-7}	3.3×10^{-7}	2.7×10^{-7}	2.4×10^{-7}	2.5×10^{-7}									
Pu-241	14.4 a	0.005	5.6×10^{-8}	5.0×10^{-4}	5.7×10^{-9}	5.5×10^{-9}	5.1×10^{-9}	4.8×10^{-9}	4.8×10^{-9}									
Pu-242	3.76×10^5 a	0.005	4.0×10^{-6}	5.0×10^{-4}	4.0×10^{-7}	3.2×10^{-7}	2.6×10^{-7}	2.3×10^{-7}	2.4×10^{-7}									
Pu-243	4.95 h	0.005	1.0×10^{-9}	5.0×10^{-4}	6.2×10^{-10}	3.1×10^{-10}	1.8×10^{-10}	1.1×10^{-10}	8.5×10^{-11}									
Pu-244	8.26×10^7 a	0.005	4.0×10^{-6}	5.0×10^{-4}	4.1×10^{-7}	3.2×10^{-7}	2.6×10^{-7}	2.3×10^{-7}	2.4×10^{-7}									
Pu-245	10.5 h	0.005	8.0×10^{-9}	5.0×10^{-4}	5.1×10^{-9}	2.6×10^{-9}	1.5×10^{-9}	8.9×10^{-10}	7.2×10^{-10}									
Pu-246	10.9 d	0.005	3.6×10^{-8}	5.0×10^{-4}	2.3×10^{-8}	1.2×10^{-8}	7.1×10^{-9}	4.1×10^{-9}	3.3×10^{-9}									

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		f_1	$e(g)$		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$				
Americium													
Am-237	1.22 h	0.005	1.7×10^{-10}	5.0×10^{-4}	1.0×10^{-10}	5.5×10^{-11}	3.3×10^{-11}	2.2×10^{-11}	1.8×10^{-11}				
Am-238	1.63 h	0.005	2.5×10^{-10}	5.0×10^{-4}	1.6×10^{-10}	9.1×10^{-11}	5.9×10^{-11}	4.0×10^{-11}	3.2×10^{-11}				
Am-239	11.9 h	0.005	2.6×10^{-9}	5.0×10^{-4}	1.7×10^{-9}	8.4×10^{-10}	5.1×10^{-10}	3.0×10^{-10}	2.4×10^{-10}				
Am-240	2.12 d	0.005	4.7×10^{-9}	5.0×10^{-4}	3.3×10^{-9}	1.8×10^{-9}	1.2×10^{-9}	7.3×10^{-10}	5.8×10^{-10}				
Am-241	4.32×10^2 a	0.005	3.7×10^{-6}	5.0×10^{-4}	3.7×10^{-7}	2.7×10^{-7}	2.2×10^{-7}	2.0×10^{-7}	2.0×10^{-7}				
Am-242	16.0 h	0.005	5.0×10^{-9}	5.0×10^{-4}	2.2×10^{-9}	1.1×10^{-9}	6.4×10^{-10}	3.7×10^{-10}	3.0×10^{-10}				
Am-242m	1.52×10^2 a	0.005	3.1×10^{-6}	5.0×10^{-4}	3.0×10^{-7}	2.3×10^{-7}	2.0×10^{-7}	1.9×10^{-7}	1.9×10^{-7}				
Am-243	7.38×10^3 a	0.005	3.6×10^{-6}	5.0×10^{-4}	3.7×10^{-7}	2.7×10^{-7}	2.2×10^{-7}	2.0×10^{-7}	2.0×10^{-7}				
Am-244	10.1 h	0.005	4.9×10^{-9}	5.0×10^{-4}	3.1×10^{-9}	1.6×10^{-9}	9.6×10^{-10}	5.8×10^{-10}	4.6×10^{-10}				
Am-244m	0.433 h	0.005	3.7×10^{-10}	5.0×10^{-4}	2.0×10^{-10}	9.6×10^{-11}	5.5×10^{-11}	3.7×10^{-11}	2.9×10^{-11}				
Am-245	2.05 h	0.005	6.8×10^{-10}	5.0×10^{-4}	4.5×10^{-10}	2.2×10^{-10}	1.3×10^{-10}	7.9×10^{-11}	6.2×10^{-11}				
Am-246	0.650 h	0.005	6.7×10^{-10}	5.0×10^{-4}	3.8×10^{-10}	1.9×10^{-10}	1.1×10^{-10}	7.3×10^{-11}	5.8×10^{-11}				
Am-246m	0.417 h	0.005	3.9×10^{-10}	5.0×10^{-4}	2.2×10^{-10}	1.1×10^{-10}	6.4×10^{-11}	4.4×10^{-11}	3.4×10^{-11}				
Curium													
Cm-238	2.40 h	0.005	7.8×10^{-10}	5.0×10^{-4}	4.9×10^{-10}	2.6×10^{-10}	1.6×10^{-10}	1.0×10^{-10}	8.0×10^{-11}				
Cm-240	27.0 d	0.005	2.2×10^{-7}	5.0×10^{-4}	4.8×10^{-8}	2.5×10^{-8}	1.5×10^{-8}	9.2×10^{-9}	7.6×10^{-9}				
Cm-241	32.8 d	0.005	1.1×10^{-8}	5.0×10^{-4}	5.7×10^{-9}	3.0×10^{-9}	1.9×10^{-9}	1.1×10^{-9}	9.1×10^{-10}				
Cm-242	163 d	0.005	5.9×10^{-7}	5.0×10^{-4}	7.6×10^{-8}	3.9×10^{-8}	2.4×10^{-8}	1.5×10^{-8}	1.2×10^{-8}				
Cm-243	28.5 a	0.005	3.2×10^{-6}	5.0×10^{-4}	3.3×10^{-7}	2.2×10^{-7}	1.6×10^{-7}	1.4×10^{-7}	1.5×10^{-7}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		f_1	$e(g)$		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$				
Cm-244	18.1 a	0.005	2.9×10^{-6}	5.0×10^{-4}	2.9×10^{-7}	1.9×10^{-7}	1.4×10^{-7}	1.2×10^{-7}	1.2×10^{-7}	1.2×10^{-7}	1.2×10^{-7}	1.2×10^{-7}	
Cm-245	8.50×10^3 a	0.005	3.7×10^{-6}	5.0×10^{-4}	3.7×10^{-7}	2.8×10^{-7}	2.3×10^{-7}	2.1×10^{-7}	2.3×10^{-7}	2.1×10^{-7}	2.1×10^{-7}	2.1×10^{-7}	
Cm-246	4.73×10^3 a	0.005	3.7×10^{-6}	5.0×10^{-4}	3.7×10^{-7}	2.8×10^{-7}	2.2×10^{-7}	2.1×10^{-7}	2.2×10^{-7}	2.1×10^{-7}	2.1×10^{-7}	2.1×10^{-7}	
Cm-247	1.56×10^7 a	0.005	3.4×10^{-6}	5.0×10^{-4}	3.5×10^{-7}	2.6×10^{-7}	2.1×10^{-7}	1.9×10^{-7}	2.1×10^{-7}	1.9×10^{-7}	1.9×10^{-7}	1.9×10^{-7}	
Cm-248	3.39×10^5 a	0.005	1.4×10^{-5}	5.0×10^{-4}	1.4×10^{-6}	1.0×10^{-6}	8.4×10^{-7}	7.7×10^{-7}	8.4×10^{-7}	7.7×10^{-7}	7.7×10^{-7}	7.7×10^{-7}	
Cm-249	1.07 h	0.005	3.9×10^{-10}	5.0×10^{-4}	2.2×10^{-10}	1.1×10^{-10}	6.1×10^{-11}	4.0×10^{-11}	6.1×10^{-11}	4.0×10^{-11}	4.0×10^{-11}	3.1×10^{-11}	
Cm-250	6.90×10^3 a	0.005	7.8×10^{-5}	5.0×10^{-4}	8.2×10^{-6}	6.0×10^{-6}	4.9×10^{-6}	4.4×10^{-6}	4.9×10^{-6}	4.4×10^{-6}	4.4×10^{-6}	4.4×10^{-6}	
Berkelium													
Bk-245	4.94 d	0.005	6.1×10^{-9}	5.0×10^{-4}	3.9×10^{-9}	2.0×10^{-9}	1.2×10^{-9}	7.2×10^{-10}	1.2×10^{-9}	7.2×10^{-10}	7.2×10^{-10}	5.7×10^{-10}	
Bk-246	1.83 d	0.005	3.7×10^{-9}	5.0×10^{-4}	2.6×10^{-9}	1.4×10^{-9}	9.4×10^{-10}	6.0×10^{-10}	9.4×10^{-10}	6.0×10^{-10}	6.0×10^{-10}	4.8×10^{-10}	
Bk-247	1.38×10^3 a	0.005	8.9×10^{-6}	5.0×10^{-4}	8.6×10^{-7}	6.3×10^{-7}	4.6×10^{-7}	3.8×10^{-7}	4.6×10^{-7}	3.8×10^{-7}	3.8×10^{-7}	3.5×10^{-7}	
Bk-249	320 d	0.005	2.2×10^{-8}	5.0×10^{-4}	2.9×10^{-9}	1.9×10^{-9}	1.4×10^{-9}	1.1×10^{-9}	1.4×10^{-9}	1.1×10^{-9}	1.1×10^{-9}	9.7×10^{-10}	
Bk-250	3.22 h	0.005	1.5×10^{-9}	5.0×10^{-4}	8.5×10^{-10}	4.4×10^{-10}	2.7×10^{-10}	1.7×10^{-10}	2.7×10^{-10}	1.7×10^{-10}	1.7×10^{-10}	1.4×10^{-10}	
Californium													
Cf-244	0.323 h	0.005	9.8×10^{-10}	5.0×10^{-4}	4.8×10^{-10}	2.4×10^{-10}	1.3×10^{-10}	8.9×10^{-11}	1.3×10^{-10}	8.9×10^{-11}	8.9×10^{-11}	7.0×10^{-11}	
Cf-246	1.49 d	0.005	5.0×10^{-8}	5.0×10^{-4}	2.4×10^{-8}	1.2×10^{-8}	7.3×10^{-9}	4.1×10^{-9}	7.3×10^{-9}	4.1×10^{-9}	4.1×10^{-9}	3.3×10^{-9}	
Cf-248	334 d	0.005	1.5×10^{-6}	5.0×10^{-4}	1.6×10^{-7}	9.9×10^{-8}	6.0×10^{-8}	3.3×10^{-8}	6.0×10^{-8}	3.3×10^{-8}	3.3×10^{-8}	2.8×10^{-8}	

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Cf-249	3.50 × 10 ² a	0.005	9.0 × 10 ^{−6}	5.0 × 10 ^{−4}	8.7 × 10 ^{−7}	6.4 × 10 ^{−7}	4.7 × 10 ^{−7}	3.8 × 10 ^{−7}	3.5 × 10 ^{−7}				
Cf-250	13.1 a	0.005	5.7 × 10 ^{−6}	5.0 × 10 ^{−4}	5.5 × 10 ^{−7}	3.7 × 10 ^{−7}	2.3 × 10 ^{−7}	1.7 × 10 ^{−7}	1.6 × 10 ^{−7}				
Cf-251	8.98 × 10 ² a	0.005	9.1 × 10 ^{−6}	5.0 × 10 ^{−4}	8.8 × 10 ^{−7}	6.5 × 10 ^{−7}	4.7 × 10 ^{−7}	3.9 × 10 ^{−7}	3.6 × 10 ^{−7}				
Cf-252	2.64 a	0.005	5.0 × 10 ^{−6}	5.0 × 10 ^{−4}	5.1 × 10 ^{−7}	3.2 × 10 ^{−7}	1.9 × 10 ^{−7}	1.0 × 10 ^{−7}	9.0 × 10 ^{−8}				
Cf-253	17.8 d	0.005	1.0 × 10 ^{−7}	5.0 × 10 ^{−4}	1.1 × 10 ^{−8}	6.0 × 10 ^{−9}	3.7 × 10 ^{−9}	1.8 × 10 ^{−9}	1.4 × 10 ^{−9}				
Cf-254	60.5 d	0.005	1.1 × 10 ^{−5}	5.0 × 10 ^{−4}	2.6 × 10 ^{−6}	1.4 × 10 ^{−6}	8.4 × 10 ^{−7}	5.0 × 10 ^{−7}	4.0 × 10 ^{−7}				
Einsteinium													
Es-250	2.10 h	0.005	2.3 × 10 ^{−10}	5.0 × 10 ^{−4}	9.9 × 10 ^{−11}	5.7 × 10 ^{−11}	3.7 × 10 ^{−11}	2.6 × 10 ^{−11}	2.1 × 10 ^{−11}				
Es-251	1.38 d	0.005	1.9 × 10 ^{−9}	5.0 × 10 ^{−4}	1.2 × 10 ^{−9}	6.1 × 10 ^{−10}	3.7 × 10 ^{−10}	2.2 × 10 ^{−10}	1.7 × 10 ^{−10}				
Es-253	20.5 d	0.005	1.7 × 10 ^{−7}	5.0 × 10 ^{−4}	4.5 × 10 ^{−8}	2.3 × 10 ^{−8}	1.4 × 10 ^{−8}	7.6 × 10 ^{−9}	6.1 × 10 ^{−9}				
Es-254	276 d	0.005	1.4 × 10 ^{−6}	5.0 × 10 ^{−4}	1.6 × 10 ^{−7}	9.8 × 10 ^{−8}	6.0 × 10 ^{−8}	3.3 × 10 ^{−8}	2.8 × 10 ^{−8}				
Es-254m	1.64 d	0.005	5.7 × 10 ^{−8}	5.0 × 10 ^{−4}	3.0 × 10 ^{−8}	1.5 × 10 ^{−8}	9.1 × 10 ^{−9}	5.2 × 10 ^{−9}	4.2 × 10 ^{−9}				
Fermium													
Fm-252	22.7 h	0.005	3.8 × 10 ^{−8}	5.0 × 10 ^{−4}	2.0 × 10 ^{−8}	9.9 × 10 ^{−9}	5.9 × 10 ^{−9}	3.3 × 10 ^{−9}	2.7 × 10 ^{−9}				
Fm-253	3.00 d	0.005	2.5 × 10 ^{−8}	5.0 × 10 ^{−4}	6.7 × 10 ^{−9}	3.4 × 10 ^{−9}	2.1 × 10 ^{−9}	1.1 × 10 ^{−9}	9.1 × 10 ^{−10}				
Fm-254	3.24 h	0.005	5.6 × 10 ^{−9}	5.0 × 10 ^{−4}	3.2 × 10 ^{−9}	1.6 × 10 ^{−9}	9.3 × 10 ^{−10}	5.6 × 10 ^{−10}	4.4 × 10 ^{−10}				
Fm-255	20.1 h	0.005	3.3 × 10 ^{−8}	5.0 × 10 ^{−4}	1.9 × 10 ^{−8}	9.5 × 10 ^{−9}	5.6 × 10 ^{−9}	3.2 × 10 ^{−9}	2.5 × 10 ^{−9}				
Fm-257	101 d	0.005	9.8 × 10 ^{−7}	5.0 × 10 ^{−4}	1.1 × 10 ^{−7}	6.5 × 10 ^{−8}	4.0 × 10 ^{−8}	1.9 × 10 ^{−8}	1.5 × 10 ^{−8}				

TABLE III.2D. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INGESTION (Sv/Bq) (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		f_1	$e(g)$		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$				
Mendelevium														
Md-257	5.20 h	0.005	3.1×10^{-9}	5.0×10^{-4}	8.8×10^{-10}	4.5×10^{-10}	2.7×10^{-10}	1.5×10^{-10}	1.2×10^{-10}					
Md-258	55.0 d	0.005	6.3×10^{-7}	5.0×10^{-4}	8.9×10^{-8}	5.0×10^{-8}	3.0×10^{-8}	1.6×10^{-8}	1.3×10^{-8}					

^a m and m' denote metastable states of the radionuclide. The metastable state m' is of higher energy than the metastable state m.

^b The f_1 value for calcium for 1–15 year olds is 0.4.

^c The f_1 value for iron for 1–15 year olds is 0.2.

^d The f_1 value for cobalt for 1–15 year olds is 0.3.

^e The f_1 value for strontium for 1–15 year olds is 0.4.

^f The f_1 value for barium for 1–15 year olds is 0.3.

^g The f_1 value for lead for 1–15 year olds is 0.4.

^h The f_1 value for radium for 1–15 year olds is 0.3.

Note: f_1 : gut transfer factor; $e(g)$: effective dose per unit intake by age group.

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a	
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)
Hydrogen																		
H-3	12.3 a	F	1.000	2.6 × 10 ^{−11}	1.000	2.0 × 10 ^{−11}	1.1 × 10 ^{−11}	8.2 × 10 ^{−12}	5.9 × 10 ^{−12}	6.2 × 10 ^{−12}								
		M	0.200	3.4 × 10 ^{−10}	0.100	2.7 × 10 ^{−10}	1.4 × 10 ^{−10}	8.2 × 10 ^{−11}	5.3 × 10 ^{−11}	4.5 × 10 ^{−11}								
		S	0.020	1.2 × 10 ^{−9}	0.010	1.0 × 10 ^{−9}	6.3 × 10 ^{−10}	3.8 × 10 ^{−10}	2.8 × 10 ^{−10}	2.6 × 10 ^{−10}								
Beryllium																		
Be-7	53.3 d	M	0.020	2.5 × 10 ^{−10}	0.005	2.1 × 10 ^{−10}	1.2 × 10 ^{−10}	8.3 × 10 ^{−11}	6.2 × 10 ^{−11}	5.0 × 10 ^{−11}								
		S	0.020	2.8 × 10 ^{−10}	0.005	2.4 × 10 ^{−10}	1.4 × 10 ^{−10}	9.6 × 10 ^{−11}	6.8 × 10 ^{−11}	5.5 × 10 ^{−11}								
Be-10	1.60 × 10 ⁶ a	M	0.020	4.1 × 10 ^{−8}	0.005	3.4 × 10 ^{−8}	2.0 × 10 ^{−8}	1.3 × 10 ^{−8}	1.1 × 10 ^{−8}	9.6 × 10 ^{−9}								
		S	0.020	9.9 × 10 ^{−8}	0.005	9.1 × 10 ^{−8}	6.1 × 10 ^{−8}	4.2 × 10 ^{−8}	3.7 × 10 ^{−8}	3.5 × 10 ^{−8}								
Carbon																		
C-11	0.340 h	F	1.000	1.0 × 10 ^{−10}	1.000	7.0 × 10 ^{−11}	3.2 × 10 ^{−11}	2.1 × 10 ^{−11}	1.3 × 10 ^{−11}	1.1 × 10 ^{−11}								
		M	0.200	1.5 × 10 ^{−10}	0.100	1.1 × 10 ^{−10}	4.9 × 10 ^{−11}	3.2 × 10 ^{−11}	2.1 × 10 ^{−11}	1.8 × 10 ^{−11}								
		S	0.020	1.6 × 10 ^{−10}	0.010	1.1 × 10 ^{−10}	5.1 × 10 ^{−11}	3.3 × 10 ^{−11}	2.2 × 10 ^{−11}	1.8 × 10 ^{−11}								
C-14	5.73 × 10 ³ a	F	1.000	6.1 × 10 ^{−10}	1.000	6.7 × 10 ^{−10}	3.6 × 10 ^{−10}	2.9 × 10 ^{−10}	1.9 × 10 ^{−10}	2.0 × 10 ^{−10}								
		M	0.200	8.3 × 10 ^{−9}	0.100	6.6 × 10 ^{−9}	4.0 × 10 ^{−9}	2.8 × 10 ^{−9}	2.5 × 10 ^{−9}	2.0 × 10 ^{−9}								
		S	0.020	1.9 × 10 ^{−8}	0.010	1.7 × 10 ^{−8}	1.1 × 10 ^{−8}	7.4 × 10 ^{−9}	6.4 × 10 ^{−9}	5.8 × 10 ^{−9}								

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Fluorine													
F-18	1.83 h	F	1.000	2.6×10^{-10}	1.000	1.9×10^{-10}	9.1×10^{-11}	5.6×10^{-11}	3.4×10^{-11}	2.8×10^{-11}			
		M	1.000	4.1×10^{-10}	1.000	2.9×10^{-10}	1.5×10^{-10}	9.7×10^{-11}	6.9×10^{-11}	5.6×10^{-11}			
		S	1.000	4.2×10^{-10}	1.000	3.1×10^{-10}	1.5×10^{-10}	1.0×10^{-10}	7.3×10^{-11}	5.9×10^{-11}			
Sodium													
Na-22	2.60 a	F	1.000	9.7×10^{-9}	1.000	7.3×10^{-9}	3.8×10^{-9}	2.4×10^{-9}	1.5×10^{-9}	1.3×10^{-9}			
Na-24	15.0 h	F	1.000	2.3×10^{-9}	1.000	1.8×10^{-9}	9.3×10^{-10}	5.7×10^{-10}	3.4×10^{-10}	2.7×10^{-10}			
Magnesium													
Mg-28	20.9 h	F	1.000	5.3×10^{-9}	0.500	4.7×10^{-9}	2.2×10^{-9}	1.3×10^{-9}	7.3×10^{-10}	6.0×10^{-10}			
		M	1.000	7.3×10^{-9}	0.500	7.2×10^{-9}	3.5×10^{-9}	2.3×10^{-9}	1.5×10^{-9}	1.2×10^{-9}			
Aluminium													
Al-26	7.16×10^5 a	F	0.020	8.1×10^{-8}	0.010	6.2×10^{-8}	3.2×10^{-8}	2.0×10^{-8}	1.3×10^{-8}	1.1×10^{-8}			
		M	0.020	8.8×10^{-8}	0.010	7.4×10^{-8}	4.4×10^{-8}	2.9×10^{-8}	2.2×10^{-8}	2.0×10^{-8}			
Silicon													
Si-31	2.62 h	F	0.020	3.6×10^{-10}	0.010	2.3×10^{-10}	9.5×10^{-11}	5.9×10^{-11}	3.2×10^{-11}	2.7×10^{-11}			
		M	0.020	6.9×10^{-10}	0.010	4.4×10^{-10}	2.0×10^{-10}	1.3×10^{-10}	8.9×10^{-11}	7.4×10^{-11}			
		S	0.020	7.2×10^{-10}	0.010	4.7×10^{-10}	2.2×10^{-10}	1.4×10^{-10}	9.5×10^{-11}	7.9×10^{-11}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)			
Si-32	4.50 × 10 ² a	F	0.020	3.0 × 10 ⁻⁸	0.010	2.3 × 10 ⁻⁸	1.1 × 10 ⁻⁸	6.4 × 10 ⁻⁹	3.8 × 10 ⁻⁹	3.2 × 10 ⁻⁹							
		M	0.020	7.1 × 10 ⁻⁸	0.010	6.0 × 10 ⁻⁸	3.6 × 10 ⁻⁸	2.4 × 10 ⁻⁸	1.9 × 10 ⁻⁸	1.7 × 10 ⁻⁸							
		S	0.020	2.8 × 10 ⁻⁷	0.010	2.7 × 10 ⁻⁷	1.9 × 10 ⁻⁷	1.3 × 10 ⁻⁷	1.1 × 10 ⁻⁷	1.1 × 10 ⁻⁷							
Phosphorus																	
P-32	14.3 d	F	1.000	1.2 × 10 ⁻⁸	0.800	7.5 × 10 ⁻⁹	3.2 × 10 ⁻⁹	1.8 × 10 ⁻⁹	9.8 × 10 ⁻¹⁰	7.7 × 10 ⁻¹⁰							
		M	1.000	2.2 × 10 ⁻⁸	0.800	1.5 × 10 ⁻⁸	8.0 × 10 ⁻⁹	5.3 × 10 ⁻⁹	4.0 × 10 ⁻⁹	3.4 × 10 ⁻⁹							
P-33	25.4 d	F	1.000	1.2 × 10 ⁻⁹	0.800	7.8 × 10 ⁻¹⁰	3.0 × 10 ⁻¹⁰	2.0 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰	9.2 × 10 ⁻¹¹							
		M	1.000	6.1 × 10 ⁻⁹	0.800	4.6 × 10 ⁻⁹	2.8 × 10 ⁻⁹	2.1 × 10 ⁻⁹	1.9 × 10 ⁻⁹	1.5 × 10 ⁻⁹							
Sulphur																	
S-35 (inorganic)	87.4 d	F	1.000	5.5 × 10 ⁻¹⁰	0.800	3.9 × 10 ⁻¹⁰	1.8 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰	6.0 × 10 ⁻¹¹	5.1 × 10 ⁻¹¹							
		M	0.200	5.9 × 10 ⁻⁹	0.100	4.5 × 10 ⁻⁹	2.8 × 10 ⁻⁹	2.0 × 10 ⁻⁹	1.8 × 10 ⁻⁹	1.4 × 10 ⁻⁹							
		S	0.020	7.7 × 10 ⁻⁹	0.010	6.0 × 10 ⁻⁹	3.6 × 10 ⁻⁹	2.6 × 10 ⁻⁹	2.3 × 10 ⁻⁹	1.9 × 10 ⁻⁹							
Chlorine																	
Cl-36	3.01 × 10 ⁵ a	F	1.000	3.9 × 10 ⁻⁹	1.000	2.6 × 10 ⁻⁹	1.1 × 10 ⁻⁹	7.1 × 10 ⁻¹⁰	3.9 × 10 ⁻¹⁰	3.3 × 10 ⁻¹⁰							
		M	1.000	3.1 × 10 ⁻⁸	1.000	2.6 × 10 ⁻⁸	1.5 × 10 ⁻⁸	1.0 × 10 ⁻⁸	8.8 × 10 ⁻⁹	7.3 × 10 ⁻⁹							
Cl-38	0.620 h	F	1.000	2.9 × 10 ⁻¹⁰	1.000	1.9 × 10 ⁻¹⁰	8.4 × 10 ⁻¹¹	5.1 × 10 ⁻¹¹	3.0 × 10 ⁻¹¹	2.5 × 10 ⁻¹¹							
		M	1.000	4.7 × 10 ⁻¹⁰	1.000	3.0 × 10 ⁻¹⁰	1.4 × 10 ⁻¹⁰	8.5 × 10 ⁻¹¹	5.4 × 10 ⁻¹¹	4.5 × 10 ⁻¹¹							
Cl-39	0.927 h	F	1.000	2.7 × 10 ⁻¹⁰	1.000	1.8 × 10 ⁻¹⁰	8.4 × 10 ⁻¹¹	5.1 × 10 ⁻¹¹	3.1 × 10 ⁻¹¹	2.5 × 10 ⁻¹¹							
		M	1.000	4.3 × 10 ⁻¹⁰	1.000	2.8 × 10 ⁻¹⁰	1.3 × 10 ⁻¹⁰	8.5 × 10 ⁻¹¹	5.6 × 10 ⁻¹¹	4.6 × 10 ⁻¹¹							

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		<i>e</i> (g)
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Potassium													
K-40	1.28 × 10 ⁹ a	F	1.000	2.4 × 10 ⁻⁸	1.000	1.7 × 10 ⁻⁸	7.5 × 10 ⁻⁹	4.5 × 10 ⁻⁹	2.5 × 10 ⁻⁹	2.1 × 10 ⁻⁹			
K-42	12.4 h	F	1.000	1.6 × 10 ⁻⁹	1.000	1.0 × 10 ⁻⁹	4.4 × 10 ⁻¹⁰	2.6 × 10 ⁻¹⁰	1.5 × 10 ⁻¹⁰	1.2 × 10 ⁻¹⁰			
K-43	22.6 h	F	1.000	1.3 × 10 ⁻⁹	1.000	9.7 × 10 ⁻¹⁰	4.7 × 10 ⁻¹⁰	2.9 × 10 ⁻¹⁰	1.7 × 10 ⁻¹⁰	1.4 × 10 ⁻¹⁰			
K-44	0.369 h	F	1.000	2.2 × 10 ⁻¹⁰	1.000	1.4 × 10 ⁻¹⁰	6.5 × 10 ⁻¹¹	4.0 × 10 ⁻¹¹	2.4 × 10 ⁻¹¹	2.0 × 10 ⁻¹¹			
K-45	0.333 h	F	1.000	1.5 × 10 ⁻¹⁰	1.000	1.0 × 10 ⁻¹⁰	4.8 × 10 ⁻¹¹	3.0 × 10 ⁻¹¹	1.8 × 10 ⁻¹¹	1.5 × 10 ⁻¹¹			
Calcium ^b													
Ca-41	1.40 × 10 ⁵ a	F	0.600	6.7 × 10 ⁻¹⁰	0.300	3.8 × 10 ⁻¹⁰	2.6 × 10 ⁻¹⁰	3.3 × 10 ⁻¹⁰	3.3 × 10 ⁻¹⁰	1.7 × 10 ⁻¹⁰			
		M	0.200	4.2 × 10 ⁻¹⁰	0.100	2.6 × 10 ⁻¹⁰	1.7 × 10 ⁻¹⁰	1.7 × 10 ⁻¹⁰	1.6 × 10 ⁻¹⁰	9.5 × 10 ⁻¹¹			
		S	0.020	6.7 × 10 ⁻¹⁰	0.010	6.0 × 10 ⁻¹⁰	3.8 × 10 ⁻¹⁰	2.4 × 10 ⁻¹⁰	1.9 × 10 ⁻¹⁰	1.8 × 10 ⁻¹⁰			
Ca-45	163 d	F	0.600	5.7 × 10 ⁻⁹	0.300	3.0 × 10 ⁻⁹	1.4 × 10 ⁻⁹	1.0 × 10 ⁻⁹	7.6 × 10 ⁻¹⁰	4.6 × 10 ⁻¹⁰			
		M	0.200	1.2 × 10 ⁻⁸	0.100	8.8 × 10 ⁻⁹	5.3 × 10 ⁻⁹	3.9 × 10 ⁻⁹	3.5 × 10 ⁻⁹	2.7 × 10 ⁻⁹			
		S	0.020	1.5 × 10 ⁻⁸	0.010	1.2 × 10 ⁻⁸	7.2 × 10 ⁻⁹	5.1 × 10 ⁻⁹	4.6 × 10 ⁻⁹	3.7 × 10 ⁻⁹			
Ca-47	4.53 d	F	0.600	4.9 × 10 ⁻⁹	0.300	3.6 × 10 ⁻⁹	1.7 × 10 ⁻⁹	1.1 × 10 ⁻⁹	6.1 × 10 ⁻¹⁰	5.5 × 10 ⁻¹⁰			
		M	0.200	1.0 × 10 ⁻⁸	0.100	7.7 × 10 ⁻⁹	4.2 × 10 ⁻⁹	2.9 × 10 ⁻⁹	2.4 × 10 ⁻⁹	1.9 × 10 ⁻⁹			
		S	0.020	1.2 × 10 ⁻⁸	0.010	8.5 × 10 ⁻⁹	4.6 × 10 ⁻⁹	3.3 × 10 ⁻⁹	2.6 × 10 ⁻⁹	2.1 × 10 ⁻⁹			
Scandium													
Sc-43	3.89 h	S	0.001	9.3 × 10 ⁻¹⁰	1.0 × 10 ⁻⁴	6.7 × 10 ⁻¹⁰	3.3 × 10 ⁻¹⁰	2.2 × 10 ⁻¹⁰	1.4 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰			
Sc-44	3.93 h	S	0.001	1.6 × 10 ⁻⁹	1.0 × 10 ⁻⁴	1.2 × 10 ⁻⁹	5.6 × 10 ⁻¹⁰	3.6 × 10 ⁻¹⁰	2.3 × 10 ⁻¹⁰	1.8 × 10 ⁻¹⁰			
Sc-44m	2.44 d	S	0.001	1.1 × 10 ⁻⁸	1.0 × 10 ⁻⁴	8.4 × 10 ⁻⁹	4.2 × 10 ⁻⁹	2.8 × 10 ⁻⁹	1.7 × 10 ⁻⁹	1.4 × 10 ⁻⁹			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Sc-46	83.8 d	S	0.001	2.8×10^{-8}	1.0×10^{-4}	2.3×10^{-8}	1.4×10^{-8}	9.8×10^{-9}	8.4×10^{-9}	6.8×10^{-9}			
Sc-47	3.35 d	S	0.001	4.0×10^{-9}	1.0×10^{-4}	2.8×10^{-9}	1.5×10^{-9}	1.1×10^{-9}	9.2×10^{-10}	7.3×10^{-10}			
Sc-48	1.82 d	S	0.001	7.8×10^{-9}	1.0×10^{-4}	5.9×10^{-9}	3.1×10^{-9}	2.0×10^{-9}	1.4×10^{-9}	1.1×10^{-9}			
Sc-49	0.956 h	S	0.001	3.9×10^{-10}	1.0×10^{-4}	2.4×10^{-10}	1.1×10^{-10}	7.1×10^{-11}	4.7×10^{-11}	4.0×10^{-11}			
Titanium													
Ti-44	47.3 a	F	0.020	3.1×10^{-7}	0.010	2.6×10^{-7}	1.5×10^{-7}	9.6×10^{-8}	6.6×10^{-8}	6.1×10^{-8}			
		M	0.020	1.7×10^{-7}	0.010	1.5×10^{-7}	9.2×10^{-8}	5.9×10^{-8}	4.6×10^{-8}	4.2×10^{-8}			
		S	0.020	3.2×10^{-7}	0.010	3.1×10^{-7}	2.1×10^{-7}	1.5×10^{-7}	1.3×10^{-7}	1.2×10^{-7}			
Ti-45	3.08 h	F	0.020	4.4×10^{-10}	0.010	3.2×10^{-10}	1.5×10^{-10}	9.1×10^{-11}	5.1×10^{-11}	4.2×10^{-11}			
		M	0.020	7.4×10^{-10}	0.010	5.2×10^{-10}	2.5×10^{-10}	1.6×10^{-10}	1.1×10^{-10}	8.8×10^{-11}			
		S	0.020	7.7×10^{-10}	0.010	5.5×10^{-10}	2.7×10^{-10}	1.7×10^{-10}	1.1×10^{-10}	9.3×10^{-11}			
Vanadium													
V-47	0.543 h	F	0.020	1.8×10^{-10}	0.010	1.2×10^{-10}	5.6×10^{-11}	3.5×10^{-11}	2.1×10^{-11}	1.7×10^{-11}			
		M	0.020	2.8×10^{-10}	0.010	1.9×10^{-10}	8.6×10^{-11}	5.5×10^{-11}	3.5×10^{-11}	2.9×10^{-11}			
V-48	16.2 d	F	0.020	8.4×10^{-9}	0.010	6.4×10^{-9}	3.3×10^{-9}	2.1×10^{-9}	1.3×10^{-9}	1.1×10^{-9}			
		M	0.020	1.4×10^{-8}	0.010	1.1×10^{-8}	6.3×10^{-9}	4.3×10^{-9}	2.9×10^{-9}	2.4×10^{-9}			
V-49	330 d	F	0.020	2.0×10^{-10}	0.010	1.6×10^{-10}	7.7×10^{-11}	4.3×10^{-11}	2.5×10^{-11}	2.1×10^{-11}			
		M	0.020	2.8×10^{-10}	0.010	2.1×10^{-10}	1.1×10^{-10}	6.3×10^{-11}	4.0×10^{-11}	3.4×10^{-11}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Chromium													
Cr-48	23.0 h	F	0.200	7.6 × 10 ^{−10}	0.100	6.0 × 10 ^{−10}	3.1 × 10 ^{−10}	2.0 × 10 ^{−10}	1.2 × 10 ^{−10}	9.9 × 10 ^{−11}			
		M	0.200	1.1 × 10 ^{−9}	0.100	9.1 × 10 ^{−10}	5.1 × 10 ^{−10}	3.4 × 10 ^{−10}	2.5 × 10 ^{−10}	2.0 × 10 ^{−10}			
		S	0.200	1.2 × 10 ^{−9}	0.100	9.8 × 10 ^{−10}	5.5 × 10 ^{−10}	3.7 × 10 ^{−10}	2.8 × 10 ^{−10}	2.2 × 10 ^{−10}			
		F	0.200	1.9 × 10 ^{−10}	0.100	1.3 × 10 ^{−10}	6.0 × 10 ^{−11}	3.7 × 10 ^{−11}	2.2 × 10 ^{−11}	1.9 × 10 ^{−11}			
Cr-49	0.702 h	M	0.200	3.0 × 10 ^{−10}	0.100	2.0 × 10 ^{−10}	9.5 × 10 ^{−11}	6.1 × 10 ^{−11}	4.0 × 10 ^{−11}	3.3 × 10 ^{−11}			
		S	0.200	3.1 × 10 ^{−10}	0.100	2.1 × 10 ^{−10}	9.9 × 10 ^{−11}	6.4 × 10 ^{−11}	4.2 × 10 ^{−11}	3.5 × 10 ^{−11}			
		F	0.200	1.7 × 10 ^{−10}	0.100	1.3 × 10 ^{−10}	6.3 × 10 ^{−11}	4.0 × 10 ^{−11}	2.4 × 10 ^{−11}	2.0 × 10 ^{−11}			
		M	0.200	2.6 × 10 ^{−10}	0.100	1.9 × 10 ^{−10}	1.0 × 10 ^{−10}	6.4 × 10 ^{−11}	3.9 × 10 ^{−11}	3.2 × 10 ^{−11}			
Cr-51	27.7 d	S	0.200	2.6 × 10 ^{−10}	0.100	2.1 × 10 ^{−10}	1.0 × 10 ^{−10}	6.6 × 10 ^{−11}	4.5 × 10 ^{−11}	3.7 × 10 ^{−11}			
		Manganese											
Mn-51	0.770 h	F	0.200	2.5 × 10 ^{−10}	0.100	1.7 × 10 ^{−10}	7.5 × 10 ^{−11}	4.6 × 10 ^{−11}	2.7 × 10 ^{−11}	2.3 × 10 ^{−11}			
		M	0.200	4.0 × 10 ^{−10}	0.100	2.7 × 10 ^{−10}	1.2 × 10 ^{−10}	7.8 × 10 ^{−11}	5.0 × 10 ^{−11}	4.1 × 10 ^{−11}			
Mn-52	5.59 d	F	0.200	7.0 × 10 ^{−9}	0.100	5.5 × 10 ^{−9}	2.9 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	9.4 × 10 ^{−10}			
		M	0.200	8.6 × 10 ^{−9}	0.100	6.8 × 10 ^{−9}	3.7 × 10 ^{−9}	2.4 × 10 ^{−9}	1.7 × 10 ^{−9}	1.4 × 10 ^{−9}			
Mn-52m	0.352 h	F	0.200	1.9 × 10 ^{−10}	0.100	1.3 × 10 ^{−10}	6.1 × 10 ^{−11}	3.8 × 10 ^{−11}	2.2 × 10 ^{−11}	1.9 × 10 ^{−11}			
		M	0.200	2.8 × 10 ^{−10}	0.100	1.9 × 10 ^{−10}	8.7 × 10 ^{−11}	5.5 × 10 ^{−11}	3.4 × 10 ^{−11}	2.9 × 10 ^{−11}			
Mn-53	3.70 × 10 ⁶ a	F	0.200	3.2 × 10 ^{−10}	0.100	2.2 × 10 ^{−10}	1.1 × 10 ^{−10}	6.0 × 10 ^{−11}	3.4 × 10 ^{−11}	2.9 × 10 ^{−11}			
		M	0.200	4.6 × 10 ^{−10}	0.100	3.4 × 10 ^{−10}	1.7 × 10 ^{−10}	1.0 × 10 ^{−10}	6.4 × 10 ^{−11}	5.4 × 10 ^{−11}			
Mn-54	312 d	F	0.200	5.2 × 10 ^{−9}	0.100	4.1 × 10 ^{−9}	2.2 × 10 ^{−9}	1.5 × 10 ^{−9}	9.9 × 10 ^{−10}	8.5 × 10 ^{−10}			
		M	0.200	7.5 × 10 ^{−9}	0.100	6.2 × 10 ^{−9}	3.8 × 10 ^{−9}	2.4 × 10 ^{−9}	1.9 × 10 ^{−9}	1.5 × 10 ^{−9}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)			
Mn-56	2.58 h	F	0.200	6.9 × 10 ^{−10}	0.100	4.9 × 10 ^{−10}	2.3 × 10 ^{−10}	1.4 × 10 ^{−10}	7.8 × 10 ^{−11}	6.4 × 10 ^{−11}			
		M	0.200	1.1 × 10 ^{−9}	0.100	7.8 × 10 ^{−10}	3.7 × 10 ^{−10}	2.4 × 10 ^{−10}	1.5 × 10 ^{−10}	1.2 × 10 ^{−10}			
Iron ^c													
Fe-52	8.28 h	F	0.600	5.2 × 10 ^{−9}	0.100	3.6 × 10 ^{−9}	1.5 × 10 ^{−9}	8.9 × 10 ^{−10}	4.9 × 10 ^{−10}	3.9 × 10 ^{−10}			
		M	0.200	5.8 × 10 ^{−9}	0.100	4.1 × 10 ^{−9}	1.9 × 10 ^{−9}	1.2 × 10 ^{−9}	7.4 × 10 ^{−10}	6.0 × 10 ^{−10}			
		S	0.020	6.0 × 10 ^{−9}	0.010	4.2 × 10 ^{−9}	2.0 × 10 ^{−9}	1.3 × 10 ^{−9}	7.7 × 10 ^{−10}	6.3 × 10 ^{−10}			
Fe-55	2.70 a	F	0.600	4.2 × 10 ^{−9}	0.100	3.2 × 10 ^{−9}	2.2 × 10 ^{−9}	1.4 × 10 ^{−9}	9.4 × 10 ^{−10}	7.7 × 10 ^{−10}			
		M	0.200	1.9 × 10 ^{−9}	0.100	1.4 × 10 ^{−9}	9.9 × 10 ^{−10}	6.2 × 10 ^{−10}	4.4 × 10 ^{−10}	3.8 × 10 ^{−10}			
		S	0.020	1.0 × 10 ^{−9}	0.010	8.5 × 10 ^{−10}	5.0 × 10 ^{−10}	2.9 × 10 ^{−10}	2.0 × 10 ^{−10}	1.8 × 10 ^{−10}			
Fe-59	44.5 d	F	0.600	2.1 × 10 ^{−8}	0.100	1.3 × 10 ^{−8}	7.1 × 10 ^{−9}	4.2 × 10 ^{−9}	2.6 × 10 ^{−9}	2.2 × 10 ^{−9}			
		M	0.200	1.8 × 10 ^{−8}	0.100	1.3 × 10 ^{−8}	7.9 × 10 ^{−9}	5.5 × 10 ^{−9}	4.6 × 10 ^{−9}	3.7 × 10 ^{−9}			
		S	0.020	1.7 × 10 ^{−8}	0.010	1.3 × 10 ^{−8}	8.1 × 10 ^{−9}	5.8 × 10 ^{−9}	5.1 × 10 ^{−9}	4.0 × 10 ^{−9}			
Fe-60	1.00 × 10 ⁵ a	F	0.600	4.4 × 10 ^{−7}	0.100	3.9 × 10 ^{−7}	3.5 × 10 ^{−7}	3.2 × 10 ^{−7}	2.9 × 10 ^{−7}	2.8 × 10 ^{−7}			
		M	0.200	2.0 × 10 ^{−7}	0.100	1.7 × 10 ^{−7}	1.6 × 10 ^{−7}	1.4 × 10 ^{−7}	1.4 × 10 ^{−7}	1.4 × 10 ^{−7}			
		S	0.020	9.3 × 10 ^{−8}	0.010	8.8 × 10 ^{−8}	6.7 × 10 ^{−8}	5.2 × 10 ^{−8}	4.9 × 10 ^{−8}	4.9 × 10 ^{−8}			
Cobalt ^d													
Co-55	17.5 h	F	0.600	2.2 × 10 ^{−9}	0.100	1.8 × 10 ^{−9}	9.0 × 10 ^{−10}	5.5 × 10 ^{−10}	3.1 × 10 ^{−10}	2.7 × 10 ^{−10}			
		M	0.200	4.1 × 10 ^{−9}	0.100	3.1 × 10 ^{−9}	1.5 × 10 ^{−9}	9.8 × 10 ^{−10}	6.1 × 10 ^{−10}	5.0 × 10 ^{−10}			
		S	0.020	4.6 × 10 ^{−9}	0.010	3.3 × 10 ^{−9}	1.6 × 10 ^{−9}	1.1 × 10 ^{−9}	6.6 × 10 ^{−10}	5.3 × 10 ^{−10}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Co-56	78.7 d	F	0.600	1.4×10^{-8}	0.100	1.0×10^{-8}	5.5×10^{-9}	3.5×10^{-9}	3.5×10^{-9}	2.2×10^{-9}	2.2×10^{-9}	1.8×10^{-9}		
		M	0.200	2.5×10^{-8}	0.100	2.1×10^{-8}	1.1×10^{-8}	7.4×10^{-9}	7.4×10^{-9}	5.8×10^{-9}	5.8×10^{-9}	4.8×10^{-9}		
		S	0.020	2.9×10^{-8}	0.010	2.5×10^{-8}	1.5×10^{-8}	1.0×10^{-8}	1.0×10^{-8}	8.0×10^{-9}	8.0×10^{-9}	6.7×10^{-9}		
Co-57	271 d	F	0.600	1.5×10^{-9}	0.100	1.1×10^{-9}	5.6×10^{-10}	3.7×10^{-10}	3.7×10^{-10}	2.3×10^{-10}	2.3×10^{-10}	1.9×10^{-10}		
		M	0.200	2.8×10^{-9}	0.100	2.2×10^{-9}	1.3×10^{-9}	8.5×10^{-10}	8.5×10^{-10}	6.7×10^{-10}	6.7×10^{-10}	5.5×10^{-10}		
		S	0.020	4.4×10^{-9}	0.010	3.7×10^{-9}	2.3×10^{-9}	1.5×10^{-9}	1.5×10^{-9}	1.2×10^{-9}	1.2×10^{-9}	1.0×10^{-9}		
Co-58	70.8 d	F	0.600	4.0×10^{-9}	0.100	3.0×10^{-9}	1.6×10^{-9}	1.0×10^{-9}	1.0×10^{-9}	6.4×10^{-10}	6.4×10^{-10}	5.3×10^{-10}		
		M	0.200	7.3×10^{-9}	0.100	6.5×10^{-9}	3.5×10^{-9}	2.4×10^{-9}	2.4×10^{-9}	2.0×10^{-9}	2.0×10^{-9}	1.6×10^{-9}		
		S	0.020	9.0×10^{-9}	0.010	7.5×10^{-9}	4.5×10^{-9}	3.1×10^{-9}	3.1×10^{-9}	2.6×10^{-9}	2.6×10^{-9}	2.1×10^{-9}		
Co-58m	9.15 h	F	0.600	4.8×10^{-11}	0.100	3.6×10^{-11}	1.7×10^{-11}	1.1×10^{-11}	1.1×10^{-11}	5.9×10^{-12}	5.9×10^{-12}	5.2×10^{-12}		
		M	0.200	1.1×10^{-10}	0.100	7.6×10^{-11}	3.8×10^{-11}	2.4×10^{-11}	2.4×10^{-11}	1.6×10^{-11}	1.6×10^{-11}	1.3×10^{-11}		
		S	0.020	1.3×10^{-10}	0.010	9.0×10^{-11}	4.5×10^{-11}	3.0×10^{-11}	3.0×10^{-11}	2.0×10^{-11}	2.0×10^{-11}	1.7×10^{-11}		
Co-60	5.27 a	F	0.600	3.0×10^{-8}	0.100	2.3×10^{-8}	1.4×10^{-8}	8.9×10^{-9}	8.9×10^{-9}	6.1×10^{-9}	6.1×10^{-9}	5.2×10^{-9}		
		M	0.200	4.2×10^{-8}	0.100	3.4×10^{-8}	2.1×10^{-8}	1.5×10^{-8}	1.5×10^{-8}	1.2×10^{-8}	1.2×10^{-8}	1.0×10^{-8}		
		S	0.020	9.2×10^{-8}	0.010	8.6×10^{-8}	5.9×10^{-8}	4.0×10^{-8}	4.0×10^{-8}	3.4×10^{-8}	3.4×10^{-8}	3.1×10^{-8}		
Co-60m	0.174 h	F	0.600	4.4×10^{-12}	0.100	2.8×10^{-12}	1.5×10^{-12}	1.0×10^{-12}	1.0×10^{-12}	8.3×10^{-13}	8.3×10^{-13}	6.9×10^{-13}		
		M	0.200	7.1×10^{-12}	0.100	4.7×10^{-12}	2.7×10^{-12}	1.8×10^{-12}	1.8×10^{-12}	1.5×10^{-12}	1.5×10^{-12}	1.2×10^{-12}		
		S	0.020	7.6×10^{-12}	0.010	5.1×10^{-12}	2.9×10^{-12}	2.0×10^{-12}	2.0×10^{-12}	1.7×10^{-12}	1.7×10^{-12}	1.4×10^{-12}		
Co-61	1.65 h	F	0.600	2.1×10^{-10}	0.100	1.4×10^{-10}	6.0×10^{-11}	3.8×10^{-11}	3.8×10^{-11}	2.2×10^{-11}	2.2×10^{-11}	1.9×10^{-11}		
		M	0.200	4.0×10^{-10}	0.100	2.7×10^{-10}	1.2×10^{-10}	8.2×10^{-11}	8.2×10^{-11}	5.7×10^{-11}	5.7×10^{-11}	4.7×10^{-11}		
		S	0.020	4.3×10^{-10}	0.010	2.8×10^{-10}	1.3×10^{-10}	8.8×10^{-11}	8.8×10^{-11}	6.1×10^{-11}	6.1×10^{-11}	5.1×10^{-11}		

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Co-62m	0.232 h	F	0.600	1.4×10^{-10}	0.100	9.5×10^{-11}	4.5×10^{-11}	2.8×10^{-11}	1.7×10^{-11}	1.4×10^{-11}							
		M	0.200	1.9×10^{-10}	0.100	1.3×10^{-10}	6.1×10^{-11}	3.8×10^{-11}	2.4×10^{-11}	2.0×10^{-11}							
		S	0.020	2.0×10^{-10}	0.010	1.3×10^{-10}	6.3×10^{-11}	4.0×10^{-11}	2.5×10^{-11}	2.1×10^{-11}							
Nickel																	
Ni-56	6.10 d	F	0.100	3.3×10^{-9}	0.050	2.8×10^{-9}	1.5×10^{-9}	9.3×10^{-10}	5.8×10^{-10}	4.9×10^{-10}							
		M	0.100	4.9×10^{-9}	0.050	4.1×10^{-9}	2.3×10^{-9}	1.5×10^{-9}	1.1×10^{-9}	8.7×10^{-10}							
		S	0.020	5.5×10^{-9}	0.010	4.6×10^{-9}	2.7×10^{-9}	1.8×10^{-9}	1.3×10^{-9}	1.0×10^{-9}							
Ni-57	1.50 d	F	0.100	2.2×10^{-9}	0.050	1.8×10^{-9}	8.9×10^{-10}	5.5×10^{-10}	3.1×10^{-10}	2.5×10^{-10}							
		M	0.100	3.6×10^{-9}	0.050	2.8×10^{-9}	1.5×10^{-9}	9.5×10^{-10}	6.2×10^{-10}	5.0×10^{-10}							
		S	0.020	3.9×10^{-9}	0.010	3.0×10^{-9}	1.5×10^{-9}	1.0×10^{-9}	6.6×10^{-10}	5.3×10^{-10}							
Ni-59	7.50×10^4 a	F	0.100	9.6×10^{-10}	0.050	8.1×10^{-10}	4.5×10^{-10}	2.8×10^{-10}	1.9×10^{-10}	1.8×10^{-10}							
		M	0.100	7.9×10^{-10}	0.050	6.2×10^{-10}	3.4×10^{-10}	2.1×10^{-10}	1.4×10^{-10}	1.3×10^{-10}							
		S	0.020	1.7×10^{-9}	0.010	1.5×10^{-9}	9.5×10^{-10}	5.9×10^{-10}	4.6×10^{-10}	4.4×10^{-10}							
Ni-63	96.0 a	F	0.100	2.3×10^{-9}	0.050	2.0×10^{-9}	1.1×10^{-9}	6.7×10^{-10}	4.6×10^{-10}	4.4×10^{-10}							
		M	0.100	2.5×10^{-9}	0.050	1.9×10^{-9}	1.1×10^{-9}	7.0×10^{-10}	5.3×10^{-10}	4.8×10^{-10}							
		S	0.020	4.8×10^{-9}	0.010	4.3×10^{-9}	2.7×10^{-9}	1.7×10^{-9}	1.3×10^{-9}	1.3×10^{-9}							
Ni-65	2.52 h	F	0.100	4.4×10^{-10}	0.050	3.0×10^{-10}	1.4×10^{-10}	8.5×10^{-11}	4.9×10^{-11}	4.1×10^{-11}							
		M	0.100	7.7×10^{-10}	0.050	5.2×10^{-10}	2.4×10^{-10}	1.6×10^{-10}	1.0×10^{-10}	8.5×10^{-11}							
		S	0.020	8.1×10^{-10}	0.010	5.5×10^{-10}	2.6×10^{-10}	1.7×10^{-10}	1.1×10^{-10}	9.0×10^{-11}							

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a	
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)
Ni-66	2.27 d	F	0.100	5.7 × 10 ^{−9}	0.050	3.8 × 10 ^{−9}	1.6 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	5.1 × 10 ^{−10}	5.1 × 10 ^{−10}	4.2 × 10 ^{−10}						
		M	0.100	1.3 × 10 ^{−8}	0.050	9.4 × 10 ^{−9}	4.5 × 10 ^{−9}	2.9 × 10 ^{−9}	2.9 × 10 ^{−9}	2.0 × 10 ^{−9}	2.0 × 10 ^{−9}	1.6 × 10 ^{−9}						
		S	0.020	1.5 × 10 ^{−8}	0.010	1.0 × 10 ^{−8}	5.0 × 10 ^{−9}	3.2 × 10 ^{−9}	3.2 × 10 ^{−9}	2.2 × 10 ^{−9}	2.2 × 10 ^{−9}	1.8 × 10 ^{−9}						
Copper																		
Cu-60	0.387 h	F	1.000	2.1 × 10 ^{−10}	0.500	1.6 × 10 ^{−10}	7.5 × 10 ^{−11}	4.6 × 10 ^{−11}	4.6 × 10 ^{−11}	2.8 × 10 ^{−11}	2.8 × 10 ^{−11}	2.3 × 10 ^{−11}						
		M	1.000	3.0 × 10 ^{−10}	0.500	2.2 × 10 ^{−10}	1.0 × 10 ^{−10}	6.5 × 10 ^{−11}	6.5 × 10 ^{−11}	4.0 × 10 ^{−11}	4.0 × 10 ^{−11}	3.3 × 10 ^{−11}						
		S	1.000	3.1 × 10 ^{−10}	0.500	2.2 × 10 ^{−10}	1.1 × 10 ^{−10}	6.7 × 10 ^{−11}	6.7 × 10 ^{−11}	4.2 × 10 ^{−11}	4.2 × 10 ^{−11}	3.4 × 10 ^{−11}						
Cu-61	3.41 h	F	1.000	3.1 × 10 ^{−10}	0.500	2.7 × 10 ^{−10}	1.3 × 10 ^{−10}	7.9 × 10 ^{−11}	7.9 × 10 ^{−11}	4.5 × 10 ^{−11}	4.5 × 10 ^{−11}	3.7 × 10 ^{−11}						
		M	1.000	4.9 × 10 ^{−10}	0.500	4.4 × 10 ^{−10}	2.1 × 10 ^{−10}	1.4 × 10 ^{−10}	1.4 × 10 ^{−10}	9.1 × 10 ^{−11}	9.1 × 10 ^{−11}	7.4 × 10 ^{−11}						
		S	1.000	5.1 × 10 ^{−10}	0.500	4.5 × 10 ^{−10}	2.2 × 10 ^{−10}	1.4 × 10 ^{−10}	1.4 × 10 ^{−10}	9.6 × 10 ^{−11}	9.6 × 10 ^{−11}	7.8 × 10 ^{−11}						
Cu-64	12.7 h	F	1.000	2.8 × 10 ^{−10}	0.500	2.7 × 10 ^{−10}	1.2 × 10 ^{−10}	7.6 × 10 ^{−11}	7.6 × 10 ^{−11}	4.2 × 10 ^{−11}	4.2 × 10 ^{−11}	3.5 × 10 ^{−11}						
		M	1.000	5.5 × 10 ^{−10}	0.500	5.4 × 10 ^{−10}	2.7 × 10 ^{−10}	1.9 × 10 ^{−10}	1.9 × 10 ^{−10}	1.4 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}						
		S	1.000	5.8 × 10 ^{−10}	0.500	5.7 × 10 ^{−10}	2.9 × 10 ^{−10}	2.0 × 10 ^{−10}	2.0 × 10 ^{−10}	1.3 × 10 ^{−10}	1.3 × 10 ^{−10}	1.2 × 10 ^{−10}						
Cu-67	2.58 d	F	1.000	9.5 × 10 ^{−10}	0.500	8.0 × 10 ^{−10}	3.5 × 10 ^{−10}	2.2 × 10 ^{−10}	2.2 × 10 ^{−10}	1.2 × 10 ^{−10}	1.2 × 10 ^{−10}	1.0 × 10 ^{−10}						
		M	1.000	2.3 × 10 ^{−9}	0.500	2.0 × 10 ^{−9}	1.1 × 10 ^{−9}	8.1 × 10 ^{−10}	8.1 × 10 ^{−10}	6.9 × 10 ^{−10}	6.9 × 10 ^{−10}	5.5 × 10 ^{−10}						
		S	1.000	2.5 × 10 ^{−9}	0.500	2.1 × 10 ^{−9}	1.2 × 10 ^{−9}	8.9 × 10 ^{−10}	8.9 × 10 ^{−10}	7.7 × 10 ^{−10}	7.7 × 10 ^{−10}	6.1 × 10 ^{−10}						
Zinc																		
Zn-62	9.26 h	F	1.000	1.7 × 10 ^{−9}	0.500	1.7 × 10 ^{−9}	7.7 × 10 ^{−10}	4.6 × 10 ^{−10}	4.6 × 10 ^{−10}	2.5 × 10 ^{−10}	2.5 × 10 ^{−10}	2.0 × 10 ^{−10}						
		M	0.200	4.5 × 10 ^{−9}	0.100	3.5 × 10 ^{−9}	1.6 × 10 ^{−9}	1.0 × 10 ^{−9}	1.0 × 10 ^{−9}	6.0 × 10 ^{−10}	6.0 × 10 ^{−10}	5.0 × 10 ^{−10}						
		S	0.020	5.1 × 10 ^{−9}	0.010	3.4 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	1.1 × 10 ^{−9}	6.6 × 10 ^{−10}	6.6 × 10 ^{−10}	5.5 × 10 ^{−10}						

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$
Zn-63	0.635 h	F	1.000	2.1×10^{-10}	0.500	1.4×10^{-10}	6.5×10^{-11}	4.0×10^{-11}	2.4×10^{-11}	2.4×10^{-11}	2.0×10^{-11}							
		M	0.200	3.4×10^{-10}	0.100	2.3×10^{-10}	1.0×10^{-10}	6.6×10^{-11}	4.2×10^{-11}	4.2×10^{-11}	3.5×10^{-11}							
		S	0.020	3.6×10^{-10}	0.010	2.4×10^{-10}	1.1×10^{-10}	6.9×10^{-11}	4.4×10^{-11}	4.4×10^{-11}	3.7×10^{-11}							
Zn-65	244 d	F	1.000	1.5×10^{-8}	0.500	1.0×10^{-8}	5.7×10^{-9}	3.8×10^{-9}	2.5×10^{-9}	2.2×10^{-9}								
		M	0.200	8.5×10^{-9}	0.100	6.5×10^{-9}	3.7×10^{-9}	2.4×10^{-9}	1.9×10^{-9}	1.9×10^{-9}	1.6×10^{-9}							
		S	0.020	7.6×10^{-9}	0.010	6.7×10^{-9}	4.4×10^{-9}	2.9×10^{-9}	2.4×10^{-9}	2.4×10^{-9}	2.0×10^{-9}							
Zn-69	0.950 h	F	1.000	1.1×10^{-10}	0.500	7.4×10^{-11}	3.2×10^{-11}	2.1×10^{-11}	1.2×10^{-11}	1.1×10^{-11}								
		M	0.200	2.2×10^{-10}	0.100	1.4×10^{-10}	6.5×10^{-11}	4.4×10^{-11}	3.1×10^{-11}	3.1×10^{-11}	2.6×10^{-11}							
		S	0.020	2.3×10^{-10}	0.010	1.5×10^{-10}	6.9×10^{-11}	4.7×10^{-11}	3.4×10^{-11}	3.4×10^{-11}	2.8×10^{-11}							
Zn-69m	13.8 h	F	1.000	6.6×10^{-10}	0.500	6.7×10^{-10}	3.0×10^{-10}	1.8×10^{-10}	9.9×10^{-11}	8.2×10^{-11}								
		M	0.200	2.1×10^{-9}	0.100	1.5×10^{-9}	7.5×10^{-10}	5.0×10^{-10}	3.0×10^{-10}	3.0×10^{-10}	2.4×10^{-10}							
		S	0.020	2.2×10^{-9}	0.010	1.7×10^{-9}	8.2×10^{-10}	5.4×10^{-10}	3.3×10^{-10}	3.3×10^{-10}	2.7×10^{-10}							
Zn-71m	3.92 h	F	1.000	6.2×10^{-10}	0.500	5.5×10^{-10}	2.6×10^{-10}	1.6×10^{-10}	9.1×10^{-11}	7.4×10^{-11}								
		M	0.200	1.3×10^{-9}	0.100	9.4×10^{-10}	4.6×10^{-10}	2.9×10^{-10}	1.9×10^{-10}	1.9×10^{-10}	1.5×10^{-10}							
		S	0.020	1.4×10^{-9}	0.010	1.0×10^{-9}	4.9×10^{-10}	3.1×10^{-10}	2.0×10^{-10}	2.0×10^{-10}	1.6×10^{-10}							
Zn-72	1.94 d	F	1.000	4.3×10^{-9}	0.500	3.5×10^{-9}	1.7×10^{-9}	1.0×10^{-9}	5.9×10^{-10}	4.9×10^{-10}								
		M	0.200	8.8×10^{-9}	0.100	6.5×10^{-9}	3.4×10^{-9}	2.3×10^{-9}	1.5×10^{-9}	1.5×10^{-9}	1.2×10^{-9}							
		S	0.020	9.7×10^{-9}	0.010	7.0×10^{-9}	3.6×10^{-9}	2.4×10^{-9}	1.6×10^{-9}	1.6×10^{-9}	1.3×10^{-9}							
Gallium																		
Ga-65	0.253 h	F	0.010	1.1×10^{-10}	0.001	7.3×10^{-11}	3.4×10^{-11}	2.1×10^{-11}	1.3×10^{-11}	1.3×10^{-11}	1.1×10^{-11}							
		M	0.010	1.6×10^{-10}	0.001	1.1×10^{-10}	4.8×10^{-11}	3.1×10^{-11}	2.0×10^{-11}	2.0×10^{-11}	1.7×10^{-11}							

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a								2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)								<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)		
Ga-66	9.40 h	F	0.010	2.8 × 10 ^{−9}	0.001	2.0 × 10 ^{−9}	9.2 × 10 ^{−10}	5.7 × 10 ^{−10}	3.0 × 10 ^{−10}	2.5 × 10 ^{−10}									
		M	0.010	4.5 × 10 ^{−9}	0.001	3.1 × 10 ^{−9}	1.5 × 10 ^{−9}	9.2 × 10 ^{−10}	5.3 × 10 ^{−10}	4.4 × 10 ^{−10}									
Ga-67	3.26 d	F	0.010	6.4 × 10 ^{−10}	0.001	4.6 × 10 ^{−10}	2.2 × 10 ^{−10}	1.4 × 10 ^{−10}	7.7 × 10 ^{−11}	6.4 × 10 ^{−11}									
		M	0.010	1.4 × 10 ^{−9}	0.001	1.0 × 10 ^{−9}	5.0 × 10 ^{−10}	3.6 × 10 ^{−10}	3.0 × 10 ^{−10}	2.4 × 10 ^{−10}									
Ga-68	1.13 h	F	0.010	2.9 × 10 ^{−10}	0.001	1.9 × 10 ^{−10}	8.8 × 10 ^{−11}	5.4 × 10 ^{−11}	3.1 × 10 ^{−11}	2.6 × 10 ^{−11}									
		M	0.010	4.6 × 10 ^{−10}	0.001	3.1 × 10 ^{−10}	1.4 × 10 ^{−10}	9.2 × 10 ^{−11}	5.9 × 10 ^{−11}	4.9 × 10 ^{−11}									
Ga-70	0.353 h	F	0.010	9.5 × 10 ^{−11}	0.001	6.0 × 10 ^{−11}	2.6 × 10 ^{−11}	1.6 × 10 ^{−11}	1.0 × 10 ^{−11}	8.8 × 10 ^{−12}									
		M	0.010	1.5 × 10 ^{−10}	0.001	9.6 × 10 ^{−11}	4.3 × 10 ^{−11}	2.8 × 10 ^{−11}	1.8 × 10 ^{−11}	1.6 × 10 ^{−11}									
Ga-72	14.1 h	F	0.010	2.9 × 10 ^{−9}	0.001	2.2 × 10 ^{−9}	1.0 × 10 ^{−9}	6.4 × 10 ^{−10}	3.6 × 10 ^{−10}	2.9 × 10 ^{−10}									
		M	0.010	4.5 × 10 ^{−9}	0.001	3.3 × 10 ^{−9}	1.6 × 10 ^{−9}	1.0 × 10 ^{−9}	6.5 × 10 ^{−10}	5.3 × 10 ^{−10}									
Ga-73	4.91 h	F	0.010	6.7 × 10 ^{−10}	0.001	4.5 × 10 ^{−10}	2.0 × 10 ^{−10}	1.2 × 10 ^{−10}	6.4 × 10 ^{−11}	5.4 × 10 ^{−11}									
		M	0.010	1.2 × 10 ^{−9}	0.001	8.4 × 10 ^{−10}	4.0 × 10 ^{−10}	2.6 × 10 ^{−10}	1.7 × 10 ^{−10}	1.4 × 10 ^{−10}									
Germanium																			
Ge-66	2.27 h	F	1.000	4.5 × 10 ^{−10}	1.000	3.5 × 10 ^{−10}	1.8 × 10 ^{−10}	1.1 × 10 ^{−10}	6.7 × 10 ^{−11}	5.4 × 10 ^{−11}									
		M	1.000	6.4 × 10 ^{−10}	1.000	4.8 × 10 ^{−10}	2.5 × 10 ^{−10}	1.6 × 10 ^{−10}	1.1 × 10 ^{−10}	9.1 × 10 ^{−11}									
Ge-67	0.312 h	F	1.000	1.7 × 10 ^{−10}	1.000	1.1 × 10 ^{−10}	4.9 × 10 ^{−11}	3.1 × 10 ^{−11}	1.8 × 10 ^{−11}	1.5 × 10 ^{−11}									
		M	1.000	2.5 × 10 ^{−10}	1.000	1.6 × 10 ^{−10}	7.3 × 10 ^{−11}	4.6 × 10 ^{−11}	2.9 × 10 ^{−11}	2.5 × 10 ^{−11}									
Ge-68	288 d	F	1.000	5.4 × 10 ^{−9}	1.000	3.8 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	6.3 × 10 ^{−10}	5.2 × 10 ^{−10}									
		M	1.000	6.0 × 10 ^{−8}	1.000	5.0 × 10 ^{−8}	3.0 × 10 ^{−8}	2.0 × 10 ^{−8}	1.6 × 10 ^{−8}	1.4 × 10 ^{−8}									
Ge-69	1.63 d	F	1.000	1.2 × 10 ^{−9}	1.000	9.0 × 10 ^{−10}	4.6 × 10 ^{−10}	2.8 × 10 ^{−10}	1.7 × 10 ^{−10}	1.3 × 10 ^{−10}									
		M	1.000	1.8 × 10 ^{−9}	1.000	1.4 × 10 ^{−9}	7.4 × 10 ^{−10}	4.9 × 10 ^{−10}	3.6 × 10 ^{−10}	2.9 × 10 ^{−10}									

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)						<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)		
Ge-71	11.8 d	F	1.000	6.0 × 10 ^{−11}	1.000	4.3 × 10 ^{−11}	2.0 × 10 ^{−11}	1.1 × 10 ^{−11}	6.1 × 10 ^{−12}	4.8 × 10 ^{−12}							
		M	1.000	1.2 × 10 ^{−10}	1.000	8.6 × 10 ^{−11}	4.1 × 10 ^{−11}	2.4 × 10 ^{−11}	1.3 × 10 ^{−11}	1.1 × 10 ^{−11}							
Ge-75	1.38 h	F	1.000	1.6 × 10 ^{−10}	1.000	1.0 × 10 ^{−10}	4.3 × 10 ^{−11}	2.8 × 10 ^{−11}	1.7 × 10 ^{−11}	1.5 × 10 ^{−11}							
		M	1.000	2.9 × 10 ^{−10}	1.000	1.9 × 10 ^{−10}	8.9 × 10 ^{−11}	6.1 × 10 ^{−11}	4.4 × 10 ^{−11}	3.6 × 10 ^{−11}							
Ge-77	11.3 h	F	1.000	1.3 × 10 ^{−9}	1.000	9.5 × 10 ^{−10}	4.7 × 10 ^{−10}	2.9 × 10 ^{−10}	1.7 × 10 ^{−10}	1.4 × 10 ^{−10}							
		M	1.000	2.3 × 10 ^{−9}	1.000	1.7 × 10 ^{−9}	8.8 × 10 ^{−10}	6.0 × 10 ^{−10}	4.5 × 10 ^{−10}	3.7 × 10 ^{−10}							
Ge-78	1.45 h	F	1.000	4.3 × 10 ^{−10}	1.000	2.9 × 10 ^{−10}	1.4 × 10 ^{−10}	8.9 × 10 ^{−11}	5.5 × 10 ^{−11}	4.5 × 10 ^{−11}							
		M	1.000	7.3 × 10 ^{−10}	1.000	5.0 × 10 ^{−10}	2.5 × 10 ^{−10}	1.6 × 10 ^{−10}	1.2 × 10 ^{−10}	9.5 × 10 ^{−11}							
Arsenic																	
As-69	0.253 h	M	1.000	2.1 × 10 ^{−10}	0.500	1.4 × 10 ^{−10}	6.3 × 10 ^{−11}	4.0 × 10 ^{−11}	2.5 × 10 ^{−11}	2.1 × 10 ^{−11}							
As-70	0.876 h	M	1.000	5.7 × 10 ^{−10}	0.500	4.3 × 10 ^{−10}	2.1 × 10 ^{−10}	1.3 × 10 ^{−10}	8.3 × 10 ^{−11}	6.7 × 10 ^{−11}							
As-71	2.70 d	M	1.000	2.2 × 10 ^{−9}	0.500	1.9 × 10 ^{−9}	1.0 × 10 ^{−9}	6.8 × 10 ^{−10}	5.0 × 10 ^{−10}	4.0 × 10 ^{−10}							
As-72	1.08 d	M	1.000	5.9 × 10 ^{−9}	0.500	5.7 × 10 ^{−9}	2.7 × 10 ^{−9}	1.7 × 10 ^{−9}	1.1 × 10 ^{−9}	9.0 × 10 ^{−10}							
As-73	80.3 d	M	1.000	5.4 × 10 ^{−9}	0.500	4.0 × 10 ^{−9}	2.3 × 10 ^{−9}	1.5 × 10 ^{−9}	1.2 × 10 ^{−9}	1.0 × 10 ^{−9}							
As-74	17.8 d	M	1.000	1.1 × 10 ^{−8}	0.500	8.4 × 10 ^{−9}	4.7 × 10 ^{−9}	3.3 × 10 ^{−9}	2.6 × 10 ^{−9}	2.1 × 10 ^{−9}							
As-76	1.10 d	M	1.000	5.1 × 10 ^{−9}	0.500	4.6 × 10 ^{−9}	2.2 × 10 ^{−9}	1.4 × 10 ^{−9}	8.8 × 10 ^{−10}	7.4 × 10 ^{−10}							
As-77	1.62 d	M	1.000	2.2 × 10 ^{−9}	0.500	1.7 × 10 ^{−9}	8.9 × 10 ^{−10}	6.2 × 10 ^{−10}	5.0 × 10 ^{−10}	3.9 × 10 ^{−10}							
As-78	1.51 h	M	1.000	8.0 × 10 ^{−10}	0.500	5.8 × 10 ^{−10}	2.7 × 10 ^{−10}	1.7 × 10 ^{−10}	1.1 × 10 ^{−10}	8.9 × 10 ^{−11}							

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Selenium													
Se-70	0.683 h	F	1.000	3.9 × 10 ^{−10}	0.800	3.0 × 10 ^{−10}	1.5 × 10 ^{−10}	9.0 × 10 ^{−11}	5.1 × 10 ^{−11}			4.2 × 10 ^{−11}	
		M	0.200	6.5 × 10 ^{−10}	0.100	4.7 × 10 ^{−10}	2.3 × 10 ^{−10}	1.4 × 10 ^{−10}	8.9 × 10 ^{−11}			7.3 × 10 ^{−11}	
		S	0.020	6.8 × 10 ^{−10}	0.010	4.8 × 10 ^{−10}	2.3 × 10 ^{−10}	1.5 × 10 ^{−10}	9.4 × 10 ^{−11}			7.6 × 10 ^{−11}	
Se-73	7.15 h	F	1.000	7.7 × 10 ^{−10}	0.800	6.5 × 10 ^{−10}	3.3 × 10 ^{−10}	2.1 × 10 ^{−10}	1.0 × 10 ^{−10}			8.0 × 10 ^{−11}	
		M	0.200	1.6 × 10 ^{−9}	0.100	1.2 × 10 ^{−9}	5.9 × 10 ^{−10}	3.8 × 10 ^{−10}	2.4 × 10 ^{−10}			1.9 × 10 ^{−10}	
		S	0.020	1.8 × 10 ^{−9}	0.010	1.3 × 10 ^{−9}	6.3 × 10 ^{−10}	4.0 × 10 ^{−10}	2.6 × 10 ^{−10}			2.1 × 10 ^{−10}	
Se-73m	0.650 h	F	1.000	9.3 × 10 ^{−11}	0.800	7.2 × 10 ^{−11}	3.5 × 10 ^{−11}	2.3 × 10 ^{−11}	1.1 × 10 ^{−11}			9.2 × 10 ^{−12}	
		M	0.200	1.8 × 10 ^{−10}	0.100	1.3 × 10 ^{−10}	6.1 × 10 ^{−11}	3.9 × 10 ^{−11}	2.5 × 10 ^{−11}			2.0 × 10 ^{−11}	
		S	0.020	1.9 × 10 ^{−10}	0.010	1.3 × 10 ^{−10}	6.5 × 10 ^{−11}	4.1 × 10 ^{−11}	2.6 × 10 ^{−11}			2.2 × 10 ^{−11}	
Se-75	120 d	F	1.000	7.8 × 10 ^{−9}	0.800	6.0 × 10 ^{−9}	3.4 × 10 ^{−9}	2.5 × 10 ^{−9}	1.2 × 10 ^{−9}			1.0 × 10 ^{−9}	
		M	0.200	5.4 × 10 ^{−9}	0.100	4.5 × 10 ^{−9}	2.5 × 10 ^{−9}	1.7 × 10 ^{−9}	1.3 × 10 ^{−9}			1.1 × 10 ^{−9}	
		S	0.020	5.6 × 10 ^{−9}	0.010	4.7 × 10 ^{−9}	2.9 × 10 ^{−9}	2.0 × 10 ^{−9}	1.6 × 10 ^{−9}			1.3 × 10 ^{−9}	
Se-79	6.50 × 10 ⁴ a	F	1.000	1.6 × 10 ^{−8}	0.800	1.3 × 10 ^{−8}	7.7 × 10 ^{−9}	5.6 × 10 ^{−9}	3.3 × 10 ^{−9}			2.6 × 10 ^{−9}	
		M	0.200	1.4 × 10 ^{−8}	0.100	1.1 × 10 ^{−8}	6.9 × 10 ^{−9}	4.9 × 10 ^{−9}	3.3 × 10 ^{−9}			2.6 × 10 ^{−9}	
		S	0.020	2.3 × 10 ^{−8}	0.010	2.0 × 10 ^{−8}	1.3 × 10 ^{−8}	8.7 × 10 ^{−9}	7.6 × 10 ^{−9}			6.8 × 10 ^{−9}	
Se-81	0.308 h	F	1.000	8.6 × 10 ^{−11}	0.800	5.4 × 10 ^{−11}	2.3 × 10 ^{−11}	1.5 × 10 ^{−11}	9.2 × 10 ^{−12}			8.0 × 10 ^{−12}	
		M	0.200	1.3 × 10 ^{−10}	0.100	8.5 × 10 ^{−11}	3.8 × 10 ^{−11}	2.5 × 10 ^{−11}	1.6 × 10 ^{−11}			1.4 × 10 ^{−11}	
		S	0.020	1.4 × 10 ^{−10}	0.010	8.9 × 10 ^{−11}	3.9 × 10 ^{−11}	2.6 × 10 ^{−11}	1.7 × 10 ^{−11}			1.5 × 10 ^{−11}	
Se-81m	0.954 h	F	1.000	1.8 × 10 ^{−10}	0.800	1.2 × 10 ^{−10}	5.4 × 10 ^{−11}	3.4 × 10 ^{−11}	1.9 × 10 ^{−11}			1.6 × 10 ^{−11}	
		M	0.200	3.8 × 10 ^{−10}	0.100	2.5 × 10 ^{−10}	1.2 × 10 ^{−10}	8.0 × 10 ^{−11}	5.8 × 10 ^{−11}			4.7 × 10 ^{−11}	
		S	0.020	4.1 × 10 ^{−10}	0.010	2.7 × 10 ^{−10}	1.3 × 10 ^{−10}	8.5 × 10 ^{−11}	6.2 × 10 ^{−11}			5.1 × 10 ^{−11}	

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)						<i>e</i> (g)		<i>e</i> (g)		<i>e</i> (g)	
Se-83	0.375 h	F	1.000	1.7 × 10 ^{−10}	0.800	1.2 × 10 ^{−10}	5.8 × 10 ^{−11}	3.6 × 10 ^{−11}	2.1 × 10 ^{−11}	2.1 × 10 ^{−11}	1.8 × 10 ^{−11}						
		M	0.200	2.7 × 10 ^{−10}	0.100	1.9 × 10 ^{−10}	9.2 × 10 ^{−11}	5.9 × 10 ^{−11}	3.9 × 10 ^{−11}	3.9 × 10 ^{−11}	3.2 × 10 ^{−11}						
		S	0.020	2.8 × 10 ^{−10}	0.010	2.0 × 10 ^{−10}	9.6 × 10 ^{−11}	6.2 × 10 ^{−11}	4.1 × 10 ^{−11}	4.1 × 10 ^{−11}	3.4 × 10 ^{−11}						
Bromine																	
Br-74	0.422 h	F	1.000	2.5 × 10 ^{−10}	1.000	1.8 × 10 ^{−10}	8.6 × 10 ^{−11}	5.3 × 10 ^{−11}	3.2 × 10 ^{−11}	3.2 × 10 ^{−11}	2.6 × 10 ^{−11}						
		M	1.000	3.6 × 10 ^{−10}	1.000	2.5 × 10 ^{−10}	1.2 × 10 ^{−10}	7.5 × 10 ^{−11}	4.6 × 10 ^{−11}	4.6 × 10 ^{−11}	3.8 × 10 ^{−11}						
Br-74m	0.691 h	F	1.000	4.0 × 10 ^{−10}	1.000	2.8 × 10 ^{−10}	1.3 × 10 ^{−10}	8.1 × 10 ^{−11}	4.8 × 10 ^{−11}	4.8 × 10 ^{−11}	3.9 × 10 ^{−11}						
		M	1.000	5.9 × 10 ^{−10}	1.000	4.1 × 10 ^{−10}	1.9 × 10 ^{−10}	1.2 × 10 ^{−10}	7.5 × 10 ^{−11}	7.5 × 10 ^{−11}	6.2 × 10 ^{−11}						
Br-75	1.63 h	F	1.000	2.9 × 10 ^{−10}	1.000	2.1 × 10 ^{−10}	9.7 × 10 ^{−11}	5.9 × 10 ^{−11}	3.5 × 10 ^{−11}	3.5 × 10 ^{−11}	2.9 × 10 ^{−11}						
		M	1.000	4.5 × 10 ^{−10}	1.000	3.1 × 10 ^{−10}	1.5 × 10 ^{−10}	9.7 × 10 ^{−11}	6.5 × 10 ^{−11}	6.5 × 10 ^{−11}	5.3 × 10 ^{−11}						
Br-76	16.2 h	F	1.000	2.2 × 10 ^{−9}	1.000	1.7 × 10 ^{−9}	8.4 × 10 ^{−10}	5.1 × 10 ^{−10}	3.0 × 10 ^{−10}	3.0 × 10 ^{−10}	2.4 × 10 ^{−10}						
		M	1.000	3.0 × 10 ^{−9}	1.000	2.3 × 10 ^{−9}	1.2 × 10 ^{−9}	7.5 × 10 ^{−10}	5.0 × 10 ^{−10}	5.0 × 10 ^{−10}	4.1 × 10 ^{−10}						
Br-77	2.33 d	F	1.000	5.3 × 10 ^{−10}	1.000	4.4 × 10 ^{−10}	2.2 × 10 ^{−10}	1.3 × 10 ^{−10}	7.7 × 10 ^{−11}	7.7 × 10 ^{−11}	6.2 × 10 ^{−11}						
		M	1.000	6.3 × 10 ^{−10}	1.000	5.1 × 10 ^{−10}	2.7 × 10 ^{−10}	1.6 × 10 ^{−10}	1.1 × 10 ^{−10}	1.1 × 10 ^{−10}	8.4 × 10 ^{−11}						
Br-80	0.290 h	F	1.000	7.1 × 10 ^{−11}	1.000	4.4 × 10 ^{−11}	1.8 × 10 ^{−11}	1.2 × 10 ^{−11}	6.9 × 10 ^{−12}	6.9 × 10 ^{−12}	5.9 × 10 ^{−12}						
		M	1.000	1.1 × 10 ^{−10}	1.000	6.5 × 10 ^{−11}	2.8 × 10 ^{−11}	1.8 × 10 ^{−11}	1.1 × 10 ^{−11}	1.1 × 10 ^{−11}	9.4 × 10 ^{−12}						
Br-80m	4.42 h	F	1.000	4.3 × 10 ^{−10}	1.000	2.8 × 10 ^{−10}	1.2 × 10 ^{−10}	7.2 × 10 ^{−11}	4.0 × 10 ^{−11}	4.0 × 10 ^{−11}	3.3 × 10 ^{−11}						
		M	1.000	6.8 × 10 ^{−10}	1.000	4.5 × 10 ^{−10}	2.1 × 10 ^{−10}	1.4 × 10 ^{−10}	9.3 × 10 ^{−11}	9.3 × 10 ^{−11}	7.6 × 10 ^{−11}						
Br-82	1.47 d	F	1.000	2.7 × 10 ^{−9}	1.000	2.2 × 10 ^{−9}	1.2 × 10 ^{−9}	7.0 × 10 ^{−10}	4.2 × 10 ^{−10}	4.2 × 10 ^{−10}	3.5 × 10 ^{−10}						
		M	1.000	3.8 × 10 ^{−9}	1.000	3.0 × 10 ^{−9}	1.7 × 10 ^{−9}	1.1 × 10 ^{−9}	7.9 × 10 ^{−10}	7.9 × 10 ^{−10}	6.3 × 10 ^{−10}						

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a						7–12 a		12–17 a		>17 a		
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)						<i>e</i> (g)		<i>e</i> (g)		<i>e</i> (g)	
Br-83	2.39 h	F	1.000	1.7 × 10 ^{−10}	1.000	1.1 × 10 ^{−10}	4.7 × 10 ^{−11}	3.0 × 10 ^{−11}	1.8 × 10 ^{−11}	1.6 × 10 ^{−11}							
		M	1.000	3.5 × 10 ^{−10}	1.000	2.3 × 10 ^{−10}	1.1 × 10 ^{−10}	7.7 × 10 ^{−11}	5.9 × 10 ^{−11}	4.8 × 10 ^{−11}							
	0.530 h	F	1.000	2.4 × 10 ^{−10}	1.000	1.6 × 10 ^{−10}	7.1 × 10 ^{−11}	4.4 × 10 ^{−11}	2.6 × 10 ^{−11}	2.2 × 10 ^{−11}							
		M	1.000	3.7 × 10 ^{−10}	1.000	2.4 × 10 ^{−10}	1.1 × 10 ^{−10}	6.9 × 10 ^{−11}	4.4 × 10 ^{−11}	3.7 × 10 ^{−11}							
Rubidium																	
Rb-79	0.382 h	F	1.000	1.6 × 10 ^{−10}	1.000	1.1 × 10 ^{−10}	5.0 × 10 ^{−11}	3.2 × 10 ^{−11}	1.9 × 10 ^{−11}	1.6 × 10 ^{−11}							
Rb-81	4.58 h	F	1.000	3.2 × 10 ^{−10}	1.000	2.5 × 10 ^{−10}	1.2 × 10 ^{−10}	7.1 × 10 ^{−11}	4.2 × 10 ^{−11}	3.4 × 10 ^{−11}							
Rb-81m	0.533 h	F	1.000	6.2 × 10 ^{−11}	1.000	4.6 × 10 ^{−11}	2.2 × 10 ^{−11}	1.4 × 10 ^{−11}	8.5 × 10 ^{−12}	7.0 × 10 ^{−12}							
Rb-82m	6.20 h	F	1.000	8.6 × 10 ^{−10}	1.000	7.3 × 10 ^{−10}	3.9 × 10 ^{−10}	2.3 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}							
Rb-83	86.2 d	F	1.000	4.9 × 10 ^{−9}	1.000	3.8 × 10 ^{−9}	2.0 × 10 ^{−9}	1.3 × 10 ^{−9}	7.9 × 10 ^{−10}	6.9 × 10 ^{−10}							
Rb-84	32.8 d	F	1.000	8.6 × 10 ^{−9}	1.000	6.4 × 10 ^{−9}	3.1 × 10 ^{−9}	2.0 × 10 ^{−9}	1.2 × 10 ^{−9}	1.0 × 10 ^{−9}							
Rb-86	18.7 d	F	1.000	1.2 × 10 ^{−8}	1.000	7.7 × 10 ^{−9}	3.4 × 10 ^{−9}	2.0 × 10 ^{−9}	1.1 × 10 ^{−9}	9.3 × 10 ^{−10}							
Rb-87	4.70 × 10 ¹⁰ a	F	1.000	6.0 × 10 ^{−9}	1.000	4.1 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	6.0 × 10 ^{−10}	5.0 × 10 ^{−10}							
Rb-88	0.297 h	F	1.000	1.9 × 10 ^{−10}	1.000	1.2 × 10 ^{−10}	5.2 × 10 ^{−11}	3.2 × 10 ^{−11}	1.9 × 10 ^{−11}	1.6 × 10 ^{−11}							
Rb-89	0.253 h	F	1.000	1.4 × 10 ^{−10}	1.000	9.3 × 10 ^{−11}	4.3 × 10 ^{−11}	2.7 × 10 ^{−11}	1.6 × 10 ^{−11}	1.4 × 10 ^{−11}							
Strontium^e																	
Sr-80	1.67 h	F	0.600	7.8 × 10 ^{−10}	0.300	5.4 × 10 ^{−10}	2.4 × 10 ^{−10}	1.4 × 10 ^{−10}	7.9 × 10 ^{−11}	7.1 × 10 ^{−11}							
		M	0.200	1.4 × 10 ^{−9}	0.100	9.0 × 10 ^{−10}	4.1 × 10 ^{−10}	2.5 × 10 ^{−10}	1.5 × 10 ^{−10}	1.3 × 10 ^{−10}							
		S	0.020	1.5 × 10 ^{−9}	0.010	9.4 × 10 ^{−10}	4.3 × 10 ^{−10}	2.7 × 10 ^{−10}	1.6 × 10 ^{−10}	1.4 × 10 ^{−10}							

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)		
Sr-81	0.425 h	F	0.600	2.1 × 10 ⁻¹⁰	0.300	1.5 × 10 ⁻¹⁰	6.7 × 10 ⁻¹¹	4.1 × 10 ⁻¹¹	2.4 × 10 ⁻¹¹	2.1 × 10 ⁻¹¹				
		M	0.200	3.3 × 10 ⁻¹⁰	0.100	2.2 × 10 ⁻¹⁰	1.0 × 10 ⁻¹⁰	6.6 × 10 ⁻¹¹	4.2 × 10 ⁻¹¹	3.5 × 10 ⁻¹¹				
		S	0.020	3.4 × 10 ⁻¹⁰	0.010	2.3 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰	6.9 × 10 ⁻¹¹	4.4 × 10 ⁻¹¹	3.7 × 10 ⁻¹¹				
Sr-82	25.0 d	F	0.600	2.8 × 10 ⁻⁸	0.300	1.5 × 10 ⁻⁸	6.6 × 10 ⁻⁹	4.6 × 10 ⁻⁹	3.2 × 10 ⁻⁹	2.1 × 10 ⁻⁹				
		M	0.200	5.5 × 10 ⁻⁸	0.100	4.0 × 10 ⁻⁸	2.1 × 10 ⁻⁸	1.4 × 10 ⁻⁸	1.0 × 10 ⁻⁸	8.9 × 10 ⁻⁹				
		S	0.020	6.1 × 10 ⁻⁸	0.010	4.6 × 10 ⁻⁸	2.5 × 10 ⁻⁸	1.7 × 10 ⁻⁸	1.2 × 10 ⁻⁸	1.1 × 10 ⁻⁸				
Sr-83	1.35 d	F	0.600	1.4 × 10 ⁻⁹	0.300	1.1 × 10 ⁻⁹	5.5 × 10 ⁻¹⁰	3.4 × 10 ⁻¹⁰	2.0 × 10 ⁻¹⁰	1.6 × 10 ⁻¹⁰				
		M	0.200	2.5 × 10 ⁻⁹	0.100	1.9 × 10 ⁻⁹	9.5 × 10 ⁻¹⁰	6.0 × 10 ⁻¹⁰	3.9 × 10 ⁻¹⁰	3.1 × 10 ⁻¹⁰				
		S	0.020	2.8 × 10 ⁻⁹	0.010	2.0 × 10 ⁻⁹	1.0 × 10 ⁻⁹	6.5 × 10 ⁻¹⁰	4.2 × 10 ⁻¹⁰	3.4 × 10 ⁻¹⁰				
Sr-85	64.8 d	F	0.600	4.4 × 10 ⁻⁹	0.300	2.3 × 10 ⁻⁹	1.1 × 10 ⁻⁹	9.6 × 10 ⁻¹⁰	8.3 × 10 ⁻¹⁰	3.8 × 10 ⁻¹⁰				
		M	0.200	4.3 × 10 ⁻⁹	0.100	3.1 × 10 ⁻⁹	1.8 × 10 ⁻⁹	1.2 × 10 ⁻⁹	8.8 × 10 ⁻¹⁰	6.4 × 10 ⁻¹⁰				
		S	0.020	4.4 × 10 ⁻⁹	0.010	3.7 × 10 ⁻⁹	2.2 × 10 ⁻⁹	1.3 × 10 ⁻⁹	1.0 × 10 ⁻⁹	8.1 × 10 ⁻¹⁰				
Sr-85m	1.16 h	F	0.600	2.4 × 10 ⁻¹¹	0.300	1.9 × 10 ⁻¹¹	9.6 × 10 ⁻¹²	6.0 × 10 ⁻¹²	3.7 × 10 ⁻¹²	2.9 × 10 ⁻¹²				
		M	0.200	3.1 × 10 ⁻¹¹	0.100	2.5 × 10 ⁻¹¹	1.3 × 10 ⁻¹¹	8.0 × 10 ⁻¹²	5.1 × 10 ⁻¹²	4.1 × 10 ⁻¹²				
		S	0.020	3.2 × 10 ⁻¹¹	0.010	2.6 × 10 ⁻¹¹	1.3 × 10 ⁻¹¹	8.3 × 10 ⁻¹²	5.4 × 10 ⁻¹²	4.3 × 10 ⁻¹²				
Sr-87m	2.80 h	F	0.600	9.7 × 10 ⁻¹¹	0.300	7.8 × 10 ⁻¹¹	3.8 × 10 ⁻¹¹	2.3 × 10 ⁻¹¹	1.3 × 10 ⁻¹¹	1.1 × 10 ⁻¹¹				
		M	0.200	1.6 × 10 ⁻¹⁰	0.100	1.2 × 10 ⁻¹⁰	5.9 × 10 ⁻¹¹	3.8 × 10 ⁻¹¹	2.5 × 10 ⁻¹¹	2.0 × 10 ⁻¹¹				
		S	0.020	1.7 × 10 ⁻¹⁰	0.010	1.2 × 10 ⁻¹⁰	6.2 × 10 ⁻¹¹	4.0 × 10 ⁻¹¹	2.6 × 10 ⁻¹¹	2.1 × 10 ⁻¹¹				
Sr-89	50.5 d	F	0.600	1.5 × 10 ⁻⁸	0.300	7.3 × 10 ⁻⁹	3.2 × 10 ⁻⁹	2.3 × 10 ⁻⁹	1.7 × 10 ⁻⁹	1.0 × 10 ⁻⁹				
		M	0.200	3.3 × 10 ⁻⁸	0.100	2.4 × 10 ⁻⁸	1.3 × 10 ⁻⁸	9.1 × 10 ⁻⁹	7.3 × 10 ⁻⁹	6.1 × 10 ⁻⁹				
		S	0.020	3.9 × 10 ⁻⁸	0.010	3.0 × 10 ⁻⁸	1.7 × 10 ⁻⁸	1.2 × 10 ⁻⁸	9.3 × 10 ⁻⁹	7.9 × 10 ⁻⁹				

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)			
Sr-90	29.1 a	F	0.600	1.3 × 10 ^{−7}	0.300	5.2 × 10 ^{−8}	3.1 × 10 ^{−8}	4.1 × 10 ^{−8}	5.3 × 10 ^{−8}	5.3 × 10 ^{−8}	2.4 × 10 ^{−8}		
		M	0.200	1.5 × 10 ^{−7}	0.100	1.1 × 10 ^{−7}	6.5 × 10 ^{−8}	5.1 × 10 ^{−8}	5.0 × 10 ^{−8}	5.0 × 10 ^{−8}	3.6 × 10 ^{−8}		
		S	0.020	4.2 × 10 ^{−7}	0.010	4.0 × 10 ^{−7}	2.7 × 10 ^{−7}	1.8 × 10 ^{−7}	1.6 × 10 ^{−7}	1.6 × 10 ^{−7}	1.6 × 10 ^{−7}		
Sr-91	9.50 h	F	0.600	1.4 × 10 ^{−9}	0.300	1.1 × 10 ^{−9}	5.2 × 10 ^{−10}	3.1 × 10 ^{−10}	1.7 × 10 ^{−10}	1.7 × 10 ^{−10}	1.6 × 10 ^{−10}		
		M	0.200	3.1 × 10 ^{−9}	0.100	2.2 × 10 ^{−9}	1.1 × 10 ^{−9}	6.9 × 10 ^{−10}	4.4 × 10 ^{−10}	4.4 × 10 ^{−10}	3.7 × 10 ^{−10}		
		S	0.020	3.5 × 10 ^{−9}	0.010	2.5 × 10 ^{−9}	1.2 × 10 ^{−9}	7.7 × 10 ^{−10}	4.9 × 10 ^{−10}	4.9 × 10 ^{−10}	4.1 × 10 ^{−10}		
Sr-92	2.71 h	F	0.600	9.0 × 10 ^{−10}	0.300	7.1 × 10 ^{−10}	3.3 × 10 ^{−10}	2.0 × 10 ^{−10}	1.0 × 10 ^{−10}	1.0 × 10 ^{−10}	9.8 × 10 ^{−11}		
		M	0.200	1.9 × 10 ^{−9}	0.100	1.4 × 10 ^{−9}	6.5 × 10 ^{−10}	4.1 × 10 ^{−10}	2.5 × 10 ^{−10}	2.5 × 10 ^{−10}	2.1 × 10 ^{−10}		
		S	0.020	2.2 × 10 ^{−9}	0.010	1.5 × 10 ^{−9}	7.0 × 10 ^{−10}	4.5 × 10 ^{−10}	2.7 × 10 ^{−10}	2.7 × 10 ^{−10}	2.3 × 10 ^{−10}		
Yttrium													
Y-86	14.7 h	M	0.001	3.7 × 10 ^{−9}	1.0 × 10 ^{−4}	2.9 × 10 ^{−9}	1.5 × 10 ^{−9}	9.3 × 10 ^{−10}	5.6 × 10 ^{−10}	5.6 × 10 ^{−10}	4.5 × 10 ^{−10}		
		S	0.001	3.8 × 10 ^{−9}	1.0 × 10 ^{−4}	3.0 × 10 ^{−9}	1.5 × 10 ^{−9}	9.6 × 10 ^{−10}	5.8 × 10 ^{−10}	5.8 × 10 ^{−10}	4.7 × 10 ^{−10}		
Y-86m	0.800 h	M	0.001	2.2 × 10 ^{−10}	1.0 × 10 ^{−4}	1.7 × 10 ^{−10}	8.7 × 10 ^{−11}	5.6 × 10 ^{−11}	3.4 × 10 ^{−11}	3.4 × 10 ^{−11}	2.7 × 10 ^{−11}		
		S	0.001	2.3 × 10 ^{−10}	1.0 × 10 ^{−4}	1.8 × 10 ^{−10}	9.0 × 10 ^{−11}	5.7 × 10 ^{−11}	3.5 × 10 ^{−11}	3.5 × 10 ^{−11}	2.8 × 10 ^{−11}		
Y-87	3.35 d	M	0.001	2.7 × 10 ^{−9}	1.0 × 10 ^{−4}	2.1 × 10 ^{−9}	1.1 × 10 ^{−9}	7.0 × 10 ^{−10}	4.7 × 10 ^{−10}	4.7 × 10 ^{−10}	3.7 × 10 ^{−10}		
		S	0.001	2.8 × 10 ^{−9}	1.0 × 10 ^{−4}	2.2 × 10 ^{−9}	1.1 × 10 ^{−9}	7.3 × 10 ^{−10}	5.0 × 10 ^{−10}	5.0 × 10 ^{−10}	3.9 × 10 ^{−10}		
Y-88	107 d	M	0.001	1.9 × 10 ^{−8}	1.0 × 10 ^{−4}	1.6 × 10 ^{−8}	1.0 × 10 ^{−8}	6.7 × 10 ^{−9}	4.9 × 10 ^{−9}	4.9 × 10 ^{−9}	4.1 × 10 ^{−9}		
		S	0.001	2.0 × 10 ^{−8}	1.0 × 10 ^{−4}	1.7 × 10 ^{−8}	9.8 × 10 ^{−9}	6.6 × 10 ^{−9}	5.4 × 10 ^{−9}	5.4 × 10 ^{−9}	4.4 × 10 ^{−9}		
Y-90	2.67 d	M	0.001	1.3 × 10 ^{−8}	1.0 × 10 ^{−4}	8.4 × 10 ^{−9}	4.0 × 10 ^{−9}	2.6 × 10 ^{−9}	1.7 × 10 ^{−9}	1.7 × 10 ^{−9}	1.4 × 10 ^{−9}		
		S	0.001	1.3 × 10 ^{−8}	1.0 × 10 ^{−4}	8.8 × 10 ^{−9}	4.2 × 10 ^{−9}	2.7 × 10 ^{−9}	1.8 × 10 ^{−9}	1.8 × 10 ^{−9}	1.5 × 10 ^{−9}		

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Y-90m	3.19 h	M	0.001	7.2×10^{-10}	1.0×10^{-4}	5.7×10^{-10}	2.8×10^{-10}	1.8×10^{-10}	1.1×10^{-10}	1.1×10^{-10}	1.1×10^{-10}	9.5×10^{-11}	
		S	0.001	7.5×10^{-10}	1.0×10^{-4}	6.0×10^{-10}	2.9×10^{-10}	1.9×10^{-10}	1.2×10^{-10}	1.2×10^{-10}	1.0×10^{-10}		
Y-91	58.5 d	M	0.001	3.9×10^{-8}	1.0×10^{-4}	3.0×10^{-8}	1.6×10^{-8}	1.1×10^{-8}	8.4×10^{-9}	8.4×10^{-9}	8.4×10^{-9}	7.1×10^{-9}	
		S	0.001	4.3×10^{-8}	1.0×10^{-4}	3.4×10^{-8}	1.9×10^{-8}	1.3×10^{-8}	1.0×10^{-8}	1.0×10^{-8}	8.9×10^{-9}		
Y-91m	0.828 h	M	0.001	7.0×10^{-11}	1.0×10^{-4}	5.5×10^{-11}	2.9×10^{-11}	1.8×10^{-11}	1.2×10^{-11}	1.2×10^{-11}	1.2×10^{-11}	1.0×10^{-11}	
		S	0.001	7.4×10^{-11}	1.0×10^{-4}	5.9×10^{-11}	3.1×10^{-11}	2.0×10^{-11}	1.4×10^{-11}	1.4×10^{-11}	1.1×10^{-11}		
Y-92	3.54 h	M	0.001	1.8×10^{-9}	1.0×10^{-4}	1.2×10^{-9}	5.3×10^{-10}	3.3×10^{-10}	2.0×10^{-10}	2.0×10^{-10}	2.0×10^{-10}	1.7×10^{-10}	
		S	0.001	1.9×10^{-9}	1.0×10^{-4}	1.2×10^{-9}	5.5×10^{-10}	3.5×10^{-10}	2.1×10^{-10}	2.1×10^{-10}	1.8×10^{-10}		
Y-93	10.1 h	M	0.001	4.4×10^{-9}	1.0×10^{-4}	2.9×10^{-9}	1.3×10^{-9}	8.1×10^{-10}	4.7×10^{-10}	4.7×10^{-10}	4.7×10^{-10}	4.0×10^{-10}	
		S	0.001	4.6×10^{-9}	1.0×10^{-4}	3.0×10^{-9}	1.4×10^{-9}	8.5×10^{-10}	5.0×10^{-10}	5.0×10^{-10}	4.2×10^{-10}		
Y-94	0.318 h	M	0.001	2.8×10^{-10}	1.0×10^{-4}	1.8×10^{-10}	8.1×10^{-11}	5.0×10^{-11}	3.1×10^{-11}	3.1×10^{-11}	3.1×10^{-11}	2.7×10^{-11}	
		S	0.001	2.9×10^{-10}	1.0×10^{-4}	1.9×10^{-10}	8.4×10^{-11}	5.2×10^{-11}	3.3×10^{-11}	3.3×10^{-11}	2.8×10^{-11}		
Y-95	0.178 h	M	0.001	1.5×10^{-10}	1.0×10^{-4}	9.8×10^{-11}	4.4×10^{-11}	2.8×10^{-11}	1.8×10^{-11}	1.8×10^{-11}	1.8×10^{-11}	1.5×10^{-11}	
		S	0.001	1.6×10^{-10}	1.0×10^{-4}	1.0×10^{-10}	4.5×10^{-11}	2.9×10^{-11}	1.8×10^{-11}	1.8×10^{-11}	1.6×10^{-11}		
Zirconium													
Zr-86	16.5 h	F	0.020	2.4×10^{-9}	0.002	1.9×10^{-9}	9.5×10^{-10}	5.9×10^{-10}	3.4×10^{-10}	3.4×10^{-10}	3.4×10^{-10}	2.7×10^{-10}	
		M	0.020	3.4×10^{-9}	0.002	2.6×10^{-9}	1.3×10^{-9}	8.4×10^{-10}	5.2×10^{-10}	5.2×10^{-10}	4.2×10^{-10}		
		S	0.020	3.5×10^{-9}	0.002	2.7×10^{-9}	1.4×10^{-9}	8.7×10^{-10}	5.4×10^{-10}	5.4×10^{-10}	4.3×10^{-10}		
Zr-88	83.4 d	F	0.020	6.9×10^{-9}	0.002	8.3×10^{-9}	5.6×10^{-9}	4.7×10^{-9}	3.6×10^{-9}	3.6×10^{-9}	3.6×10^{-9}	3.5×10^{-9}	
		M	0.020	8.5×10^{-9}	0.002	7.8×10^{-9}	5.1×10^{-9}	3.6×10^{-9}	3.0×10^{-9}	3.0×10^{-9}	2.6×10^{-9}		
		S	0.020	1.3×10^{-8}	0.002	1.2×10^{-8}	7.7×10^{-9}	5.2×10^{-9}	4.3×10^{-9}	4.3×10^{-9}	3.6×10^{-9}		

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)				
Zr-89	3.27 d	F	0.020	2.6 × 10 ^{−9}	0.002	2.0 × 10 ^{−9}	9.9 × 10 ^{−10}	6.1 × 10 ^{−10}	3.6 × 10 ^{−10}	2.9 × 10 ^{−10}				
		M	0.020	3.7 × 10 ^{−9}	0.002	2.8 × 10 ^{−9}	1.5 × 10 ^{−9}	9.6 × 10 ^{−10}	6.5 × 10 ^{−10}	5.2 × 10 ^{−10}				
		S	0.020	3.9 × 10 ^{−9}	0.002	2.9 × 10 ^{−9}	1.5 × 10 ^{−9}	1.0 × 10 ^{−9}	6.8 × 10 ^{−10}	5.5 × 10 ^{−10}				
Zr-93	1.53 × 10 ⁶ a	F	0.020	3.5 × 10 ^{−9}	0.002	4.8 × 10 ^{−9}	5.3 × 10 ^{−9}	9.7 × 10 ^{−9}	1.8 × 10 ^{−8}	2.5 × 10 ^{−8}				
		M	0.020	3.3 × 10 ^{−9}	0.002	3.1 × 10 ^{−9}	2.8 × 10 ^{−9}	4.1 × 10 ^{−9}	7.5 × 10 ^{−9}	1.0 × 10 ^{−8}				
		S	0.020	7.0 × 10 ^{−9}	0.002	6.4 × 10 ^{−9}	4.5 × 10 ^{−9}	3.3 × 10 ^{−9}	3.3 × 10 ^{−9}	3.3 × 10 ^{−9}				
Zr-95	64.0 d	F	0.020	1.2 × 10 ^{−8}	0.002	1.1 × 10 ^{−8}	6.4 × 10 ^{−9}	4.2 × 10 ^{−9}	2.8 × 10 ^{−9}	2.5 × 10 ^{−9}				
		M	0.020	2.0 × 10 ^{−8}	0.002	1.6 × 10 ^{−8}	9.7 × 10 ^{−9}	6.8 × 10 ^{−9}	5.9 × 10 ^{−9}	4.8 × 10 ^{−9}				
		S	0.020	2.4 × 10 ^{−8}	0.002	1.9 × 10 ^{−8}	1.2 × 10 ^{−8}	8.3 × 10 ^{−9}	7.3 × 10 ^{−9}	5.9 × 10 ^{−9}				
Zr-97	16.9 h	F	0.020	5.0 × 10 ^{−9}	0.002	3.4 × 10 ^{−9}	1.5 × 10 ^{−9}	9.1 × 10 ^{−10}	4.8 × 10 ^{−10}	3.9 × 10 ^{−10}				
		M	0.020	7.8 × 10 ^{−9}	0.002	5.3 × 10 ^{−9}	2.8 × 10 ^{−9}	1.8 × 10 ^{−9}	1.1 × 10 ^{−9}	9.2 × 10 ^{−10}				
		S	0.020	8.2 × 10 ^{−9}	0.002	5.6 × 10 ^{−9}	2.9 × 10 ^{−9}	1.9 × 10 ^{−9}	1.2 × 10 ^{−9}	8.9 × 10 ^{−10}				
Niobium														
Nb-88	0.238 h	F	0.020	1.8 × 10 ^{−10}	0.010	1.3 × 10 ^{−10}	6.3 × 10 ^{−11}	3.9 × 10 ^{−11}	2.4 × 10 ^{−11}	1.9 × 10 ^{−11}				
		M	0.020	2.5 × 10 ^{−10}	0.010	1.8 × 10 ^{−10}	8.5 × 10 ^{−11}	5.3 × 10 ^{−11}	3.3 × 10 ^{−11}	2.7 × 10 ^{−11}				
		S	0.020	2.6 × 10 ^{−10}	0.010	1.8 × 10 ^{−10}	8.7 × 10 ^{−11}	5.5 × 10 ^{−11}	3.5 × 10 ^{−11}	2.8 × 10 ^{−11}				
Nb-89	2.03 h	F	0.020	7.0 × 10 ^{−10}	0.010	4.8 × 10 ^{−10}	2.2 × 10 ^{−10}	1.3 × 10 ^{−10}	7.4 × 10 ^{−11}	6.1 × 10 ^{−11}				
		M	0.020	1.1 × 10 ^{−9}	0.010	7.6 × 10 ^{−10}	3.6 × 10 ^{−10}	2.2 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}				
		S	0.020	1.2 × 10 ^{−9}	0.010	7.9 × 10 ^{−10}	3.7 × 10 ^{−10}	2.3 × 10 ^{−10}	1.5 × 10 ^{−10}	1.2 × 10 ^{−10}				

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Nb-89m	1.10 h	F	0.020	4.0×10^{-10}	0.010	2.9×10^{-10}	1.4×10^{-10}	8.3×10^{-11}	4.8×10^{-11}	3.9×10^{-11}				
		M	0.020	6.2×10^{-10}	0.010	4.3×10^{-10}	2.1×10^{-10}	1.3×10^{-10}	8.2×10^{-11}	6.8×10^{-11}				
		S	0.020	6.4×10^{-10}	0.010	4.4×10^{-10}	2.1×10^{-10}	1.4×10^{-10}	8.6×10^{-11}	7.1×10^{-11}				
Nb-90	14.6 h	F	0.020	3.5×10^{-9}	0.010	2.7×10^{-9}	1.3×10^{-9}	8.2×10^{-10}	4.7×10^{-10}	3.8×10^{-10}				
		M	0.020	5.1×10^{-9}	0.010	3.9×10^{-9}	1.9×10^{-9}	1.3×10^{-9}	7.8×10^{-10}	6.3×10^{-10}				
		S	0.020	5.3×10^{-9}	0.010	4.0×10^{-9}	2.0×10^{-9}	1.3×10^{-9}	8.1×10^{-10}	6.6×10^{-10}				
Nb-93m	13.6 a	F	0.020	1.8×10^{-9}	0.010	1.4×10^{-9}	7.0×10^{-10}	4.4×10^{-10}	2.7×10^{-10}	2.2×10^{-10}				
		M	0.020	3.1×10^{-9}	0.010	2.4×10^{-9}	1.3×10^{-9}	8.2×10^{-10}	5.9×10^{-10}	5.1×10^{-10}				
		S	0.020	7.4×10^{-9}	0.010	6.5×10^{-9}	4.0×10^{-9}	2.5×10^{-9}	1.9×10^{-9}	1.8×10^{-9}				
Nb-94	2.03×10^4 a	F	0.020	3.1×10^{-8}	0.010	2.7×10^{-8}	1.5×10^{-8}	1.0×10^{-8}	6.7×10^{-9}	5.8×10^{-9}				
		M	0.020	4.3×10^{-8}	0.010	3.7×10^{-8}	2.3×10^{-8}	1.6×10^{-8}	1.3×10^{-8}	1.1×10^{-8}				
		S	0.020	1.2×10^{-7}	0.010	1.2×10^{-7}	8.3×10^{-8}	5.8×10^{-8}	5.2×10^{-8}	4.9×10^{-8}				
Nb-95	35.1 d	F	0.020	4.1×10^{-9}	0.010	3.1×10^{-9}	1.6×10^{-9}	1.2×10^{-9}	7.5×10^{-10}	5.7×10^{-10}				
		M	0.020	6.8×10^{-9}	0.010	5.2×10^{-9}	3.1×10^{-9}	2.2×10^{-9}	1.9×10^{-9}	1.5×10^{-9}				
		S	0.020	7.7×10^{-9}	0.010	5.9×10^{-9}	3.6×10^{-9}	2.5×10^{-9}	2.2×10^{-9}	1.8×10^{-9}				
Nb-95m	3.61 d	F	0.020	2.3×10^{-9}	0.010	1.6×10^{-9}	7.0×10^{-10}	4.2×10^{-10}	2.4×10^{-10}	2.0×10^{-10}				
		M	0.020	4.3×10^{-9}	0.010	3.1×10^{-9}	1.7×10^{-9}	1.2×10^{-9}	1.0×10^{-9}	7.9×10^{-10}				
		S	0.020	4.6×10^{-9}	0.010	3.4×10^{-9}	1.9×10^{-9}	1.3×10^{-9}	1.1×10^{-9}	8.8×10^{-10}				
Nb-96	23.3 h	F	0.020	3.1×10^{-9}	0.010	2.4×10^{-9}	1.2×10^{-9}	7.3×10^{-10}	4.2×10^{-10}	3.4×10^{-10}				
		M	0.020	4.7×10^{-9}	0.010	3.6×10^{-9}	1.8×10^{-9}	1.2×10^{-9}	7.8×10^{-10}	6.3×10^{-10}				
		S	0.020	4.9×10^{-9}	0.010	3.7×10^{-9}	1.9×10^{-9}	1.2×10^{-9}	8.3×10^{-10}	6.6×10^{-10}				

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)			
Nb-97	1.20 h	F	0.020	2.2 × 10 ⁻¹⁰	0.010	1.5 × 10 ⁻¹⁰	6.8 × 10 ⁻¹¹	4.2 × 10 ⁻¹¹	2.5 × 10 ⁻¹¹	2.1 × 10 ⁻¹¹			
		M	0.020	3.7 × 10 ⁻¹⁰	0.010	2.5 × 10 ⁻¹⁰	1.2 × 10 ⁻¹⁰	7.7 × 10 ⁻¹¹	5.2 × 10 ⁻¹¹	4.3 × 10 ⁻¹¹			
		S	0.020	3.8 × 10 ⁻¹⁰	0.010	2.6 × 10 ⁻¹⁰	1.2 × 10 ⁻¹⁰	8.1 × 10 ⁻¹¹	5.5 × 10 ⁻¹¹	4.5 × 10 ⁻¹¹			
Nb-98	0.858 h	F	0.020	3.4 × 10 ⁻¹⁰	0.010	2.4 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰	6.9 × 10 ⁻¹¹	4.1 × 10 ⁻¹¹	3.3 × 10 ⁻¹¹			
		M	0.020	5.2 × 10 ⁻¹⁰	0.010	3.6 × 10 ⁻¹⁰	1.7 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰	6.8 × 10 ⁻¹¹	5.6 × 10 ⁻¹¹			
		S	0.020	5.3 × 10 ⁻¹⁰	0.010	3.7 × 10 ⁻¹⁰	1.8 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰	7.1 × 10 ⁻¹¹	5.8 × 10 ⁻¹¹			
Molybdenum													
Mo-90	5.67 h	F	1.000	1.2 × 10 ⁻⁹	0.800	1.1 × 10 ⁻⁹	5.3 × 10 ⁻¹⁰	3.2 × 10 ⁻¹⁰	1.9 × 10 ⁻¹⁰	1.5 × 10 ⁻¹⁰			
		M	0.200	2.6 × 10 ⁻⁹	0.100	2.0 × 10 ⁻⁹	9.9 × 10 ⁻¹⁰	6.5 × 10 ⁻¹⁰	4.2 × 10 ⁻¹⁰	3.4 × 10 ⁻¹⁰			
		S	0.020	2.8 × 10 ⁻⁹	0.010	2.1 × 10 ⁻⁹	1.1 × 10 ⁻⁹	6.9 × 10 ⁻¹⁰	4.5 × 10 ⁻¹⁰	3.6 × 10 ⁻¹⁰			
Mo-93	3.50 × 10 ³ a	F	1.000	3.1 × 10 ⁻⁹	0.800	2.6 × 10 ⁻⁹	1.7 × 10 ⁻⁹	1.3 × 10 ⁻⁹	1.1 × 10 ⁻⁹	1.0 × 10 ⁻⁹			
		M	0.200	2.2 × 10 ⁻⁹	0.100	1.8 × 10 ⁻⁹	1.1 × 10 ⁻⁹	7.9 × 10 ⁻¹⁰	6.6 × 10 ⁻¹⁰	5.9 × 10 ⁻¹⁰			
		S	0.020	6.0 × 10 ⁻⁹	0.010	5.8 × 10 ⁻⁹	4.0 × 10 ⁻⁹	2.8 × 10 ⁻⁹	2.4 × 10 ⁻⁹	2.3 × 10 ⁻⁹			
Mo-93m	6.85 h	F	1.000	7.3 × 10 ⁻¹⁰	0.800	6.4 × 10 ⁻¹⁰	3.3 × 10 ⁻¹⁰	2.0 × 10 ⁻¹⁰	1.2 × 10 ⁻¹⁰	9.6 × 10 ⁻¹¹			
		M	0.200	1.2 × 10 ⁻⁹	0.100	9.7 × 10 ⁻¹⁰	5.0 × 10 ⁻¹⁰	3.2 × 10 ⁻¹⁰	2.0 × 10 ⁻¹⁰	1.6 × 10 ⁻¹⁰			
		S	0.020	1.3 × 10 ⁻⁹	0.010	1.0 × 10 ⁻⁹	5.2 × 10 ⁻¹⁰	3.4 × 10 ⁻¹⁰	2.1 × 10 ⁻¹⁰	1.7 × 10 ⁻¹⁰			
Mo-99	2.75 d	F	1.000	2.3 × 10 ⁻⁹	0.800	1.7 × 10 ⁻⁹	7.7 × 10 ⁻¹⁰	4.7 × 10 ⁻¹⁰	2.6 × 10 ⁻¹⁰	2.2 × 10 ⁻¹⁰			
		M	0.200	6.0 × 10 ⁻⁹	0.100	4.4 × 10 ⁻⁹	2.2 × 10 ⁻⁹	1.5 × 10 ⁻⁹	1.1 × 10 ⁻⁹	8.9 × 10 ⁻¹⁰			
		S	0.020	6.9 × 10 ⁻⁹	0.010	4.8 × 10 ⁻⁹	2.4 × 10 ⁻⁹	1.7 × 10 ⁻⁹	1.2 × 10 ⁻⁹	9.9 × 10 ⁻¹⁰			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Mo-101	0.244 h	F	1.000	1.4×10^{-10}	0.800	9.7×10^{-11}	4.4×10^{-11}	2.8×10^{-11}	1.7×10^{-11}	1.4×10^{-11}							
		M	0.200	2.2×10^{-10}	0.100	1.5×10^{-10}	7.0×10^{-11}	4.5×10^{-11}	3.0×10^{-11}	2.5×10^{-11}							
		S	0.020	2.3×10^{-10}	0.010	1.6×10^{-10}	7.2×10^{-11}	4.7×10^{-11}	3.1×10^{-11}	2.6×10^{-11}							
Technetium																	
Tc-93	2.75 h	F	1.000	2.4×10^{-10}	0.800	2.1×10^{-10}	1.1×10^{-10}	6.7×10^{-11}	4.0×10^{-11}	3.2×10^{-11}							
		M	0.200	2.7×10^{-10}	0.100	2.3×10^{-10}	1.2×10^{-10}	7.5×10^{-11}	4.4×10^{-11}	3.5×10^{-11}							
		S	0.020	2.8×10^{-10}	0.010	2.3×10^{-10}	1.2×10^{-10}	7.6×10^{-11}	4.5×10^{-11}	3.5×10^{-11}							
Tc-93m	0.725 h	F	1.000	1.2×10^{-10}	0.800	9.8×10^{-11}	4.9×10^{-11}	2.9×10^{-11}	1.8×10^{-11}	1.4×10^{-11}							
		M	0.200	1.4×10^{-10}	0.100	1.1×10^{-10}	5.4×10^{-11}	3.4×10^{-11}	2.1×10^{-11}	1.7×10^{-11}							
		S	0.020	1.4×10^{-10}	0.010	1.1×10^{-10}	5.4×10^{-11}	3.4×10^{-11}	2.1×10^{-11}	1.7×10^{-11}							
Tc-94	4.88 h	F	1.000	8.9×10^{-10}	0.800	7.5×10^{-10}	3.9×10^{-10}	2.3×10^{-10}	1.4×10^{-10}	1.1×10^{-10}							
		M	0.200	9.8×10^{-10}	0.100	8.1×10^{-10}	4.2×10^{-10}	2.6×10^{-10}	1.6×10^{-10}	1.2×10^{-10}							
		S	0.020	9.9×10^{-10}	0.010	8.2×10^{-10}	4.3×10^{-10}	2.7×10^{-10}	1.6×10^{-10}	1.3×10^{-10}							
Tc-94m	0.867 h	F	1.000	4.8×10^{-10}	0.800	3.4×10^{-10}	1.6×10^{-10}	8.6×10^{-11}	5.2×10^{-11}	4.1×10^{-11}							
		M	0.200	4.4×10^{-10}	0.100	3.0×10^{-10}	1.4×10^{-10}	8.8×10^{-11}	5.5×10^{-11}	4.5×10^{-11}							
		S	0.020	4.3×10^{-10}	0.010	3.0×10^{-10}	1.4×10^{-10}	8.8×10^{-11}	5.6×10^{-11}	4.6×10^{-11}							
Tc-95	20.0 h	F	1.000	7.5×10^{-10}	0.800	6.3×10^{-10}	3.3×10^{-10}	2.0×10^{-10}	1.2×10^{-10}	9.6×10^{-11}							
		M	0.200	8.3×10^{-10}	0.100	6.9×10^{-10}	3.6×10^{-10}	2.2×10^{-10}	1.3×10^{-10}	1.0×10^{-10}							
		S	0.020	8.5×10^{-10}	0.010	7.0×10^{-10}	3.6×10^{-10}	2.3×10^{-10}	1.4×10^{-10}	1.1×10^{-10}							

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$
Tc-95m	61.0 d	F	1.000	2.4×10^{-9}	0.800	1.8×10^{-9}	9.3×10^{-10}	5.7×10^{-10}	3.6×10^{-10}	2.9×10^{-10}								
		M	0.200	4.9×10^{-9}	0.100	4.0×10^{-9}	2.3×10^{-9}	1.5×10^{-9}	1.1×10^{-9}	8.8×10^{-10}								
		S	0.020	6.0×10^{-9}	0.010	5.0×10^{-9}	2.7×10^{-9}	1.8×10^{-9}	1.5×10^{-9}	1.2×10^{-9}								
Tc-96	4.28 d	F	1.000	4.2×10^{-9}	0.800	3.4×10^{-9}	1.8×10^{-9}	1.1×10^{-9}	7.0×10^{-10}	5.7×10^{-10}								
		M	0.200	4.7×10^{-9}	0.100	3.9×10^{-9}	2.1×10^{-9}	1.3×10^{-9}	8.6×10^{-10}	6.8×10^{-10}								
		S	0.020	4.8×10^{-9}	0.010	3.9×10^{-9}	2.1×10^{-9}	1.4×10^{-9}	8.9×10^{-10}	7.0×10^{-10}								
Tc-96m	0.858 h	F	1.000	5.3×10^{-11}	0.800	4.1×10^{-11}	2.1×10^{-11}	1.3×10^{-11}	7.7×10^{-12}	6.2×10^{-12}								
		M	0.200	5.6×10^{-11}	0.100	4.4×10^{-11}	2.3×10^{-11}	1.4×10^{-11}	9.3×10^{-12}	7.4×10^{-12}								
		S	0.020	5.7×10^{-11}	0.010	4.4×10^{-11}	2.3×10^{-11}	1.5×10^{-11}	9.5×10^{-12}	7.5×10^{-12}								
Tc-97	2.60×10^6 a	F	1.000	5.2×10^{-10}	0.800	3.7×10^{-10}	1.7×10^{-10}	9.4×10^{-11}	5.6×10^{-11}	4.3×10^{-11}								
		M	0.200	1.2×10^{-9}	0.100	1.0×10^{-9}	5.7×10^{-10}	3.6×10^{-10}	2.8×10^{-10}	2.2×10^{-10}								
		S	0.020	5.0×10^{-9}	0.010	4.8×10^{-9}	3.3×10^{-9}	2.2×10^{-9}	1.9×10^{-9}	1.8×10^{-9}								
Tc-97m	87.0 d	F	1.000	3.4×10^{-9}	0.800	2.3×10^{-9}	9.8×10^{-10}	5.6×10^{-10}	3.0×10^{-10}	2.7×10^{-10}								
		M	0.200	1.3×10^{-8}	0.100	1.0×10^{-8}	6.1×10^{-9}	4.4×10^{-9}	4.1×10^{-9}	3.2×10^{-9}								
		S	0.020	1.6×10^{-8}	0.010	1.3×10^{-8}	7.8×10^{-9}	5.7×10^{-9}	5.2×10^{-9}	4.1×10^{-9}								
Tc-98	4.20×10^6 a	F	1.000	1.0×10^{-8}	0.800	6.8×10^{-9}	3.2×10^{-9}	1.9×10^{-9}	1.2×10^{-9}	9.7×10^{-10}								
		M	0.200	3.5×10^{-8}	0.100	2.9×10^{-8}	1.7×10^{-8}	1.2×10^{-8}	1.0×10^{-8}	8.3×10^{-9}								
		S	0.020	1.1×10^{-7}	0.010	1.1×10^{-7}	7.6×10^{-8}	5.4×10^{-8}	4.8×10^{-8}	4.5×10^{-8}								
Tc-99	2.13×10^5 a	F	1.000	4.0×10^{-9}	0.800	2.5×10^{-9}	1.0×10^{-9}	5.9×10^{-10}	3.6×10^{-10}	2.9×10^{-10}								
		M	0.200	1.7×10^{-8}	0.100	1.3×10^{-8}	8.0×10^{-9}	5.7×10^{-9}	5.0×10^{-9}	4.0×10^{-9}								
		S	0.020	4.1×10^{-8}	0.010	3.7×10^{-8}	2.4×10^{-8}	1.7×10^{-8}	1.5×10^{-8}	1.3×10^{-8}								

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)			
Tc-99m	6.02 h	F	1.000	1.2 × 10 ^{−10}	0.800	8.7 × 10 ^{−11}	4.1 × 10 ^{−11}	2.4 × 10 ^{−11}	2.4 × 10 ^{−11}	1.5 × 10 ^{−11}	1.2 × 10 ^{−11}						
		M	0.200	1.3 × 10 ^{−10}	0.100	9.9 × 10 ^{−11}	5.1 × 10 ^{−11}	3.4 × 10 ^{−11}	3.4 × 10 ^{−11}	2.4 × 10 ^{−11}	1.9 × 10 ^{−11}						
		S	0.020	1.3 × 10 ^{−10}	0.010	1.0 × 10 ^{−10}	5.2 × 10 ^{−11}	3.5 × 10 ^{−11}	3.5 × 10 ^{−11}	2.5 × 10 ^{−11}	2.0 × 10 ^{−11}						
Tc-101	0.237 h	F	1.000	8.5 × 10 ^{−11}	0.800	5.6 × 10 ^{−11}	2.5 × 10 ^{−11}	1.6 × 10 ^{−11}	1.6 × 10 ^{−11}	9.7 × 10 ^{−12}	8.2 × 10 ^{−12}						
		M	0.200	1.1 × 10 ^{−10}	0.100	7.1 × 10 ^{−11}	3.2 × 10 ^{−11}	2.1 × 10 ^{−11}	2.1 × 10 ^{−11}	1.4 × 10 ^{−11}	1.2 × 10 ^{−11}						
		S	0.020	1.1 × 10 ^{−10}	0.010	7.3 × 10 ^{−11}	3.3 × 10 ^{−11}	2.2 × 10 ^{−11}	2.2 × 10 ^{−11}	1.4 × 10 ^{−11}	1.2 × 10 ^{−11}						
Tc-104	0.303 h	F	1.000	2.7 × 10 ^{−10}	0.800	1.8 × 10 ^{−10}	8.0 × 10 ^{−11}	4.6 × 10 ^{−11}	4.6 × 10 ^{−11}	2.8 × 10 ^{−11}	2.3 × 10 ^{−11}						
		M	0.200	2.9 × 10 ^{−10}	0.100	1.9 × 10 ^{−10}	8.6 × 10 ^{−11}	5.4 × 10 ^{−11}	5.4 × 10 ^{−11}	3.3 × 10 ^{−11}	2.8 × 10 ^{−11}						
		S	0.020	2.9 × 10 ^{−10}	0.010	1.9 × 10 ^{−10}	8.7 × 10 ^{−11}	5.4 × 10 ^{−11}	5.4 × 10 ^{−11}	3.4 × 10 ^{−11}	2.9 × 10 ^{−11}						
Ruthenium																	
Ru-94	0.863 h	F	0.100	2.5 × 10 ^{−10}	0.050	1.9 × 10 ^{−10}	9.0 × 10 ^{−11}	5.4 × 10 ^{−11}	5.4 × 10 ^{−11}	3.1 × 10 ^{−11}	2.5 × 10 ^{−11}						
		M	0.100	3.8 × 10 ^{−10}	0.050	2.8 × 10 ^{−10}	1.3 × 10 ^{−10}	8.4 × 10 ^{−11}	8.4 × 10 ^{−11}	5.2 × 10 ^{−11}	4.2 × 10 ^{−11}						
		S	0.020	4.0 × 10 ^{−10}	0.010	2.9 × 10 ^{−10}	1.4 × 10 ^{−10}	8.7 × 10 ^{−11}	8.7 × 10 ^{−11}	5.4 × 10 ^{−11}	4.4 × 10 ^{−11}						
Ru-97	2.90 d	F	0.100	5.5 × 10 ^{−10}	0.050	4.4 × 10 ^{−10}	2.2 × 10 ^{−10}	1.3 × 10 ^{−10}	1.3 × 10 ^{−10}	7.7 × 10 ^{−11}	6.2 × 10 ^{−11}						
		M	0.100	7.7 × 10 ^{−10}	0.050	6.1 × 10 ^{−10}	3.1 × 10 ^{−10}	2.0 × 10 ^{−10}	2.0 × 10 ^{−10}	1.3 × 10 ^{−10}	1.0 × 10 ^{−10}						
		S	0.020	8.1 × 10 ^{−10}	0.010	6.3 × 10 ^{−10}	3.3 × 10 ^{−10}	2.1 × 10 ^{−10}	2.1 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}						
Ru-103	39.3 d	F	0.100	4.2 × 10 ^{−9}	0.050	3.0 × 10 ^{−9}	1.5 × 10 ^{−9}	9.3 × 10 ^{−10}	9.3 × 10 ^{−10}	5.6 × 10 ^{−10}	4.8 × 10 ^{−10}						
		M	0.100	1.1 × 10 ^{−8}	0.050	8.4 × 10 ^{−9}	5.0 × 10 ^{−9}	3.5 × 10 ^{−9}	3.5 × 10 ^{−9}	3.0 × 10 ^{−9}	2.4 × 10 ^{−9}						
		S	0.020	1.3 × 10 ^{−8}	0.010	1.0 × 10 ^{−8}	6.0 × 10 ^{−9}	4.2 × 10 ^{−9}	4.2 × 10 ^{−9}	3.7 × 10 ^{−9}	3.0 × 10 ^{−9}						

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1-2 a		2-7 a		7-12 a		12-17 a		<i>e(g)</i>
		Type	<i>f</i> ₁		<i>e(g)</i>		<i>e(g)</i>		<i>e(g)</i>		<i>e(g)</i>		
Ru-105	4.44 h	F	0.100	7.1×10^{-10}	0.050	5.1×10^{-10}	2.3×10^{-10}	1.4×10^{-10}	7.9×10^{-11}	6.5×10^{-11}			
		M	0.100	1.3×10^{-9}	0.050	9.2×10^{-10}	4.5×10^{-10}	3.0×10^{-10}	2.0×10^{-10}	1.7×10^{-10}			
		S	0.020	1.4×10^{-9}	0.010	9.8×10^{-10}	4.8×10^{-10}	3.2×10^{-10}	2.2×10^{-10}	1.8×10^{-10}			
Ru-106	1.01 a	F	0.100	7.2×10^{-8}	0.050	5.4×10^{-8}	2.6×10^{-8}	1.6×10^{-8}	9.2×10^{-9}	7.9×10^{-9}			
		M	0.100	1.4×10^{-7}	0.050	1.1×10^{-7}	6.4×10^{-8}	4.1×10^{-8}	3.1×10^{-8}	2.8×10^{-8}			
		S	0.020	2.6×10^{-7}	0.010	2.3×10^{-7}	1.4×10^{-7}	9.1×10^{-8}	7.1×10^{-8}	6.6×10^{-8}			
Rhodium													
Rh-99	16.0 d	F	0.100	2.6×10^{-9}	0.050	2.0×10^{-9}	9.9×10^{-10}	6.2×10^{-10}	3.8×10^{-10}	3.2×10^{-10}			
		M	0.100	4.5×10^{-9}	0.050	3.5×10^{-9}	2.0×10^{-9}	1.3×10^{-9}	9.6×10^{-10}	7.7×10^{-10}			
		S	0.100	4.9×10^{-9}	0.050	3.8×10^{-9}	2.2×10^{-9}	1.3×10^{-9}	1.1×10^{-9}	8.7×10^{-10}			
Rh-99m	4.70 h	F	0.100	2.4×10^{-10}	0.050	2.0×10^{-10}	1.0×10^{-10}	6.1×10^{-11}	3.5×10^{-11}	2.8×10^{-11}			
		M	0.100	3.1×10^{-10}	0.050	2.5×10^{-10}	1.3×10^{-10}	8.0×10^{-11}	4.9×10^{-11}	3.9×10^{-11}			
		S	0.100	3.2×10^{-10}	0.050	2.6×10^{-10}	1.3×10^{-10}	8.2×10^{-11}	5.1×10^{-11}	4.0×10^{-11}			
Rh-100	20.8 h	F	0.100	2.1×10^{-9}	0.050	1.8×10^{-9}	9.1×10^{-10}	5.6×10^{-10}	3.3×10^{-10}	2.6×10^{-10}			
		M	0.100	2.7×10^{-9}	0.050	2.2×10^{-9}	1.1×10^{-9}	7.1×10^{-10}	4.3×10^{-10}	3.4×10^{-10}			
		S	0.100	2.8×10^{-9}	0.050	2.2×10^{-9}	1.2×10^{-9}	7.3×10^{-10}	4.4×10^{-10}	3.5×10^{-10}			
Rh-101	3.20 a	F	0.100	7.4×10^{-9}	0.050	6.1×10^{-9}	3.5×10^{-9}	2.3×10^{-9}	1.5×10^{-9}	1.4×10^{-9}			
		M	0.100	9.8×10^{-9}	0.050	8.0×10^{-9}	4.9×10^{-9}	3.4×10^{-9}	2.8×10^{-9}	2.3×10^{-9}			
		S	0.100	1.9×10^{-8}	0.050	1.7×10^{-8}	1.1×10^{-8}	7.4×10^{-9}	6.2×10^{-9}	5.4×10^{-9}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Rh-101m	4.34 d	F	0.100	8.4×10^{-10}	0.050	6.6×10^{-10}	3.3×10^{-10}	2.0×10^{-10}	2.0×10^{-10}	1.2×10^{-10}	1.2×10^{-10}	9.7×10^{-11}	
		M	0.100	1.3×10^{-9}	0.050	9.8×10^{-10}	5.2×10^{-10}	3.5×10^{-10}	3.5×10^{-10}	2.5×10^{-10}	2.5×10^{-10}	1.9×10^{-10}	
		S	0.100	1.3×10^{-9}	0.050	1.0×10^{-9}	5.5×10^{-10}	3.7×10^{-10}	3.7×10^{-10}	2.7×10^{-10}	2.7×10^{-10}	2.1×10^{-10}	
Rh-102	2.90 a	F	0.100	3.3×10^{-8}	0.050	2.8×10^{-8}	1.7×10^{-8}	1.1×10^{-8}	1.1×10^{-8}	7.9×10^{-9}	7.9×10^{-9}	7.3×10^{-9}	
		M	0.100	3.0×10^{-8}	0.050	2.5×10^{-8}	1.5×10^{-8}	1.0×10^{-8}	1.0×10^{-8}	7.9×10^{-9}	7.9×10^{-9}	6.9×10^{-9}	
		S	0.100	5.4×10^{-8}	0.050	5.0×10^{-8}	3.5×10^{-8}	2.4×10^{-8}	2.4×10^{-8}	2.0×10^{-8}	2.0×10^{-8}	1.7×10^{-8}	
Rh-102m	207 d	F	0.100	1.2×10^{-8}	0.050	8.7×10^{-9}	4.4×10^{-9}	2.7×10^{-9}	2.7×10^{-9}	1.7×10^{-9}	1.7×10^{-9}	1.5×10^{-9}	
		M	0.100	2.0×10^{-8}	0.050	1.6×10^{-8}	9.0×10^{-9}	6.0×10^{-9}	6.0×10^{-9}	4.7×10^{-9}	4.7×10^{-9}	4.0×10^{-9}	
		S	0.100	3.0×10^{-8}	0.050	2.5×10^{-8}	1.5×10^{-8}	1.0×10^{-8}	1.0×10^{-8}	8.2×10^{-9}	8.2×10^{-9}	7.1×10^{-9}	
Rh-103m	0.935 h	F	0.100	8.6×10^{-12}	0.050	5.9×10^{-12}	2.7×10^{-12}	1.6×10^{-12}	1.6×10^{-12}	1.0×10^{-12}	1.0×10^{-12}	8.6×10^{-13}	
		M	0.100	1.9×10^{-11}	0.050	1.2×10^{-11}	6.3×10^{-12}	4.0×10^{-12}	4.0×10^{-12}	3.0×10^{-12}	3.0×10^{-12}	2.5×10^{-12}	
		S	0.100	2.0×10^{-11}	0.050	1.3×10^{-11}	6.7×10^{-12}	4.3×10^{-12}	4.3×10^{-12}	3.2×10^{-12}	3.2×10^{-12}	2.7×10^{-12}	
Rh-105	1.47 d	F	0.100	1.0×10^{-9}	0.050	6.9×10^{-10}	3.0×10^{-10}	1.8×10^{-10}	1.8×10^{-10}	9.6×10^{-11}	9.6×10^{-11}	8.2×10^{-11}	
		M	0.100	2.2×10^{-9}	0.050	1.6×10^{-9}	7.4×10^{-10}	5.2×10^{-10}	5.2×10^{-10}	4.1×10^{-10}	4.1×10^{-10}	3.2×10^{-10}	
		S	0.100	2.4×10^{-9}	0.050	1.7×10^{-9}	8.0×10^{-10}	5.6×10^{-10}	5.6×10^{-10}	4.5×10^{-10}	4.5×10^{-10}	3.5×10^{-10}	
Rh-106m	2.20 h	F	0.100	5.7×10^{-10}	0.050	4.5×10^{-10}	2.2×10^{-10}	1.4×10^{-10}	1.4×10^{-10}	8.0×10^{-11}	8.0×10^{-11}	6.5×10^{-11}	
		M	0.100	8.2×10^{-10}	0.050	6.3×10^{-10}	3.2×10^{-10}	2.0×10^{-10}	2.0×10^{-10}	1.3×10^{-10}	1.3×10^{-10}	1.1×10^{-10}	
		S	0.100	8.5×10^{-10}	0.050	6.5×10^{-10}	3.3×10^{-10}	2.1×10^{-10}	2.1×10^{-10}	1.4×10^{-10}	1.4×10^{-10}	1.1×10^{-10}	
Rh-107	0.362 h	F	0.100	8.9×10^{-11}	0.050	5.9×10^{-11}	2.6×10^{-11}	1.7×10^{-11}	1.7×10^{-11}	1.0×10^{-11}	1.0×10^{-11}	9.0×10^{-12}	
		M	0.100	1.4×10^{-10}	0.050	9.3×10^{-11}	4.2×10^{-11}	2.8×10^{-11}	2.8×10^{-11}	1.9×10^{-11}	1.9×10^{-11}	1.6×10^{-11}	
		S	0.100	1.5×10^{-10}	0.050	9.7×10^{-11}	4.4×10^{-11}	2.9×10^{-11}	2.9×10^{-11}	1.9×10^{-11}	1.9×10^{-11}	1.7×10^{-11}	

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)		
Palladium														
Pd-100	3.63 d	F	0.050	3.9 × 10 ^{−9}	0.005	3.0 × 10 ^{−9}	1.5 × 10 ^{−9}	9.7 × 10 ^{−10}	5.8 × 10 ^{−10}	4.7 × 10 ^{−10}				
		M	0.050	5.2 × 10 ^{−9}	0.005	4.0 × 10 ^{−9}	2.2 × 10 ^{−9}	1.4 × 10 ^{−9}	9.9 × 10 ^{−10}	8.0 × 10 ^{−10}				
		S	0.050	5.3 × 10 ^{−9}	0.005	4.1 × 10 ^{−9}	2.2 × 10 ^{−9}	1.5 × 10 ^{−9}	1.0 × 10 ^{−9}	8.5 × 10 ^{−10}				
Pd-101	8.27 h	F	0.050	3.6 × 10 ^{−10}	0.005	2.9 × 10 ^{−10}	1.4 × 10 ^{−10}	8.6 × 10 ^{−11}	4.9 × 10 ^{−11}	3.9 × 10 ^{−11}				
		M	0.050	4.8 × 10 ^{−10}	0.005	3.8 × 10 ^{−10}	1.9 × 10 ^{−10}	1.2 × 10 ^{−10}	7.5 × 10 ^{−11}	5.9 × 10 ^{−11}				
		S	0.050	5.0 × 10 ^{−10}	0.005	3.9 × 10 ^{−10}	2.0 × 10 ^{−10}	1.2 × 10 ^{−10}	7.8 × 10 ^{−11}	6.2 × 10 ^{−11}				
Pd-103	17.0 d	F	0.050	9.7 × 10 ^{−10}	0.005	6.5 × 10 ^{−10}	3.0 × 10 ^{−10}	1.9 × 10 ^{−10}	1.1 × 10 ^{−10}	8.9 × 10 ^{−11}				
		M	0.050	2.3 × 10 ^{−9}	0.005	1.6 × 10 ^{−9}	9.0 × 10 ^{−10}	5.9 × 10 ^{−10}	4.5 × 10 ^{−10}	3.8 × 10 ^{−10}				
		S	0.050	2.5 × 10 ^{−9}	0.005	1.8 × 10 ^{−9}	1.0 × 10 ^{−9}	6.8 × 10 ^{−10}	5.3 × 10 ^{−10}	4.5 × 10 ^{−10}				
Pd-107	6.50 × 10 ⁶ a	F	0.050	2.6 × 10 ^{−10}	0.005	1.8 × 10 ^{−10}	8.2 × 10 ^{−11}	5.2 × 10 ^{−11}	3.1 × 10 ^{−11}	2.5 × 10 ^{−11}				
		M	0.050	6.5 × 10 ^{−10}	0.005	5.0 × 10 ^{−10}	2.6 × 10 ^{−10}	1.5 × 10 ^{−10}	1.0 × 10 ^{−10}	8.5 × 10 ^{−11}				
		S	0.050	2.2 × 10 ^{−9}	0.005	2.0 × 10 ^{−9}	1.3 × 10 ^{−9}	7.8 × 10 ^{−10}	6.2 × 10 ^{−10}	5.9 × 10 ^{−10}				
Pd-109	13.4 h	F	0.050	1.5 × 10 ^{−9}	0.005	9.9 × 10 ^{−10}	4.2 × 10 ^{−10}	2.6 × 10 ^{−10}	1.4 × 10 ^{−10}	1.2 × 10 ^{−10}				
		M	0.050	2.6 × 10 ^{−9}	0.005	1.8 × 10 ^{−9}	8.8 × 10 ^{−10}	5.9 × 10 ^{−10}	4.3 × 10 ^{−10}	3.4 × 10 ^{−10}				
		S	0.050	2.7 × 10 ^{−9}	0.005	1.9 × 10 ^{−9}	9.3 × 10 ^{−10}	6.3 × 10 ^{−10}	4.6 × 10 ^{−10}	3.7 × 10 ^{−10}				
Silver														
Ag-102	0.215 h	F	0.100	1.2 × 10 ^{−10}	0.050	8.6 × 10 ^{−11}	4.2 × 10 ^{−11}	2.6 × 10 ^{−11}	1.5 × 10 ^{−11}	1.3 × 10 ^{−11}				
		M	0.100	1.6 × 10 ^{−10}	0.050	1.1 × 10 ^{−10}	5.5 × 10 ^{−11}	3.4 × 10 ^{−11}	2.1 × 10 ^{−11}	1.7 × 10 ^{−11}				
		S	0.020	1.6 × 10 ^{−10}	0.010	1.2 × 10 ^{−10}	5.6 × 10 ^{−11}	3.5 × 10 ^{−11}	2.2 × 10 ^{−11}	1.8 × 10 ^{−11}				

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Ag-103	1.09 h	F	0.100	1.4×10^{-10}	0.050	1.0×10^{-10}	4.9×10^{-11}	3.0×10^{-11}	1.8×10^{-11}	1.4×10^{-11}			
		M	0.100	2.2×10^{-10}	0.050	1.6×10^{-10}	7.6×10^{-11}	4.8×10^{-11}	3.2×10^{-11}	2.6×10^{-11}			
		S	0.020	2.3×10^{-10}	0.010	1.6×10^{-10}	7.9×10^{-11}	5.1×10^{-11}	3.3×10^{-11}	2.7×10^{-11}			
Ag-104	1.15 h	F	0.100	2.3×10^{-10}	0.050	1.9×10^{-10}	9.8×10^{-11}	5.9×10^{-11}	3.5×10^{-11}	2.8×10^{-11}			
		M	0.100	2.9×10^{-10}	0.050	2.3×10^{-10}	1.2×10^{-10}	7.4×10^{-11}	4.5×10^{-11}	3.6×10^{-11}			
		S	0.020	2.9×10^{-10}	0.010	2.4×10^{-10}	1.2×10^{-10}	7.6×10^{-11}	4.6×10^{-11}	3.7×10^{-11}			
Ag-104m	0.558 h	F	0.100	1.6×10^{-10}	0.050	1.1×10^{-10}	5.5×10^{-11}	3.4×10^{-11}	2.0×10^{-11}	1.6×10^{-11}			
		M	0.100	2.3×10^{-10}	0.050	1.6×10^{-10}	7.7×10^{-11}	4.8×10^{-11}	3.0×10^{-11}	2.5×10^{-11}			
		S	0.020	2.4×10^{-10}	0.010	1.7×10^{-10}	8.0×10^{-11}	5.0×10^{-11}	3.1×10^{-11}	2.6×10^{-11}			
Ag-105	41.0 d	F	0.100	3.9×10^{-9}	0.050	3.4×10^{-9}	1.7×10^{-9}	1.0×10^{-9}	6.4×10^{-10}	5.4×10^{-10}			
		M	0.100	4.5×10^{-9}	0.050	3.5×10^{-9}	2.0×10^{-9}	1.3×10^{-9}	9.0×10^{-10}	7.3×10^{-10}			
		S	0.020	4.5×10^{-9}	0.010	3.6×10^{-9}	2.1×10^{-9}	1.3×10^{-9}	1.0×10^{-9}	8.1×10^{-10}			
Ag-106	0.399 h	F	0.100	9.4×10^{-11}	0.050	6.4×10^{-11}	2.9×10^{-11}	1.8×10^{-11}	1.1×10^{-11}	9.1×10^{-12}			
		M	0.100	1.4×10^{-10}	0.050	9.5×10^{-11}	4.4×10^{-11}	2.8×10^{-11}	1.8×10^{-11}	1.5×10^{-11}			
		S	0.020	1.5×10^{-10}	0.010	9.9×10^{-11}	4.5×10^{-11}	2.9×10^{-11}	1.9×10^{-11}	1.6×10^{-11}			
Ag-106m	8.41 d	F	0.100	7.7×10^{-9}	0.050	6.1×10^{-9}	3.2×10^{-9}	2.1×10^{-9}	1.3×10^{-9}	1.1×10^{-9}			
		M	0.100	7.2×10^{-9}	0.050	5.8×10^{-9}	3.2×10^{-9}	2.1×10^{-9}	1.4×10^{-9}	1.1×10^{-9}			
		S	0.020	7.0×10^{-9}	0.010	5.7×10^{-9}	3.2×10^{-9}	2.1×10^{-9}	1.4×10^{-9}	1.1×10^{-9}			
Ag-108m	1.27×10^2 a	F	0.100	3.5×10^{-8}	0.050	2.8×10^{-8}	1.6×10^{-8}	1.0×10^{-8}	6.9×10^{-9}	6.1×10^{-9}			
		M	0.100	3.3×10^{-8}	0.050	2.7×10^{-8}	1.7×10^{-8}	1.1×10^{-8}	8.6×10^{-9}	7.4×10^{-9}			
		S	0.020	8.9×10^{-8}	0.010	8.7×10^{-8}	6.2×10^{-8}	4.4×10^{-8}	3.9×10^{-8}	3.7×10^{-8}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Ag-110m	250 d	F	0.100	3.5×10^{-8}	0.050	2.8×10^{-8}	1.5×10^{-8}	9.7×10^{-9}	6.3×10^{-9}	5.5×10^{-9}				
		M	0.100	3.5×10^{-8}	0.050	2.8×10^{-8}	1.7×10^{-8}	1.2×10^{-8}	9.2×10^{-9}	7.6×10^{-9}				
		S	0.020	4.6×10^{-8}	0.010	4.1×10^{-8}	2.6×10^{-8}	1.8×10^{-8}	1.5×10^{-8}	1.2×10^{-8}				
Ag-111	7.45 d	F	0.100	4.8×10^{-9}	0.050	3.2×10^{-9}	1.4×10^{-9}	8.8×10^{-10}	4.8×10^{-10}	4.0×10^{-10}				
		M	0.100	9.2×10^{-9}	0.050	6.6×10^{-9}	3.5×10^{-9}	2.4×10^{-9}	1.9×10^{-9}	1.5×10^{-9}				
		S	0.020	9.9×10^{-9}	0.010	7.1×10^{-9}	3.8×10^{-9}	2.7×10^{-9}	2.1×10^{-9}	1.7×10^{-9}				
Ag-112	3.12 h	F	0.100	9.8×10^{-10}	0.050	6.4×10^{-10}	2.8×10^{-10}	1.7×10^{-10}	9.1×10^{-11}	7.6×10^{-11}				
		M	0.100	1.7×10^{-9}	0.050	1.1×10^{-9}	5.1×10^{-10}	3.2×10^{-10}	2.0×10^{-10}	1.6×10^{-10}				
		S	0.020	1.8×10^{-9}	0.010	1.2×10^{-9}	5.4×10^{-10}	3.4×10^{-10}	2.1×10^{-10}	1.7×10^{-10}				
Ag-115	0.333 h	F	0.100	1.6×10^{-10}	0.050	1.0×10^{-10}	4.6×10^{-11}	2.9×10^{-11}	1.7×10^{-11}	1.5×10^{-11}				
		M	0.100	2.5×10^{-10}	0.050	1.7×10^{-10}	7.6×10^{-11}	4.9×10^{-11}	3.2×10^{-11}	2.7×10^{-11}				
		S	0.020	2.7×10^{-10}	0.010	1.7×10^{-10}	8.0×10^{-11}	5.2×10^{-11}	3.4×10^{-11}	2.9×10^{-11}				
Cadmium														
Cd-104	0.961 h	F	0.100	2.0×10^{-10}	0.050	1.7×10^{-10}	8.7×10^{-11}	5.2×10^{-11}	3.1×10^{-11}	2.4×10^{-11}				
		M	0.100	2.6×10^{-10}	0.050	2.1×10^{-10}	1.1×10^{-10}	6.9×10^{-11}	4.2×10^{-11}	3.4×10^{-11}				
		S	0.100	2.7×10^{-10}	0.050	2.2×10^{-10}	1.1×10^{-10}	7.0×10^{-11}	4.4×10^{-11}	3.5×10^{-11}				
Cd-107	6.49 h	F	0.100	2.3×10^{-10}	0.050	1.7×10^{-10}	7.4×10^{-11}	4.6×10^{-11}	2.5×10^{-11}	2.1×10^{-11}				
		M	0.100	5.2×10^{-10}	0.050	3.7×10^{-10}	2.0×10^{-10}	1.3×10^{-10}	8.8×10^{-11}	8.3×10^{-11}				
		S	0.100	5.5×10^{-10}	0.050	3.9×10^{-10}	2.1×10^{-10}	1.4×10^{-10}	9.7×10^{-11}	7.7×10^{-11}				

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$
Cd-109	1.27 a	F	0.100	4.5×10^{-8}	0.050	3.7×10^{-8}	2.1×10^{-8}	1.4×10^{-8}	9.3×10^{-9}	8.1×10^{-9}								
		M	0.100	3.0×10^{-8}	0.050	2.3×10^{-8}	1.4×10^{-8}	9.5×10^{-9}	7.8×10^{-9}	6.6×10^{-9}								
		S	0.100	2.7×10^{-8}	0.050	2.1×10^{-8}	1.3×10^{-8}	8.9×10^{-9}	7.6×10^{-9}	6.2×10^{-9}								
Cd-113	9.30×10^{15} a	F	0.100	2.6×10^{-7}	0.050	2.4×10^{-7}	1.7×10^{-7}	1.4×10^{-7}	1.2×10^{-7}	1.2×10^{-7}								
		M	0.100	1.2×10^{-7}	0.050	1.0×10^{-7}	7.6×10^{-8}	6.1×10^{-8}	5.7×10^{-8}	5.5×10^{-8}								
		S	0.100	7.8×10^{-8}	0.050	5.8×10^{-8}	4.1×10^{-8}	3.0×10^{-8}	2.7×10^{-8}	2.6×10^{-8}								
Cd-113m	13.6 a	F	0.100	3.0×10^{-7}	0.050	2.7×10^{-7}	1.8×10^{-7}	1.3×10^{-7}	1.1×10^{-7}	1.1×10^{-7}								
		M	0.100	1.4×10^{-7}	0.050	1.2×10^{-7}	8.1×10^{-8}	6.0×10^{-8}	5.3×10^{-8}	5.2×10^{-8}								
		S	0.100	1.1×10^{-7}	0.050	8.4×10^{-8}	5.5×10^{-8}	3.9×10^{-8}	3.3×10^{-8}	3.1×10^{-8}								
Cd-115	2.23 d	F	0.100	4.0×10^{-9}	0.050	2.6×10^{-9}	1.2×10^{-9}	7.5×10^{-10}	4.3×10^{-10}	3.5×10^{-10}								
		M	0.100	6.7×10^{-9}	0.050	4.8×10^{-9}	2.4×10^{-9}	1.7×10^{-9}	1.2×10^{-9}	9.8×10^{-10}								
		S	0.100	7.2×10^{-9}	0.050	5.1×10^{-9}	2.6×10^{-9}	1.8×10^{-9}	1.3×10^{-9}	1.1×10^{-9}								
Cd-115m	44.6 d	F	0.100	4.6×10^{-8}	0.050	3.2×10^{-8}	1.5×10^{-8}	1.0×10^{-8}	6.4×10^{-9}	5.3×10^{-9}								
		M	0.100	4.0×10^{-8}	0.050	2.5×10^{-8}	1.4×10^{-8}	9.4×10^{-9}	7.3×10^{-9}	6.2×10^{-9}								
		S	0.100	3.9×10^{-8}	0.050	3.0×10^{-8}	1.7×10^{-8}	1.1×10^{-8}	8.9×10^{-9}	7.7×10^{-9}								
Cd-117	2.49 h	F	0.100	7.4×10^{-10}	0.050	5.2×10^{-10}	2.4×10^{-10}	1.5×10^{-10}	8.1×10^{-11}	6.7×10^{-11}								
		M	0.100	1.3×10^{-9}	0.050	9.3×10^{-10}	4.5×10^{-10}	2.9×10^{-10}	2.0×10^{-10}	1.6×10^{-10}								
		S	0.100	1.4×10^{-9}	0.050	9.8×10^{-10}	4.8×10^{-10}	3.1×10^{-10}	2.1×10^{-10}	1.7×10^{-10}								
Cd-117m	3.36 h	F	0.100	8.9×10^{-10}	0.050	6.7×10^{-10}	3.3×10^{-10}	2.0×10^{-10}	1.1×10^{-10}	9.4×10^{-11}								
		M	0.100	1.5×10^{-9}	0.050	1.1×10^{-9}	5.5×10^{-10}	3.6×10^{-10}	2.4×10^{-10}	2.0×10^{-10}								
		S	0.100	1.5×10^{-9}	0.050	1.1×10^{-9}	5.7×10^{-10}	3.8×10^{-10}	2.6×10^{-10}	2.1×10^{-10}								

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		> 17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Indium													
In-109	4.20 h	F	0.040	2.6×10^{-10}	0.020	2.1×10^{-10}	1.0×10^{-10}	6.3×10^{-11}	3.6×10^{-11}			2.9×10^{-11}	
		M	0.040	3.3×10^{-10}	0.020	2.6×10^{-10}	1.3×10^{-10}	8.4×10^{-11}	5.3×10^{-11}			4.2×10^{-11}	
In-110	4.90 h	F	0.040	8.2×10^{-10}	0.020	7.1×10^{-10}	3.7×10^{-10}	2.3×10^{-10}	1.3×10^{-10}			1.1×10^{-10}	
		M	0.040	9.9×10^{-10}	0.020	8.3×10^{-10}	4.4×10^{-10}	2.7×10^{-10}	1.6×10^{-10}			1.3×10^{-10}	
In-110m	1.15 h	F	0.040	3.0×10^{-10}	0.020	2.1×10^{-10}	9.9×10^{-11}	6.0×10^{-11}	3.5×10^{-11}			2.8×10^{-11}	
		M	0.040	4.5×10^{-10}	0.020	3.1×10^{-10}	1.5×10^{-10}	9.2×10^{-11}	5.8×10^{-11}			4.7×10^{-11}	
In-111	2.83 d	F	0.040	1.2×10^{-9}	0.020	8.6×10^{-10}	4.2×10^{-10}	2.6×10^{-10}	1.5×10^{-10}			1.3×10^{-10}	
		M	0.040	1.5×10^{-9}	0.020	1.2×10^{-9}	6.2×10^{-10}	4.1×10^{-10}	2.9×10^{-10}			2.3×10^{-10}	
In-112	0.240 h	F	0.040	4.4×10^{-11}	0.020	3.0×10^{-11}	1.3×10^{-11}	8.7×10^{-12}	5.4×10^{-12}			4.7×10^{-12}	
		M	0.040	6.5×10^{-11}	0.020	4.4×10^{-11}	2.0×10^{-11}	1.3×10^{-11}	8.7×10^{-12}			7.4×10^{-12}	
In-113m	1.66 h	F	0.040	1.0×10^{-10}	0.020	7.0×10^{-11}	3.2×10^{-11}	2.0×10^{-11}	1.2×10^{-11}			9.7×10^{-12}	
		M	0.040	1.6×10^{-10}	0.020	1.1×10^{-10}	5.5×10^{-11}	3.6×10^{-11}	2.4×10^{-11}			2.0×10^{-11}	
In-114m	49.5 d	F	0.040	1.2×10^{-7}	0.020	7.7×10^{-8}	3.4×10^{-8}	1.9×10^{-8}	1.1×10^{-8}			9.3×10^{-9}	
		M	0.040	4.8×10^{-8}	0.020	3.3×10^{-8}	1.6×10^{-8}	1.0×10^{-8}	7.8×10^{-9}			6.1×10^{-9}	
In-115	5.10×10^{15} a	F	0.040	8.3×10^{-7}	0.020	7.8×10^{-7}	5.5×10^{-7}	5.0×10^{-7}	4.2×10^{-7}			3.9×10^{-7}	
		M	0.040	3.0×10^{-7}	0.020	2.8×10^{-7}	2.1×10^{-7}	1.9×10^{-7}	1.7×10^{-7}			1.6×10^{-7}	
In-115m	4.49 h	F	0.040	2.8×10^{-10}	0.020	1.9×10^{-10}	8.4×10^{-11}	5.1×10^{-11}	2.8×10^{-11}			2.4×10^{-11}	
		M	0.040	4.7×10^{-10}	0.020	3.3×10^{-10}	1.6×10^{-10}	1.0×10^{-10}	7.2×10^{-11}			5.9×10^{-11}	
In-116m	0.902 h	F	0.040	2.5×10^{-10}	0.020	1.9×10^{-10}	9.2×10^{-11}	5.7×10^{-11}	3.4×10^{-11}			2.8×10^{-11}	
		M	0.040	3.6×10^{-10}	0.020	2.7×10^{-10}	1.3×10^{-10}	8.5×10^{-11}	5.6×10^{-11}			4.5×10^{-11}	

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)			
In-117	0.730 h	F	0.040	1.4×10^{-10}	0.020	9.7×10^{-11}	4.5×10^{-11}	2.8×10^{-11}	1.7×10^{-11}	1.5×10^{-11}			
		M	0.040	2.3×10^{-10}	0.020	1.6×10^{-10}	7.5×10^{-11}	5.0×10^{-11}	3.5×10^{-11}	2.9×10^{-11}			
In-117m	1.94 h	F	0.040	3.4×10^{-10}	0.020	2.3×10^{-10}	1.0×10^{-10}	6.2×10^{-11}	3.5×10^{-11}	2.9×10^{-11}			
		M	0.040	6.0×10^{-10}	0.020	4.0×10^{-10}	1.9×10^{-10}	1.3×10^{-10}	8.7×10^{-11}	7.2×10^{-11}			
In-119m	0.300 h	F	0.040	1.2×10^{-10}	0.020	7.3×10^{-11}	3.1×10^{-11}	2.0×10^{-11}	1.2×10^{-11}	1.0×10^{-11}			
		M	0.040	1.8×10^{-10}	0.020	1.1×10^{-10}	4.9×10^{-11}	3.2×10^{-11}	2.0×10^{-11}	1.7×10^{-11}			
Tin													
Sn-110	4.00 h	F	0.040	1.0×10^{-9}	0.020	7.6×10^{-10}	3.6×10^{-10}	2.2×10^{-10}	1.2×10^{-10}	9.9×10^{-11}			
		M	0.040	1.5×10^{-9}	0.020	1.1×10^{-9}	5.1×10^{-10}	3.2×10^{-10}	1.9×10^{-10}	1.6×10^{-10}			
Sn-111	0.588 h	F	0.040	7.7×10^{-11}	0.020	5.4×10^{-11}	2.6×10^{-11}	1.6×10^{-11}	9.4×10^{-12}	7.8×10^{-12}			
		M	0.040	1.1×10^{-10}	0.020	8.0×10^{-11}	3.8×10^{-11}	2.5×10^{-11}	1.6×10^{-11}	1.3×10^{-11}			
Sn-113	115 d	F	0.040	5.1×10^{-9}	0.020	3.7×10^{-9}	1.8×10^{-9}	1.1×10^{-9}	6.4×10^{-10}	5.4×10^{-10}			
		M	0.040	1.3×10^{-8}	0.020	1.0×10^{-8}	5.8×10^{-9}	4.0×10^{-9}	3.2×10^{-9}	2.7×10^{-9}			
Sn-117m	13.6 d	F	0.040	3.3×10^{-9}	0.020	2.2×10^{-9}	1.0×10^{-9}	6.1×10^{-10}	3.4×10^{-10}	2.8×10^{-10}			
		M	0.040	1.0×10^{-8}	0.020	7.7×10^{-9}	4.6×10^{-9}	3.4×10^{-9}	3.1×10^{-9}	2.4×10^{-9}			
Sn-119m	293 d	F	0.040	3.0×10^{-9}	0.020	2.2×10^{-9}	1.0×10^{-9}	6.0×10^{-10}	3.4×10^{-10}	2.8×10^{-10}			
		M	0.040	1.0×10^{-8}	0.020	7.9×10^{-9}	4.7×10^{-9}	3.1×10^{-9}	2.6×10^{-9}	2.2×10^{-9}			
Sn-121	1.13 d	F	0.040	7.7×10^{-10}	0.020	5.0×10^{-10}	2.2×10^{-10}	1.3×10^{-10}	7.0×10^{-11}	6.0×10^{-11}			
		M	0.040	1.5×10^{-9}	0.020	1.1×10^{-9}	5.1×10^{-10}	3.6×10^{-10}	2.9×10^{-10}	2.3×10^{-10}			
Sn-121m	55.0 a	F	0.040	6.9×10^{-9}	0.020	5.4×10^{-9}	2.8×10^{-9}	1.6×10^{-9}	9.4×10^{-10}	8.0×10^{-10}			
		M	0.040	1.9×10^{-8}	0.020	1.5×10^{-8}	9.2×10^{-9}	6.4×10^{-9}	5.5×10^{-9}	4.5×10^{-9}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Sn-123	129 d	F	0.040	1.4×10^{-8}	0.020	9.9×10^{-9}	4.5×10^{-9}	2.6×10^{-9}	1.4×10^{-9}	1.4×10^{-9}	1.2×10^{-9}	1.2×10^{-9}	1.2×10^{-9}	
		M	0.040	4.0×10^{-8}	0.020	3.1×10^{-8}	1.8×10^{-8}	1.2×10^{-8}	9.5×10^{-9}	9.5×10^{-9}	8.1×10^{-9}	8.1×10^{-9}		
Sn-123m	0.668 h	F	0.040	1.4×10^{-10}	0.020	8.9×10^{-11}	3.9×10^{-11}	2.5×10^{-11}	1.5×10^{-11}	1.5×10^{-11}	1.3×10^{-11}	1.3×10^{-11}	1.3×10^{-11}	
		M	0.040	2.3×10^{-10}	0.020	1.5×10^{-10}	7.0×10^{-11}	4.6×10^{-11}	3.2×10^{-11}	3.2×10^{-11}	2.7×10^{-11}	2.7×10^{-11}		
Sn-125	9.64 d	F	0.040	1.2×10^{-8}	0.020	8.0×10^{-9}	3.5×10^{-9}	2.0×10^{-9}	1.1×10^{-9}	1.1×10^{-9}	8.9×10^{-10}	8.9×10^{-10}	8.9×10^{-10}	
		M	0.040	2.1×10^{-8}	0.020	1.5×10^{-8}	7.6×10^{-9}	5.0×10^{-9}	3.6×10^{-9}	3.6×10^{-9}	3.1×10^{-9}	3.1×10^{-9}		
Sn-126	1.00×10^5 a	F	0.040	7.3×10^{-8}	0.020	5.9×10^{-8}	3.2×10^{-8}	2.0×10^{-8}	1.3×10^{-8}	1.3×10^{-8}	1.1×10^{-8}	1.1×10^{-8}	1.1×10^{-8}	
		M	0.040	1.2×10^{-7}	0.020	1.0×10^{-7}	6.2×10^{-8}	4.1×10^{-8}	3.3×10^{-8}	3.3×10^{-8}	2.8×10^{-8}	2.8×10^{-8}		
Sn-127	2.10 h	F	0.040	6.6×10^{-10}	0.020	4.7×10^{-10}	2.3×10^{-10}	1.4×10^{-10}	7.9×10^{-11}	7.9×10^{-11}	6.5×10^{-11}	6.5×10^{-11}	6.5×10^{-11}	
		M	0.040	1.0×10^{-9}	0.020	7.4×10^{-10}	3.7×10^{-10}	2.4×10^{-10}	1.6×10^{-10}	1.6×10^{-10}	1.3×10^{-10}	1.3×10^{-10}		
Sn-128	0.985 h	F	0.040	5.1×10^{-10}	0.020	3.6×10^{-10}	1.7×10^{-10}	1.0×10^{-10}	6.1×10^{-11}	6.1×10^{-11}	5.0×10^{-11}	5.0×10^{-11}	5.0×10^{-11}	
		M	0.040	8.0×10^{-10}	0.020	5.5×10^{-10}	2.7×10^{-10}	1.7×10^{-10}	1.1×10^{-10}	1.1×10^{-10}	9.2×10^{-11}	9.2×10^{-11}		
Antimony														
Sb-115	0.530 h	F	0.200	8.1×10^{-11}	0.100	5.9×10^{-11}	2.8×10^{-11}	1.7×10^{-11}	1.0×10^{-11}	1.0×10^{-11}	8.5×10^{-12}	8.5×10^{-12}	8.5×10^{-12}	
		M	0.020	1.2×10^{-10}	0.010	8.3×10^{-11}	4.0×10^{-11}	2.5×10^{-11}	1.6×10^{-11}	1.6×10^{-11}	1.3×10^{-11}	1.3×10^{-11}		
Sb-116	0.263 h	S	0.020	1.2×10^{-10}	0.010	8.6×10^{-11}	4.1×10^{-11}	2.6×10^{-11}	1.7×10^{-11}	1.7×10^{-11}	1.4×10^{-11}	1.4×10^{-11}	1.4×10^{-11}	
		F	0.200	8.4×10^{-11}	0.100	6.2×10^{-11}	3.0×10^{-11}	1.9×10^{-11}	1.1×10^{-11}	1.1×10^{-11}	9.1×10^{-12}	9.1×10^{-12}	9.1×10^{-12}	
		M	0.020	1.1×10^{-10}	0.010	8.2×10^{-11}	4.0×10^{-11}	2.5×10^{-11}	1.5×10^{-11}	1.5×10^{-11}	1.3×10^{-11}	1.3×10^{-11}	1.3×10^{-11}	
		S	0.020	1.2×10^{-10}	0.010	8.5×10^{-11}	4.1×10^{-11}	2.6×10^{-11}	1.6×10^{-11}	1.6×10^{-11}	1.3×10^{-11}	1.3×10^{-11}	1.3×10^{-11}	

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)			
Sb-116m	1.00 h	F	0.200	2.6 × 10 ⁻¹⁰	0.100	2.1 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰	6.6 × 10 ⁻¹¹	4.0 × 10 ⁻¹¹	3.2 × 10 ⁻¹¹				
		M	0.020	3.6 × 10 ⁻¹⁰	0.010	2.8 × 10 ⁻¹⁰	1.5 × 10 ⁻¹⁰	9.1 × 10 ⁻¹¹	5.9 × 10 ⁻¹¹	4.7 × 10 ⁻¹¹				
		S	0.020	3.7 × 10 ⁻¹⁰	0.010	2.9 × 10 ⁻¹⁰	1.5 × 10 ⁻¹⁰	9.4 × 10 ⁻¹¹	6.1 × 10 ⁻¹¹	4.9 × 10 ⁻¹¹				
Sb-117	2.80 h	F	0.200	7.7 × 10 ⁻¹¹	0.100	6.0 × 10 ⁻¹¹	2.9 × 10 ⁻¹¹	1.8 × 10 ⁻¹¹	1.0 × 10 ⁻¹¹	8.5 × 10 ⁻¹²				
		M	0.020	1.2 × 10 ⁻¹⁰	0.010	9.1 × 10 ⁻¹¹	4.6 × 10 ⁻¹¹	3.0 × 10 ⁻¹¹	2.0 × 10 ⁻¹¹	1.6 × 10 ⁻¹¹				
		S	0.020	1.3 × 10 ⁻¹⁰	0.010	9.5 × 10 ⁻¹¹	4.8 × 10 ⁻¹¹	3.1 × 10 ⁻¹¹	2.2 × 10 ⁻¹¹	1.7 × 10 ⁻¹¹				
Sb-118m	5.00 h	F	0.200	7.3 × 10 ⁻¹⁰	0.100	6.2 × 10 ⁻¹⁰	3.3 × 10 ⁻¹⁰	2.0 × 10 ⁻¹⁰	1.2 × 10 ⁻¹⁰	9.3 × 10 ⁻¹¹				
		M	0.020	9.3 × 10 ⁻¹⁰	0.010	7.6 × 10 ⁻¹⁰	4.0 × 10 ⁻¹⁰	2.5 × 10 ⁻¹⁰	1.5 × 10 ⁻¹⁰	1.2 × 10 ⁻¹⁰				
		S	0.020	9.5 × 10 ⁻¹⁰	0.010	7.8 × 10 ⁻¹⁰	4.1 × 10 ⁻¹⁰	2.5 × 10 ⁻¹⁰	1.5 × 10 ⁻¹⁰	1.2 × 10 ⁻¹⁰				
Sb-119	1.59 d	F	0.200	2.7 × 10 ⁻¹⁰	0.100	2.0 × 10 ⁻¹⁰	9.4 × 10 ⁻¹¹	5.5 × 10 ⁻¹¹	2.9 × 10 ⁻¹¹	2.3 × 10 ⁻¹¹				
		M	0.020	4.0 × 10 ⁻¹⁰	0.010	2.8 × 10 ⁻¹⁰	1.3 × 10 ⁻¹⁰	7.9 × 10 ⁻¹¹	4.4 × 10 ⁻¹¹	3.5 × 10 ⁻¹¹				
		S	0.020	4.1 × 10 ⁻¹⁰	0.010	2.9 × 10 ⁻¹⁰	1.4 × 10 ⁻¹⁰	8.2 × 10 ⁻¹¹	4.5 × 10 ⁻¹¹	3.6 × 10 ⁻¹¹				
Sb-120	0.265 h	F	0.200	4.6 × 10 ⁻¹¹	0.100	3.1 × 10 ⁻¹¹	1.4 × 10 ⁻¹¹	8.9 × 10 ⁻¹²	5.4 × 10 ⁻¹²	4.6 × 10 ⁻¹²				
		M	0.020	6.6 × 10 ⁻¹¹	0.010	4.4 × 10 ⁻¹¹	2.0 × 10 ⁻¹¹	1.3 × 10 ⁻¹¹	8.3 × 10 ⁻¹²	7.0 × 10 ⁻¹²				
		S	0.020	6.8 × 10 ⁻¹¹	0.010	4.6 × 10 ⁻¹¹	2.1 × 10 ⁻¹¹	1.4 × 10 ⁻¹¹	8.7 × 10 ⁻¹²	7.3 × 10 ⁻¹²				
Sb-120m	5.76 d	F	0.200	4.1 × 10 ⁻⁹	0.100	3.3 × 10 ⁻⁹	1.8 × 10 ⁻⁹	1.1 × 10 ⁻⁹	6.7 × 10 ⁻¹⁰	5.5 × 10 ⁻¹⁰				
		M	0.020	6.3 × 10 ⁻⁹	0.010	5.0 × 10 ⁻⁹	2.8 × 10 ⁻⁹	1.8 × 10 ⁻⁹	1.3 × 10 ⁻⁹	1.0 × 10 ⁻⁹				
		S	0.020	6.6 × 10 ⁻⁹	0.010	5.3 × 10 ⁻⁹	2.9 × 10 ⁻⁹	1.9 × 10 ⁻⁹	1.4 × 10 ⁻⁹	1.1 × 10 ⁻⁹				
Sb-122	2.70 d	F	0.200	4.2 × 10 ⁻⁹	0.100	2.8 × 10 ⁻⁹	1.4 × 10 ⁻⁹	8.4 × 10 ⁻¹⁰	4.4 × 10 ⁻¹⁰	3.6 × 10 ⁻¹⁰				
		M	0.020	8.3 × 10 ⁻⁹	0.010	5.7 × 10 ⁻⁹	2.8 × 10 ⁻⁹	1.8 × 10 ⁻⁹	1.3 × 10 ⁻⁹	1.0 × 10 ⁻⁹				
		S	0.020	8.8 × 10 ⁻⁹	0.010	6.1 × 10 ⁻⁹	3.0 × 10 ⁻⁹	2.0 × 10 ⁻⁹	1.4 × 10 ⁻⁹	1.1 × 10 ⁻⁹				

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)		
Sb-124	60.2 d	F	0.200	1.2 × 10 ^{−8}	0.100	8.8 × 10 ^{−9}	4.3 × 10 ^{−9}	2.6 × 10 ^{−9}	1.6 × 10 ^{−9}	1.3 × 10 ^{−9}				
		M	0.020	3.1 × 10 ^{−8}	0.010	2.4 × 10 ^{−8}	1.4 × 10 ^{−8}	9.6 × 10 ^{−9}	7.7 × 10 ^{−9}	6.4 × 10 ^{−9}				
		S	0.020	3.9 × 10 ^{−8}	0.010	3.1 × 10 ^{−8}	1.8 × 10 ^{−8}	1.3 × 10 ^{−8}	1.0 × 10 ^{−8}	8.6 × 10 ^{−9}				
Sb-124m	0.337 h	F	0.200	2.7 × 10 ^{−11}	0.100	1.9 × 10 ^{−11}	9.0 × 10 ^{−12}	5.6 × 10 ^{−12}	3.4 × 10 ^{−12}	2.8 × 10 ^{−12}				
		M	0.020	4.3 × 10 ^{−11}	0.010	3.1 × 10 ^{−11}	1.5 × 10 ^{−11}	9.6 × 10 ^{−12}	6.5 × 10 ^{−12}	5.4 × 10 ^{−12}				
		S	0.020	4.6 × 10 ^{−11}	0.010	3.3 × 10 ^{−11}	1.6 × 10 ^{−11}	1.0 × 10 ^{−11}	7.2 × 10 ^{−12}	5.9 × 10 ^{−12}				
Sb-125	2.77 a	F	0.200	8.7 × 10 ^{−9}	0.100	6.8 × 10 ^{−9}	3.7 × 10 ^{−9}	2.3 × 10 ^{−9}	1.5 × 10 ^{−9}	1.4 × 10 ^{−9}				
		M	0.020	2.0 × 10 ^{−8}	0.010	1.6 × 10 ^{−8}	1.0 × 10 ^{−8}	6.8 × 10 ^{−9}	5.8 × 10 ^{−9}	4.8 × 10 ^{−9}				
		S	0.020	4.2 × 10 ^{−8}	0.010	3.8 × 10 ^{−8}	2.4 × 10 ^{−8}	1.6 × 10 ^{−8}	1.4 × 10 ^{−8}	1.2 × 10 ^{−8}				
Sb-126	12.4 d	F	0.200	8.8 × 10 ^{−9}	0.100	6.6 × 10 ^{−9}	3.3 × 10 ^{−9}	2.1 × 10 ^{−9}	1.2 × 10 ^{−9}	1.0 × 10 ^{−9}				
		M	0.020	1.7 × 10 ^{−8}	0.010	1.3 × 10 ^{−8}	7.4 × 10 ^{−9}	5.1 × 10 ^{−9}	3.5 × 10 ^{−9}	2.8 × 10 ^{−9}				
		S	0.020	1.9 × 10 ^{−8}	0.010	1.5 × 10 ^{−8}	8.2 × 10 ^{−9}	5.0 × 10 ^{−9}	4.0 × 10 ^{−9}	3.2 × 10 ^{−9}				
Sb-126m	0.317 h	F	0.200	1.2 × 10 ^{−10}	0.100	8.2 × 10 ^{−11}	3.8 × 10 ^{−11}	2.4 × 10 ^{−11}	1.5 × 10 ^{−11}	1.2 × 10 ^{−11}				
		M	0.020	1.7 × 10 ^{−10}	0.010	1.2 × 10 ^{−10}	5.5 × 10 ^{−11}	3.5 × 10 ^{−11}	2.3 × 10 ^{−11}	1.9 × 10 ^{−11}				
		S	0.020	1.8 × 10 ^{−10}	0.010	1.2 × 10 ^{−10}	5.7 × 10 ^{−11}	3.7 × 10 ^{−11}	2.4 × 10 ^{−11}	2.0 × 10 ^{−11}				
Sb-127	3.85 d	F	0.200	5.1 × 10 ^{−9}	0.100	3.5 × 10 ^{−9}	1.6 × 10 ^{−9}	9.7 × 10 ^{−10}	5.2 × 10 ^{−10}	4.3 × 10 ^{−10}				
		M	0.020	1.0 × 10 ^{−8}	0.010	7.3 × 10 ^{−9}	3.9 × 10 ^{−9}	2.7 × 10 ^{−9}	2.1 × 10 ^{−9}	1.7 × 10 ^{−9}				
		S	0.020	1.1 × 10 ^{−8}	0.010	7.9 × 10 ^{−9}	4.2 × 10 ^{−9}	3.0 × 10 ^{−9}	2.3 × 10 ^{−9}	1.9 × 10 ^{−9}				
Sb-128	9.01 h	F	0.200	2.1 × 10 ^{−9}	0.100	1.7 × 10 ^{−9}	8.3 × 10 ^{−10}	5.1 × 10 ^{−10}	2.9 × 10 ^{−10}	2.3 × 10 ^{−10}				
		M	0.020	3.3 × 10 ^{−9}	0.010	2.5 × 10 ^{−9}	1.2 × 10 ^{−9}	7.9 × 10 ^{−10}	5.0 × 10 ^{−10}	4.0 × 10 ^{−10}				
		S	0.020	3.4 × 10 ^{−9}	0.010	2.6 × 10 ^{−9}	1.3 × 10 ^{−9}	8.3 × 10 ^{−10}	5.2 × 10 ^{−10}	4.2 × 10 ^{−10}				

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Sb-128m	0.173 h	F	0.200	9.8×10^{-11}	0.100	6.9×10^{-11}	3.2×10^{-11}	2.0×10^{-11}	1.2×10^{-11}	1.0×10^{-11}							
		M	0.020	1.3×10^{-10}	0.010	9.2×10^{-11}	4.3×10^{-11}	2.7×10^{-11}	1.7×10^{-11}	1.4×10^{-11}							
		S	0.020	1.4×10^{-10}	0.010	9.4×10^{-11}	4.4×10^{-11}	2.8×10^{-11}	1.8×10^{-11}	1.5×10^{-11}							
Sb-129	4.32 h	F	0.200	1.1×10^{-9}	0.100	8.2×10^{-10}	3.8×10^{-10}	2.3×10^{-10}	1.3×10^{-10}	1.0×10^{-10}							
		M	0.020	2.0×10^{-9}	0.010	1.4×10^{-9}	6.8×10^{-10}	4.4×10^{-10}	2.9×10^{-10}	2.3×10^{-10}							
		S	0.020	2.1×10^{-9}	0.010	1.5×10^{-9}	7.2×10^{-10}	4.6×10^{-10}	3.0×10^{-10}	2.5×10^{-10}							
Sb-130	0.667 h	F	0.200	3.0×10^{-10}	0.100	2.2×10^{-10}	1.1×10^{-10}	6.6×10^{-11}	4.0×10^{-11}	3.3×10^{-11}							
		M	0.020	4.5×10^{-10}	0.010	3.2×10^{-10}	1.6×10^{-10}	9.8×10^{-11}	6.3×10^{-11}	5.1×10^{-11}							
		S	0.020	4.6×10^{-10}	0.010	3.3×10^{-10}	1.6×10^{-10}	1.0×10^{-10}	6.5×10^{-11}	5.3×10^{-11}							
Sb-131	0.383 h	F	0.200	3.5×10^{-10}	0.100	2.8×10^{-10}	1.4×10^{-10}	7.7×10^{-11}	4.6×10^{-11}	3.5×10^{-11}							
		M	0.020	3.9×10^{-10}	0.010	2.6×10^{-10}	1.3×10^{-10}	8.0×10^{-11}	5.3×10^{-11}	4.4×10^{-11}							
		S	0.020	3.8×10^{-10}	0.010	2.6×10^{-10}	1.2×10^{-10}	7.9×10^{-11}	5.3×10^{-11}	4.4×10^{-11}							
Tellurium																	
Te-116	2.49 h	F	0.600	5.3×10^{-10}	0.300	4.2×10^{-10}	2.1×10^{-10}	1.3×10^{-10}	7.2×10^{-11}	5.8×10^{-11}							
		M	0.200	8.6×10^{-10}	0.100	6.4×10^{-10}	3.2×10^{-10}	2.0×10^{-10}	1.3×10^{-10}	1.0×10^{-10}							
		S	0.020	9.1×10^{-10}	0.010	6.7×10^{-10}	3.3×10^{-10}	2.1×10^{-10}	1.4×10^{-10}	1.1×10^{-10}							
Te-121	17.0 d	F	0.600	1.7×10^{-9}	0.300	1.4×10^{-9}	7.2×10^{-10}	4.6×10^{-10}	2.9×10^{-10}	2.4×10^{-10}							
		M	0.200	2.3×10^{-9}	0.100	1.9×10^{-9}	1.0×10^{-9}	6.8×10^{-10}	4.7×10^{-10}	3.8×10^{-10}							
		S	0.020	2.4×10^{-9}	0.010	2.0×10^{-9}	1.1×10^{-9}	7.2×10^{-10}	5.1×10^{-10}	4.1×10^{-10}							

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	
Te-121m	154 d	F	0.600		1.4×10^{-8}	0.300	1.0×10^{-8}	5.3×10^{-9}	3.3×10^{-9}	2.1×10^{-9}	2.1×10^{-9}	1.8×10^{-9}	
		M	0.200		1.9×10^{-8}	0.100	1.5×10^{-8}	8.8×10^{-9}	6.1×10^{-9}	5.1×10^{-9}	5.1×10^{-9}	4.2×10^{-9}	
		S	0.020		2.3×10^{-8}	0.010	1.9×10^{-8}	1.2×10^{-8}	8.1×10^{-9}	6.9×10^{-9}	6.9×10^{-9}	5.7×10^{-9}	
Te-123	1.00×10^{13} a	F	0.600		1.1×10^{-8}	0.300	9.1×10^{-9}	6.2×10^{-9}	4.8×10^{-9}	4.0×10^{-9}	4.0×10^{-9}	3.9×10^{-9}	
		M	0.200		5.6×10^{-9}	0.100	4.4×10^{-9}	3.0×10^{-9}	2.3×10^{-9}	2.0×10^{-9}	2.0×10^{-9}	1.9×10^{-9}	
		S	0.020		5.3×10^{-9}	0.010	5.0×10^{-9}	3.5×10^{-9}	2.4×10^{-9}	2.1×10^{-9}	2.1×10^{-9}	2.0×10^{-9}	
Te-123m	120 d	F	0.600		9.8×10^{-9}	0.300	6.8×10^{-9}	3.4×10^{-9}	1.9×10^{-9}	1.1×10^{-9}	1.1×10^{-9}	9.5×10^{-10}	
		M	0.200		1.8×10^{-8}	0.100	1.3×10^{-8}	8.0×10^{-9}	5.7×10^{-9}	5.0×10^{-9}	5.0×10^{-9}	4.0×10^{-9}	
		S	0.020		2.0×10^{-8}	0.010	1.6×10^{-8}	9.8×10^{-9}	7.1×10^{-9}	6.3×10^{-9}	6.3×10^{-9}	5.1×10^{-9}	
Te-125m	58.0 d	F	0.600		6.2×10^{-9}	0.300	4.2×10^{-9}	2.0×10^{-9}	1.1×10^{-9}	6.1×10^{-10}	6.1×10^{-10}	5.1×10^{-10}	
		M	0.200		1.5×10^{-8}	0.100	1.1×10^{-8}	6.6×10^{-9}	4.8×10^{-9}	4.3×10^{-9}	4.3×10^{-9}	3.4×10^{-9}	
		S	0.020		1.7×10^{-8}	0.010	1.3×10^{-8}	7.8×10^{-9}	5.8×10^{-9}	5.3×10^{-9}	5.3×10^{-9}	4.2×10^{-9}	
Te-127	9.35 h	F	0.600		4.3×10^{-10}	0.300	3.2×10^{-10}	1.4×10^{-10}	8.5×10^{-11}	4.5×10^{-11}	4.5×10^{-11}	3.9×10^{-11}	
		M	0.200		1.0×10^{-9}	0.100	7.3×10^{-10}	3.6×10^{-10}	2.4×10^{-10}	1.6×10^{-10}	1.6×10^{-10}	1.3×10^{-10}	
		S	0.020		1.2×10^{-9}	0.010	7.9×10^{-10}	3.9×10^{-10}	2.6×10^{-10}	1.7×10^{-10}	1.7×10^{-10}	1.4×10^{-10}	
Te-127m	109 d	F	0.600		2.1×10^{-8}	0.300	1.4×10^{-8}	6.5×10^{-9}	3.5×10^{-9}	2.0×10^{-9}	2.0×10^{-9}	1.5×10^{-9}	
		M	0.200		3.5×10^{-8}	0.100	2.6×10^{-8}	1.5×10^{-8}	1.1×10^{-8}	9.2×10^{-9}	9.2×10^{-9}	7.4×10^{-9}	
		S	0.020		4.1×10^{-8}	0.010	3.3×10^{-8}	2.0×10^{-8}	1.4×10^{-8}	1.2×10^{-8}	1.2×10^{-8}	9.8×10^{-9}	
Te-129	1.16 h	F	0.600		1.8×10^{-10}	0.300	1.2×10^{-10}	5.1×10^{-11}	3.2×10^{-11}	1.9×10^{-11}	1.9×10^{-11}	1.6×10^{-11}	
		M	0.200		3.3×10^{-10}	0.100	2.2×10^{-10}	9.9×10^{-11}	6.5×10^{-11}	4.4×10^{-11}	4.4×10^{-11}	3.7×10^{-11}	
		S	0.020		3.5×10^{-10}	0.010	2.3×10^{-10}	1.0×10^{-10}	6.9×10^{-11}	4.7×10^{-11}	4.7×10^{-11}	3.9×10^{-11}	

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Te-129m	33.6 d	F	0.600	2.0×10^{-8}	0.300	1.3×10^{-8}	5.8×10^{-9}	3.1×10^{-9}	1.7×10^{-9}	1.3×10^{-9}			
		M	0.200	3.5×10^{-8}	0.100	2.6×10^{-8}	1.4×10^{-8}	9.8×10^{-9}	8.0×10^{-9}	6.6×10^{-9}			
		S	0.020	3.8×10^{-8}	0.010	2.9×10^{-8}	1.7×10^{-8}	1.2×10^{-8}	9.6×10^{-9}	7.9×10^{-9}			
Te-131	0.417 h	F	0.600	2.3×10^{-10}	0.300	2.0×10^{-10}	9.9×10^{-11}	5.3×10^{-11}	3.3×10^{-11}	2.3×10^{-11}			
		M	0.200	2.6×10^{-10}	0.100	1.7×10^{-10}	8.1×10^{-11}	5.2×10^{-11}	3.5×10^{-11}	2.8×10^{-11}			
		S	0.020	2.4×10^{-10}	0.010	1.6×10^{-10}	7.4×10^{-11}	4.9×10^{-11}	3.3×10^{-11}	2.8×10^{-11}			
Te-131m	1.25 d	F	0.600	8.7×10^{-9}	0.300	7.6×10^{-9}	3.9×10^{-9}	2.0×10^{-9}	1.2×10^{-9}	8.6×10^{-10}			
		M	0.200	7.9×10^{-9}	0.100	5.8×10^{-9}	3.0×10^{-9}	1.9×10^{-9}	1.2×10^{-9}	9.4×10^{-10}			
		S	0.020	7.0×10^{-9}	0.010	5.1×10^{-9}	2.6×10^{-9}	1.8×10^{-9}	1.1×10^{-9}	9.1×10^{-10}			
Te-132	3.26 d	F	0.600	2.2×10^{-8}	0.300	1.8×10^{-8}	8.5×10^{-9}	4.2×10^{-9}	2.6×10^{-9}	1.8×10^{-9}			
		M	0.200	1.6×10^{-8}	0.100	1.3×10^{-8}	6.4×10^{-9}	4.0×10^{-9}	2.6×10^{-9}	2.0×10^{-9}			
		S	0.020	1.5×10^{-8}	0.010	1.1×10^{-8}	5.8×10^{-9}	3.8×10^{-9}	2.5×10^{-9}	2.0×10^{-9}			
Te-133	0.207 h	F	0.600	2.4×10^{-10}	0.300	2.1×10^{-10}	9.6×10^{-11}	4.6×10^{-11}	2.8×10^{-11}	1.9×10^{-11}			
		M	0.200	2.0×10^{-10}	0.100	1.3×10^{-10}	6.1×10^{-11}	3.8×10^{-11}	2.4×10^{-11}	2.0×10^{-11}			
		S	0.020	1.7×10^{-10}	0.010	1.2×10^{-10}	5.4×10^{-11}	3.5×10^{-11}	2.2×10^{-11}	1.9×10^{-11}			
Te-133m	0.923 h	F	0.600	1.0×10^{-9}	0.300	8.9×10^{-10}	4.1×10^{-10}	2.0×10^{-10}	1.2×10^{-10}	8.1×10^{-11}			
		M	0.200	8.5×10^{-10}	0.100	5.8×10^{-10}	2.8×10^{-10}	1.7×10^{-10}	1.1×10^{-10}	8.7×10^{-11}			
		S	0.020	7.4×10^{-10}	0.010	5.1×10^{-10}	2.5×10^{-10}	1.6×10^{-10}	1.0×10^{-10}	8.4×10^{-11}			
Te-134	0.696 h	F	0.600	4.7×10^{-10}	0.300	3.7×10^{-10}	1.8×10^{-10}	1.0×10^{-10}	6.0×10^{-11}	4.7×10^{-11}			
		M	0.200	5.5×10^{-10}	0.100	3.9×10^{-10}	1.9×10^{-10}	1.2×10^{-10}	8.1×10^{-11}	6.6×10^{-11}			
		S	0.020	5.6×10^{-10}	0.010	4.0×10^{-10}	1.9×10^{-10}	1.3×10^{-10}	8.4×10^{-11}	6.8×10^{-11}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Iodine														
I-120	1.35 h	F	1.000	1.3×10^{-9}	1.000	1.0×10^{-9}	4.8×10^{-10}	2.3×10^{-10}	2.3×10^{-10}	1.4×10^{-10}	1.0×10^{-10}	1.0×10^{-10}	1.0×10^{-10}	1.0×10^{-10}
		M	0.200	1.1×10^{-9}	0.100	7.3×10^{-10}	3.4×10^{-10}	2.1×10^{-10}	2.1×10^{-10}	1.3×10^{-10}	1.0×10^{-10}	1.0×10^{-10}	1.0×10^{-10}	1.0×10^{-10}
		S	0.020	1.0×10^{-9}	0.010	6.9×10^{-10}	3.2×10^{-10}	2.0×10^{-10}	2.0×10^{-10}	1.2×10^{-10}	1.0×10^{-10}	1.0×10^{-10}	1.0×10^{-10}	1.0×10^{-10}
I-120m	0.883 h	F	1.000	8.6×10^{-10}	1.000	6.9×10^{-10}	3.3×10^{-10}	1.8×10^{-10}	1.8×10^{-10}	1.1×10^{-10}	8.2×10^{-11}	8.2×10^{-11}	8.2×10^{-11}	8.2×10^{-11}
		M	0.200	8.2×10^{-10}	0.100	5.9×10^{-10}	2.9×10^{-10}	1.8×10^{-10}	1.8×10^{-10}	1.1×10^{-10}	8.7×10^{-11}	8.7×10^{-11}	8.7×10^{-11}	8.7×10^{-11}
		S	0.020	8.2×10^{-10}	0.010	5.8×10^{-10}	2.8×10^{-10}	1.8×10^{-10}	1.8×10^{-10}	1.1×10^{-10}	8.8×10^{-11}	8.8×10^{-11}	8.8×10^{-11}	8.8×10^{-11}
I-121	2.12 h	F	1.000	2.3×10^{-10}	1.000	2.1×10^{-10}	1.1×10^{-10}	6.0×10^{-11}	6.0×10^{-11}	3.8×10^{-11}	2.7×10^{-11}	2.7×10^{-11}	2.7×10^{-11}	2.7×10^{-11}
		M	0.200	2.1×10^{-10}	0.100	1.5×10^{-10}	7.8×10^{-11}	4.9×10^{-11}	4.9×10^{-11}	3.2×10^{-11}	2.5×10^{-11}	2.5×10^{-11}	2.5×10^{-11}	2.5×10^{-11}
		S	0.020	1.9×10^{-10}	0.010	1.4×10^{-10}	7.0×10^{-11}	4.5×10^{-11}	4.5×10^{-11}	3.0×10^{-11}	2.4×10^{-11}	2.4×10^{-11}	2.4×10^{-11}	2.4×10^{-11}
I-123	13.2 h	F	1.000	8.7×10^{-10}	1.000	7.9×10^{-10}	3.8×10^{-10}	1.8×10^{-10}	1.8×10^{-10}	1.1×10^{-10}	7.4×10^{-11}	7.4×10^{-11}	7.4×10^{-11}	7.4×10^{-11}
		M	0.200	5.3×10^{-10}	0.100	3.9×10^{-10}	2.0×10^{-10}	1.2×10^{-10}	1.2×10^{-10}	8.2×10^{-11}	6.4×10^{-11}	6.4×10^{-11}	6.4×10^{-11}	6.4×10^{-11}
		S	0.020	4.3×10^{-10}	0.010	3.2×10^{-10}	1.7×10^{-10}	1.1×10^{-10}	1.1×10^{-10}	7.6×10^{-11}	6.0×10^{-11}	6.0×10^{-11}	6.0×10^{-11}	6.0×10^{-11}
I-124	4.18 d	F	1.000	4.7×10^{-8}	1.000	4.5×10^{-8}	2.2×10^{-8}	1.1×10^{-8}	1.1×10^{-8}	6.7×10^{-9}	4.4×10^{-9}	4.4×10^{-9}	4.4×10^{-9}	4.4×10^{-9}
		M	0.200	1.4×10^{-8}	0.100	9.3×10^{-9}	4.6×10^{-9}	2.5×10^{-9}	2.5×10^{-9}	1.6×10^{-9}	1.2×10^{-9}	1.2×10^{-9}	1.2×10^{-9}	1.2×10^{-9}
		S	0.020	6.2×10^{-9}	0.010	4.4×10^{-9}	2.2×10^{-9}	1.4×10^{-9}	1.4×10^{-9}	9.4×10^{-10}	7.7×10^{-10}	7.7×10^{-10}	7.7×10^{-10}	7.7×10^{-10}
I-125	60.1 d	F	1.000	2.0×10^{-8}	1.000	2.3×10^{-8}	1.5×10^{-8}	1.1×10^{-8}	1.1×10^{-8}	7.2×10^{-9}	5.1×10^{-9}	5.1×10^{-9}	5.1×10^{-9}	5.1×10^{-9}
		M	0.200	6.9×10^{-9}	0.100	5.6×10^{-9}	3.6×10^{-9}	2.6×10^{-9}	2.6×10^{-9}	1.8×10^{-9}	1.4×10^{-9}	1.4×10^{-9}	1.4×10^{-9}	1.4×10^{-9}
		S	0.020	2.4×10^{-9}	0.010	1.8×10^{-9}	1.0×10^{-9}	6.7×10^{-10}	6.7×10^{-10}	4.8×10^{-10}	3.8×10^{-10}	3.8×10^{-10}	3.8×10^{-10}	3.8×10^{-10}
I-126	13.0 d	F	1.000	8.1×10^{-8}	1.000	8.3×10^{-8}	4.5×10^{-8}	2.4×10^{-8}	2.4×10^{-8}	1.5×10^{-8}	9.8×10^{-9}	9.8×10^{-9}	9.8×10^{-9}	9.8×10^{-9}
		M	0.200	2.4×10^{-8}	0.100	1.7×10^{-8}	9.5×10^{-9}	5.5×10^{-9}	5.5×10^{-9}	3.8×10^{-9}	2.7×10^{-9}	2.7×10^{-9}	2.7×10^{-9}	2.7×10^{-9}
		S	0.020	8.3×10^{-9}	0.010	5.9×10^{-9}	3.3×10^{-9}	2.2×10^{-9}	2.2×10^{-9}	1.8×10^{-9}	1.4×10^{-9}	1.4×10^{-9}	1.4×10^{-9}	1.4×10^{-9}

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)		
I-128	0.416 h	F	1.000	1.5 × 10 ^{−10}	1.000	1.1 × 10 ^{−10}	4.7 × 10 ^{−11}	2.7 × 10 ^{−11}	1.6 × 10 ^{−11}	1.3 × 10 ^{−11}				
		M	0.200	1.9 × 10 ^{−10}	0.100	1.2 × 10 ^{−10}	5.3 × 10 ^{−11}	3.4 × 10 ^{−11}	2.2 × 10 ^{−11}	1.9 × 10 ^{−11}				
		S	0.020	1.9 × 10 ^{−10}	0.010	1.2 × 10 ^{−10}	5.4 × 10 ^{−11}	3.5 × 10 ^{−11}	2.3 × 10 ^{−11}	2.0 × 10 ^{−11}				
I-129	1.57 × 10 ⁷ a	F	1.000	7.2 × 10 ^{−8}	1.000	8.6 × 10 ^{−8}	6.1 × 10 ^{−8}	6.7 × 10 ^{−8}	4.6 × 10 ^{−8}	3.6 × 10 ^{−8}				
		M	0.200	3.6 × 10 ^{−8}	0.100	3.3 × 10 ^{−8}	2.4 × 10 ^{−8}	2.4 × 10 ^{−8}	1.9 × 10 ^{−8}	1.5 × 10 ^{−8}				
		S	0.020	2.9 × 10 ^{−8}	0.010	2.6 × 10 ^{−8}	1.8 × 10 ^{−8}	1.3 × 10 ^{−8}	1.1 × 10 ^{−8}	9.8 × 10 ^{−9}				
I-130	12.4 h	F	1.000	8.2 × 10 ^{−9}	1.000	7.4 × 10 ^{−9}	3.5 × 10 ^{−9}	1.6 × 10 ^{−9}	1.0 × 10 ^{−9}	6.7 × 10 ^{−10}				
		M	0.200	4.3 × 10 ^{−9}	0.100	3.1 × 10 ^{−9}	1.5 × 10 ^{−9}	9.2 × 10 ^{−10}	5.8 × 10 ^{−10}	4.5 × 10 ^{−10}				
		S	0.020	3.3 × 10 ^{−9}	0.010	2.4 × 10 ^{−9}	1.2 × 10 ^{−9}	7.9 × 10 ^{−10}	5.1 × 10 ^{−10}	4.1 × 10 ^{−10}				
I-131	8.04 d	F	1.000	7.2 × 10 ^{−8}	1.000	7.2 × 10 ^{−8}	3.7 × 10 ^{−8}	1.9 × 10 ^{−8}	1.1 × 10 ^{−8}	7.4 × 10 ^{−9}				
		M	0.200	2.2 × 10 ^{−8}	0.100	1.5 × 10 ^{−8}	8.2 × 10 ^{−9}	4.7 × 10 ^{−9}	3.4 × 10 ^{−9}	2.4 × 10 ^{−9}				
		S	0.020	8.8 × 10 ^{−9}	0.010	6.2 × 10 ^{−9}	3.5 × 10 ^{−9}	2.4 × 10 ^{−9}	2.0 × 10 ^{−9}	1.6 × 10 ^{−9}				
I-132	2.30 h	F	1.000	1.1 × 10 ^{−9}	1.000	9.6 × 10 ^{−10}	4.5 × 10 ^{−10}	2.2 × 10 ^{−10}	1.3 × 10 ^{−10}	9.4 × 10 ^{−11}				
		M	0.200	9.9 × 10 ^{−10}	0.100	7.3 × 10 ^{−10}	3.6 × 10 ^{−10}	2.2 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}				
		S	0.020	9.3 × 10 ^{−10}	0.010	6.8 × 10 ^{−10}	3.4 × 10 ^{−10}	2.1 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}				
I-132m	1.39 h	F	1.000	9.6 × 10 ^{−10}	1.000	8.4 × 10 ^{−10}	4.0 × 10 ^{−10}	1.9 × 10 ^{−10}	1.2 × 10 ^{−10}	7.9 × 10 ^{−11}				
		M	0.200	7.2 × 10 ^{−10}	0.100	5.3 × 10 ^{−10}	2.6 × 10 ^{−10}	1.6 × 10 ^{−10}	1.1 × 10 ^{−10}	8.7 × 10 ^{−11}				
		S	0.020	6.6 × 10 ^{−10}	0.010	4.8 × 10 ^{−10}	2.4 × 10 ^{−10}	1.6 × 10 ^{−10}	1.1 × 10 ^{−10}	8.5 × 10 ^{−11}				
I-133	20.8 h	F	1.000	1.9 × 10 ^{−8}	1.000	1.8 × 10 ^{−8}	8.3 × 10 ^{−9}	3.8 × 10 ^{−9}	2.2 × 10 ^{−9}	1.5 × 10 ^{−9}				
		M	0.200	6.6 × 10 ^{−9}	0.100	4.4 × 10 ^{−9}	2.1 × 10 ^{−9}	1.2 × 10 ^{−9}	7.4 × 10 ^{−10}	5.5 × 10 ^{−10}				
		S	0.020	3.8 × 10 ^{−9}	0.010	2.9 × 10 ^{−9}	1.4 × 10 ^{−9}	9.0 × 10 ^{−10}	5.3 × 10 ^{−10}	4.3 × 10 ^{−10}				

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		<i>e</i> (g)
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)			
I-134	0.876 h	F	1.000	4.6×10^{-10}	1.000	3.7×10^{-10}	1.8×10^{-10}	9.7×10^{-11}	5.9×10^{-11}	5.9×10^{-11}	4.5×10^{-11}	>17 a	
		M	0.200	4.8×10^{-10}	0.100	3.4×10^{-10}	1.7×10^{-10}	1.0×10^{-10}	6.7×10^{-11}	6.7×10^{-11}	5.4×10^{-11}		
		S	0.020	4.8×10^{-10}	0.010	3.4×10^{-10}	1.7×10^{-10}	1.1×10^{-10}	6.8×10^{-11}	6.8×10^{-11}	5.5×10^{-11}		
I-135	6.61 h	F	1.000	4.1×10^{-9}	1.000	3.7×10^{-9}	1.7×10^{-9}	7.9×10^{-10}	4.8×10^{-10}	4.8×10^{-10}	3.2×10^{-10}	>17 a	
		M	0.200	2.2×10^{-9}	0.100	1.6×10^{-9}	7.8×10^{-10}	4.7×10^{-10}	3.0×10^{-10}	3.0×10^{-10}	2.4×10^{-10}		
		S	0.020	1.8×10^{-9}	0.010	1.3×10^{-9}	6.5×10^{-10}	4.2×10^{-10}	2.7×10^{-10}	2.7×10^{-10}	2.2×10^{-10}		
Caesium													
Cs-125	0.750 h	F	1.000	1.2×10^{-10}	1.000	8.3×10^{-11}	3.9×10^{-11}	2.4×10^{-11}	1.4×10^{-11}	1.4×10^{-11}	1.2×10^{-11}	>17 a	
		M	0.200	2.0×10^{-10}	0.100	1.4×10^{-10}	6.5×10^{-11}	4.2×10^{-11}	2.7×10^{-11}	2.7×10^{-11}	2.2×10^{-11}		
		S	0.020	2.1×10^{-10}	0.010	1.4×10^{-10}	6.8×10^{-11}	4.4×10^{-11}	2.8×10^{-11}	2.8×10^{-11}	2.3×10^{-11}		
Cs-127	6.25 h	F	1.000	1.6×10^{-10}	1.000	1.3×10^{-10}	6.9×10^{-11}	4.2×10^{-11}	2.5×10^{-11}	2.5×10^{-11}	2.0×10^{-11}	>17 a	
		M	0.200	2.8×10^{-10}	0.100	2.2×10^{-10}	1.1×10^{-10}	7.3×10^{-11}	4.6×10^{-11}	4.6×10^{-11}	3.6×10^{-11}		
		S	0.020	3.0×10^{-10}	0.010	2.3×10^{-10}	1.2×10^{-10}	7.6×10^{-11}	4.8×10^{-11}	4.8×10^{-11}	3.8×10^{-11}		
Cs-129	1.34 d	F	1.000	3.4×10^{-10}	1.000	2.8×10^{-10}	1.4×10^{-10}	8.7×10^{-11}	5.2×10^{-11}	5.2×10^{-11}	4.2×10^{-11}	>17 a	
		M	0.200	5.7×10^{-10}	0.100	4.6×10^{-10}	2.4×10^{-10}	1.5×10^{-10}	9.1×10^{-11}	9.1×10^{-11}	7.3×10^{-11}		
		S	0.020	6.3×10^{-10}	0.010	4.9×10^{-10}	2.5×10^{-10}	1.6×10^{-10}	9.7×10^{-11}	9.7×10^{-11}	7.7×10^{-11}		
Cs-130	0.498 h	F	1.000	8.3×10^{-11}	1.000	5.6×10^{-11}	2.5×10^{-11}	1.6×10^{-11}	9.4×10^{-12}	9.4×10^{-12}	7.8×10^{-12}	>17 a	
		M	0.200	1.3×10^{-10}	0.100	8.7×10^{-11}	4.0×10^{-11}	2.5×10^{-11}	1.6×10^{-11}	1.6×10^{-11}	1.4×10^{-11}		
		S	0.020	1.4×10^{-10}	0.010	9.0×10^{-11}	4.1×10^{-11}	2.6×10^{-11}	1.7×10^{-11}	1.7×10^{-11}	1.4×10^{-11}		

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Cs-131	9.69 d	F	1.000	2.4×10^{-10}	1.000	1.7×10^{-10}	8.4×10^{-11}	5.3×10^{-11}	3.2×10^{-11}	3.2×10^{-11}	2.7×10^{-11}			
		M	0.200	3.5×10^{-10}	0.100	2.6×10^{-10}	1.4×10^{-10}	8.5×10^{-11}	5.5×10^{-11}	5.5×10^{-11}	4.4×10^{-11}			
		S	0.020	3.8×10^{-10}	0.010	2.8×10^{-10}	1.4×10^{-10}	9.1×10^{-11}	5.9×10^{-11}	5.9×10^{-11}	4.7×10^{-11}			
Cs-132	6.48 d	F	1.000	1.5×10^{-9}	1.000	1.2×10^{-9}	6.4×10^{-10}	4.1×10^{-10}	2.7×10^{-10}	2.7×10^{-10}	2.3×10^{-10}			
		M	0.200	1.9×10^{-9}	0.100	1.5×10^{-9}	8.4×10^{-10}	5.4×10^{-10}	3.7×10^{-10}	3.7×10^{-10}	2.9×10^{-10}			
		S	0.020	2.0×10^{-9}	0.010	1.6×10^{-9}	8.7×10^{-10}	5.6×10^{-10}	3.8×10^{-10}	3.8×10^{-10}	3.0×10^{-10}			
Cs-134	2.06 a	F	1.000	1.1×10^{-8}	1.000	7.3×10^{-9}	5.2×10^{-9}	5.3×10^{-9}	6.3×10^{-9}	6.3×10^{-9}	6.6×10^{-9}			
		M	0.200	3.2×10^{-8}	0.100	2.6×10^{-8}	1.6×10^{-8}	1.2×10^{-8}	1.1×10^{-8}	1.1×10^{-8}	9.1×10^{-9}			
		S	0.020	7.0×10^{-8}	0.010	6.3×10^{-8}	4.1×10^{-8}	2.8×10^{-8}	2.3×10^{-8}	2.3×10^{-8}	2.0×10^{-8}			
Cs-134m	2.90 h	F	1.000	1.3×10^{-10}	1.000	8.6×10^{-11}	3.8×10^{-11}	2.5×10^{-11}	1.6×10^{-11}	1.6×10^{-11}	1.4×10^{-11}			
		M	0.200	3.3×10^{-10}	0.100	2.3×10^{-10}	1.2×10^{-10}	8.3×10^{-11}	6.6×10^{-11}	6.6×10^{-11}	5.4×10^{-11}			
		S	0.020	3.6×10^{-10}	0.010	2.5×10^{-10}	1.3×10^{-10}	9.2×10^{-11}	7.4×10^{-11}	7.4×10^{-11}	6.0×10^{-11}			
Cs-135	2.30×10^6 a	F	1.000	1.7×10^{-9}	1.000	9.9×10^{-10}	6.2×10^{-10}	6.1×10^{-10}	6.8×10^{-10}	6.8×10^{-10}	6.9×10^{-10}			
		M	0.200	1.2×10^{-8}	0.100	9.3×10^{-9}	5.7×10^{-9}	4.1×10^{-9}	3.8×10^{-9}	3.8×10^{-9}	3.1×10^{-9}			
		S	0.020	2.7×10^{-8}	0.010	2.4×10^{-8}	1.6×10^{-8}	1.1×10^{-8}	9.5×10^{-9}	9.5×10^{-9}	8.6×10^{-9}			
Cs-135m	0.883 h	F	1.000	9.2×10^{-11}	1.000	7.8×10^{-11}	4.1×10^{-11}	2.4×10^{-11}	1.5×10^{-11}	1.5×10^{-11}	1.2×10^{-11}			
		M	0.200	1.2×10^{-10}	0.100	9.9×10^{-11}	5.2×10^{-11}	3.2×10^{-11}	1.9×10^{-11}	1.9×10^{-11}	1.5×10^{-11}			
		S	0.020	1.2×10^{-10}	0.010	1.0×10^{-10}	5.3×10^{-11}	3.3×10^{-11}	2.0×10^{-11}	2.0×10^{-11}	1.6×10^{-11}			
Cs-136	13.1 d	F	1.000	7.3×10^{-9}	1.000	5.2×10^{-9}	2.9×10^{-9}	2.0×10^{-9}	1.4×10^{-9}	1.4×10^{-9}	1.2×10^{-9}			
		M	0.200	1.3×10^{-8}	0.100	1.0×10^{-8}	6.0×10^{-9}	3.7×10^{-9}	3.1×10^{-9}	3.1×10^{-9}	2.5×10^{-9}			
		S	0.020	1.5×10^{-8}	0.010	1.1×10^{-8}	5.7×10^{-9}	4.1×10^{-9}	3.5×10^{-9}	3.5×10^{-9}	2.8×10^{-9}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)			
Cs-137	30.0 a	F	1.000	8.8 × 10 ^{−9}	1.000	5.4 × 10 ^{−9}	3.6 × 10 ^{−9}	3.7 × 10 ^{−9}	4.4 × 10 ^{−9}	4.4 × 10 ^{−9}	4.6 × 10 ^{−9}		
		M	0.200	3.6 × 10 ^{−8}	0.100	2.9 × 10 ^{−8}	1.8 × 10 ^{−8}	1.3 × 10 ^{−8}	1.1 × 10 ^{−8}	1.1 × 10 ^{−8}	9.7 × 10 ^{−9}		
		S	0.020	1.1 × 10 ^{−7}	0.010	1.0 × 10 ^{−7}	7.0 × 10 ^{−8}	4.8 × 10 ^{−8}	4.2 × 10 ^{−8}	4.2 × 10 ^{−8}	3.9 × 10 ^{−8}		
Cs-138	0.536 h	F	1.000	2.6 × 10 ^{−10}	1.000	1.8 × 10 ^{−10}	8.1 × 10 ^{−11}	5.0 × 10 ^{−11}	2.9 × 10 ^{−11}	2.9 × 10 ^{−11}	2.4 × 10 ^{−11}		
		M	0.200	4.0 × 10 ^{−10}	0.100	2.7 × 10 ^{−10}	1.3 × 10 ^{−10}	7.8 × 10 ^{−11}	4.9 × 10 ^{−11}	4.9 × 10 ^{−11}	4.1 × 10 ^{−11}		
		S	0.020	4.2 × 10 ^{−10}	0.010	2.8 × 10 ^{−10}	1.3 × 10 ^{−10}	8.2 × 10 ^{−11}	5.1 × 10 ^{−11}	5.1 × 10 ^{−11}	4.3 × 10 ^{−11}		
Barium^f													
Ba-126	1.61 h	F	0.600	6.7 × 10 ^{−10}	0.200	5.2 × 10 ^{−10}	2.4 × 10 ^{−10}	1.4 × 10 ^{−10}	6.9 × 10 ^{−11}	6.9 × 10 ^{−11}	7.4 × 10 ^{−11}		
		M	0.200	1.0 × 10 ^{−9}	0.100	7.0 × 10 ^{−10}	3.2 × 10 ^{−10}	2.0 × 10 ^{−10}	1.2 × 10 ^{−10}	1.2 × 10 ^{−10}	1.0 × 10 ^{−10}		
		S	0.020	1.1 × 10 ^{−9}	0.010	7.2 × 10 ^{−10}	3.3 × 10 ^{−10}	2.1 × 10 ^{−10}	1.3 × 10 ^{−10}	1.3 × 10 ^{−10}	1.1 × 10 ^{−10}		
Ba-128	2.43 d	F	0.600	5.9 × 10 ^{−9}	0.200	5.4 × 10 ^{−9}	2.5 × 10 ^{−9}	1.4 × 10 ^{−9}	7.4 × 10 ^{−10}	7.4 × 10 ^{−10}	7.6 × 10 ^{−10}		
		M	0.200	1.1 × 10 ^{−8}	0.100	7.8 × 10 ^{−9}	3.7 × 10 ^{−9}	2.4 × 10 ^{−9}	1.5 × 10 ^{−9}	1.5 × 10 ^{−9}	1.3 × 10 ^{−9}		
		S	0.020	1.2 × 10 ^{−8}	0.010	8.3 × 10 ^{−9}	4.0 × 10 ^{−9}	2.6 × 10 ^{−9}	1.6 × 10 ^{−9}	1.6 × 10 ^{−9}	1.4 × 10 ^{−9}		
Ba-131	11.8 d	F	0.600	2.1 × 10 ^{−9}	0.200	1.4 × 10 ^{−9}	7.1 × 10 ^{−10}	4.7 × 10 ^{−10}	3.1 × 10 ^{−10}	3.1 × 10 ^{−10}	2.2 × 10 ^{−10}		
		M	0.200	3.7 × 10 ^{−9}	0.100	3.1 × 10 ^{−9}	1.6 × 10 ^{−9}	1.1 × 10 ^{−9}	9.7 × 10 ^{−10}	9.7 × 10 ^{−10}	7.6 × 10 ^{−10}		
		S	0.020	4.0 × 10 ^{−9}	0.010	3.0 × 10 ^{−9}	1.8 × 10 ^{−9}	1.3 × 10 ^{−9}	1.1 × 10 ^{−9}	1.1 × 10 ^{−9}	8.7 × 10 ^{−10}		
Ba-131m	0.243 h	F	0.600	2.7 × 10 ^{−11}	0.200	2.1 × 10 ^{−11}	1.0 × 10 ^{−11}	6.7 × 10 ^{−12}	4.7 × 10 ^{−12}	4.7 × 10 ^{−12}	4.0 × 10 ^{−12}		
		M	0.200	4.8 × 10 ^{−11}	0.100	3.3 × 10 ^{−11}	1.7 × 10 ^{−11}	1.2 × 10 ^{−11}	9.0 × 10 ^{−12}	9.0 × 10 ^{−12}	7.4 × 10 ^{−12}		
		S	0.020	5.0 × 10 ^{−11}	0.010	3.5 × 10 ^{−11}	1.8 × 10 ^{−11}	1.2 × 10 ^{−11}	9.5 × 10 ^{−12}	9.5 × 10 ^{−12}	7.8 × 10 ^{−12}		

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Ba-133	10.7 a	F	0.600	1.1×10^{-8}	0.200	4.5×10^{-9}	2.6×10^{-9}	3.7×10^{-9}	6.0×10^{-9}	6.0×10^{-9}	1.5×10^{-9}			
		M	0.200	1.5×10^{-8}	0.100	1.0×10^{-8}	6.4×10^{-9}	5.1×10^{-9}	5.5×10^{-9}	5.5×10^{-9}	3.1×10^{-9}			
		S	0.020	3.2×10^{-8}	0.010	2.9×10^{-8}	2.0×10^{-8}	1.3×10^{-8}	1.1×10^{-8}	1.1×10^{-8}	1.0×10^{-8}			
Ba-133m	1.62 d	F	0.600	1.4×10^{-9}	0.200	1.1×10^{-9}	4.9×10^{-10}	3.1×10^{-10}	1.5×10^{-10}	1.5×10^{-10}	1.8×10^{-10}			
		M	0.200	3.0×10^{-9}	0.100	2.2×10^{-9}	1.0×10^{-9}	6.9×10^{-10}	5.2×10^{-10}	5.2×10^{-10}	4.2×10^{-10}			
		S	0.020	3.1×10^{-9}	0.010	2.4×10^{-9}	1.1×10^{-9}	7.6×10^{-10}	5.8×10^{-10}	5.8×10^{-10}	4.6×10^{-10}			
Ba-135m	1.20 d	F	0.600	1.1×10^{-9}	0.200	1.0×10^{-9}	4.6×10^{-10}	2.5×10^{-10}	1.2×10^{-10}	1.2×10^{-10}	1.4×10^{-10}			
		M	0.200	2.4×10^{-9}	0.100	1.8×10^{-9}	8.9×10^{-10}	5.4×10^{-10}	4.1×10^{-10}	4.1×10^{-10}	3.3×10^{-10}			
		S	0.020	2.7×10^{-9}	0.010	1.9×10^{-9}	8.6×10^{-10}	5.9×10^{-10}	4.5×10^{-10}	4.5×10^{-10}	3.6×10^{-10}			
Ba-139	1.38 h	F	0.600	3.3×10^{-10}	0.200	2.4×10^{-10}	1.1×10^{-10}	6.0×10^{-11}	3.1×10^{-11}	3.1×10^{-11}	3.4×10^{-11}			
		M	0.200	5.4×10^{-10}	0.100	3.5×10^{-10}	1.6×10^{-10}	1.0×10^{-10}	6.6×10^{-11}	6.6×10^{-11}	5.6×10^{-11}			
		S	0.020	5.7×10^{-10}	0.010	3.6×10^{-10}	1.6×10^{-10}	1.1×10^{-10}	7.0×10^{-11}	7.0×10^{-11}	5.9×10^{-11}			
Ba-140	12.7 d	F	0.600	1.4×10^{-8}	0.200	7.8×10^{-9}	3.6×10^{-9}	2.4×10^{-9}	1.6×10^{-9}	1.6×10^{-9}	1.0×10^{-9}			
		M	0.200	2.7×10^{-8}	0.100	2.0×10^{-8}	1.1×10^{-8}	7.6×10^{-9}	6.2×10^{-9}	6.2×10^{-9}	5.1×10^{-9}			
		S	0.020	2.9×10^{-8}	0.010	2.2×10^{-8}	1.2×10^{-8}	8.6×10^{-9}	7.1×10^{-9}	7.1×10^{-9}	5.8×10^{-9}			
Ba-141	0.305 h	F	0.600	1.9×10^{-10}	0.200	1.4×10^{-10}	6.4×10^{-11}	3.8×10^{-11}	2.1×10^{-11}	2.1×10^{-11}	2.1×10^{-11}			
		M	0.200	3.0×10^{-10}	0.100	2.0×10^{-10}	9.3×10^{-11}	5.9×10^{-11}	3.8×10^{-11}	3.8×10^{-11}	3.2×10^{-11}			
		S	0.020	3.2×10^{-10}	0.010	2.1×10^{-10}	9.7×10^{-11}	6.2×10^{-11}	4.0×10^{-11}	4.0×10^{-11}	3.4×10^{-11}			
Ba-142	0.177 h	F	0.600	1.3×10^{-10}	0.200	9.6×10^{-11}	4.5×10^{-11}	2.7×10^{-11}	1.6×10^{-11}	1.6×10^{-11}	1.5×10^{-11}			
		M	0.200	1.8×10^{-10}	0.100	1.3×10^{-10}	6.1×10^{-11}	3.9×10^{-11}	2.5×10^{-11}	2.5×10^{-11}	2.1×10^{-11}			
		S	0.020	1.9×10^{-10}	0.010	1.3×10^{-10}	6.2×10^{-11}	4.0×10^{-11}	2.6×10^{-11}	2.6×10^{-11}	2.2×10^{-11}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Lanthanum														
La-131	0.983 h	F	0.005	1.2×10^{-10}	5.0×10^{-4}	8.7×10^{-11}	4.2×10^{-11}	2.6×10^{-11}	1.5×10^{-11}	1.3×10^{-11}				
		M	0.005	1.8×10^{-10}	5.0×10^{-4}	1.3×10^{-10}	6.4×10^{-11}	4.1×10^{-11}	2.8×10^{-11}	2.3×10^{-11}				
La-132	4.80 h	F	0.005	1.0×10^{-9}	5.0×10^{-4}	7.7×10^{-10}	3.7×10^{-10}	2.2×10^{-10}	1.2×10^{-10}	1.0×10^{-10}				
		M	0.005	1.5×10^{-9}	5.0×10^{-4}	1.1×10^{-9}	5.4×10^{-10}	3.4×10^{-10}	2.0×10^{-10}	1.6×10^{-10}				
La-135	19.5 h	F	0.005	1.0×10^{-10}	5.0×10^{-4}	7.7×10^{-11}	3.8×10^{-11}	2.3×10^{-11}	1.3×10^{-11}	1.0×10^{-11}				
		M	0.005	1.3×10^{-10}	5.0×10^{-4}	1.0×10^{-10}	4.9×10^{-11}	3.0×10^{-11}	1.7×10^{-11}	1.4×10^{-11}				
La-137	6.00×10^4 a	F	0.005	2.5×10^{-8}	5.0×10^{-4}	2.3×10^{-8}	1.5×10^{-8}	1.1×10^{-8}	8.9×10^{-9}	8.7×10^{-9}				
		M	0.005	8.6×10^{-9}	5.0×10^{-4}	8.1×10^{-9}	5.6×10^{-9}	4.0×10^{-9}	3.6×10^{-9}	3.6×10^{-9}				
La-138	1.35×10^{11} a	F	0.005	3.7×10^{-7}	5.0×10^{-4}	3.5×10^{-7}	2.4×10^{-7}	1.8×10^{-7}	1.6×10^{-7}	1.5×10^{-7}				
		M	0.005	1.3×10^{-7}	5.0×10^{-4}	1.2×10^{-7}	9.1×10^{-8}	6.8×10^{-8}	6.4×10^{-8}	6.4×10^{-8}				
La-140	1.68 d	F	0.005	5.8×10^{-9}	5.0×10^{-4}	4.2×10^{-9}	2.0×10^{-9}	1.2×10^{-9}	6.9×10^{-10}	5.7×10^{-10}				
		M	0.005	8.8×10^{-9}	5.0×10^{-4}	6.3×10^{-9}	3.1×10^{-9}	2.0×10^{-9}	1.3×10^{-9}	1.1×10^{-9}				
La-141	3.93 h	F	0.005	8.6×10^{-10}	5.0×10^{-4}	5.5×10^{-10}	2.3×10^{-10}	1.4×10^{-10}	7.5×10^{-11}	6.3×10^{-11}				
		M	0.005	1.4×10^{-9}	5.0×10^{-4}	9.3×10^{-10}	4.3×10^{-10}	2.8×10^{-10}	1.8×10^{-10}	1.5×10^{-10}				
La-142	1.54 h	F	0.005	5.3×10^{-10}	5.0×10^{-4}	3.8×10^{-10}	1.8×10^{-10}	1.1×10^{-10}	6.3×10^{-11}	5.2×10^{-11}				
		M	0.005	8.1×10^{-10}	5.0×10^{-4}	5.7×10^{-10}	2.7×10^{-10}	1.7×10^{-10}	1.1×10^{-10}	8.9×10^{-11}				
La-143	0.237 h	F	0.005	1.4×10^{-10}	5.0×10^{-4}	8.6×10^{-11}	3.7×10^{-11}	2.3×10^{-11}	1.4×10^{-11}	1.2×10^{-11}				
		M	0.005	2.1×10^{-10}	5.0×10^{-4}	1.3×10^{-10}	6.0×10^{-11}	3.9×10^{-11}	2.5×10^{-11}	2.1×10^{-11}				

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Cerium														
Ce-134	3.00 d	F	0.005	7.6×10^{-9}	5.0×10^{-4}	5.3×10^{-9}	2.3×10^{-9}	1.4×10^{-9}	1.4×10^{-9}	7.7×10^{-10}	5.7×10^{-10}			
		M	0.005	1.1×10^{-8}	5.0×10^{-4}	7.6×10^{-9}	3.7×10^{-9}	2.4×10^{-9}	2.4×10^{-9}	1.5×10^{-9}	1.3×10^{-9}			
		S	0.005	1.2×10^{-8}	5.0×10^{-4}	8.0×10^{-9}	3.8×10^{-9}	2.5×10^{-9}	2.5×10^{-9}	1.6×10^{-9}	1.3×10^{-9}			
Ce-135	17.6 h	F	0.005	2.3×10^{-9}	5.0×10^{-4}	1.7×10^{-9}	8.5×10^{-10}	5.3×10^{-10}	5.3×10^{-10}	3.0×10^{-10}	2.4×10^{-10}			
		M	0.005	3.6×10^{-9}	5.0×10^{-4}	2.7×10^{-9}	1.4×10^{-9}	8.9×10^{-10}	8.9×10^{-10}	5.9×10^{-10}	4.8×10^{-10}			
		S	0.005	3.7×10^{-9}	5.0×10^{-4}	2.8×10^{-9}	1.4×10^{-9}	9.4×10^{-10}	9.4×10^{-10}	6.3×10^{-10}	5.0×10^{-10}			
Ce-137	9.00 h	F	0.005	7.5×10^{-11}	5.0×10^{-4}	5.6×10^{-11}	2.7×10^{-11}	1.6×10^{-11}	1.6×10^{-11}	8.7×10^{-12}	7.0×10^{-12}			
		M	0.005	1.1×10^{-10}	5.0×10^{-4}	7.6×10^{-11}	3.6×10^{-11}	2.2×10^{-11}	2.2×10^{-11}	1.2×10^{-11}	9.8×10^{-12}			
		S	0.005	1.1×10^{-10}	5.0×10^{-4}	7.8×10^{-11}	3.7×10^{-11}	2.3×10^{-11}	2.3×10^{-11}	1.3×10^{-11}	1.0×10^{-11}			
Ce-137m	1.43 d	F	0.005	1.6×10^{-9}	5.0×10^{-4}	1.1×10^{-9}	4.6×10^{-10}	2.8×10^{-10}	2.8×10^{-10}	1.5×10^{-10}	1.2×10^{-10}			
		M	0.005	3.1×10^{-9}	5.0×10^{-4}	2.2×10^{-9}	1.1×10^{-9}	6.7×10^{-10}	6.7×10^{-10}	5.1×10^{-10}	4.1×10^{-10}			
		S	0.005	3.3×10^{-9}	5.0×10^{-4}	2.3×10^{-9}	1.0×10^{-9}	7.3×10^{-10}	7.3×10^{-10}	5.6×10^{-10}	4.4×10^{-10}			
Ce-139	138 d	F	0.005	1.1×10^{-8}	5.0×10^{-4}	8.5×10^{-9}	4.5×10^{-9}	2.8×10^{-9}	2.8×10^{-9}	1.8×10^{-9}	1.5×10^{-9}			
		M	0.005	7.5×10^{-9}	5.0×10^{-4}	6.1×10^{-9}	3.6×10^{-9}	2.5×10^{-9}	2.5×10^{-9}	2.1×10^{-9}	1.7×10^{-9}			
		S	0.005	7.8×10^{-9}	5.0×10^{-4}	6.3×10^{-9}	3.9×10^{-9}	2.7×10^{-9}	2.7×10^{-9}	2.4×10^{-9}	1.9×10^{-9}			
Ce-141	32.5 d	F	0.005	1.1×10^{-8}	5.0×10^{-4}	7.3×10^{-9}	3.5×10^{-9}	2.0×10^{-9}	2.0×10^{-9}	1.2×10^{-9}	9.3×10^{-10}			
		M	0.005	1.4×10^{-8}	5.0×10^{-4}	1.1×10^{-8}	6.3×10^{-9}	4.6×10^{-9}	4.6×10^{-9}	4.1×10^{-9}	3.2×10^{-9}			
		S	0.005	1.6×10^{-8}	5.0×10^{-4}	1.2×10^{-8}	7.1×10^{-9}	5.3×10^{-9}	5.3×10^{-9}	4.8×10^{-9}	3.8×10^{-9}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a	7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)		
Ce-143	1.38 d	F	0.005	3.6 × 10 ^{−9}	5.0 × 10 ^{−4}	2.3 × 10 ^{−9}	1.0 × 10 ^{−9}	6.2 × 10 ^{−10}	3.3 × 10 ^{−10}	2.7 × 10 ^{−10}		
		M	0.005	5.6 × 10 ^{−9}	5.0 × 10 ^{−4}	3.9 × 10 ^{−9}	1.9 × 10 ^{−9}	1.3 × 10 ^{−9}	9.3 × 10 ^{−10}	7.5 × 10 ^{−10}		
		S	0.005	5.9 × 10 ^{−9}	5.0 × 10 ^{−4}	4.1 × 10 ^{−9}	2.1 × 10 ^{−9}	1.4 × 10 ^{−9}	1.0 × 10 ^{−9}	8.3 × 10 ^{−10}		
Ce-144	284 d	F	0.005	3.6 × 10 ^{−7}	5.0 × 10 ^{−4}	2.7 × 10 ^{−7}	1.4 × 10 ^{−7}	7.8 × 10 ^{−8}	4.8 × 10 ^{−8}	4.0 × 10 ^{−8}		
		M	0.005	1.9 × 10 ^{−7}	5.0 × 10 ^{−4}	1.6 × 10 ^{−7}	8.8 × 10 ^{−8}	5.5 × 10 ^{−8}	4.1 × 10 ^{−8}	3.6 × 10 ^{−8}		
		S	0.005	2.1 × 10 ^{−7}	5.0 × 10 ^{−4}	1.8 × 10 ^{−7}	1.1 × 10 ^{−7}	7.3 × 10 ^{−8}	5.8 × 10 ^{−8}	5.3 × 10 ^{−8}		
Praseodymium												
Pr-136	0.218 h	M	0.005	1.3 × 10 ^{−10}	5.0 × 10 ^{−4}	8.8 × 10 ^{−11}	4.2 × 10 ^{−11}	2.6 × 10 ^{−11}	1.6 × 10 ^{−11}	1.3 × 10 ^{−11}		
		S	0.005	1.3 × 10 ^{−10}	5.0 × 10 ^{−4}	9.0 × 10 ^{−11}	4.3 × 10 ^{−11}	2.7 × 10 ^{−11}	1.7 × 10 ^{−11}	1.4 × 10 ^{−11}		
Pr-137	1.28 h	M	0.005	1.8 × 10 ^{−10}	5.0 × 10 ^{−4}	1.3 × 10 ^{−10}	6.1 × 10 ^{−11}	3.9 × 10 ^{−11}	2.4 × 10 ^{−11}	2.0 × 10 ^{−11}		
		S	0.005	1.9 × 10 ^{−10}	5.0 × 10 ^{−4}	1.3 × 10 ^{−10}	6.4 × 10 ^{−11}	4.0 × 10 ^{−11}	2.5 × 10 ^{−11}	2.1 × 10 ^{−11}		
Pr-138m	2.10 h	M	0.005	5.9 × 10 ^{−10}	5.0 × 10 ^{−4}	4.5 × 10 ^{−10}	2.3 × 10 ^{−10}	1.4 × 10 ^{−10}	9.0 × 10 ^{−11}	7.2 × 10 ^{−11}		
		S	0.005	6.0 × 10 ^{−10}	5.0 × 10 ^{−4}	4.7 × 10 ^{−10}	2.4 × 10 ^{−10}	1.5 × 10 ^{−10}	9.3 × 10 ^{−11}	7.4 × 10 ^{−11}		
Pr-139	4.51 h	M	0.005	1.5 × 10 ^{−10}	5.0 × 10 ^{−4}	1.1 × 10 ^{−10}	5.5 × 10 ^{−11}	3.5 × 10 ^{−11}	2.3 × 10 ^{−11}	1.8 × 10 ^{−11}		
		S	0.005	1.6 × 10 ^{−10}	5.0 × 10 ^{−4}	1.2 × 10 ^{−10}	5.7 × 10 ^{−11}	3.7 × 10 ^{−11}	2.4 × 10 ^{−11}	2.0 × 10 ^{−11}		
Pr-142	19.1 h	M	0.005	5.3 × 10 ^{−9}	5.0 × 10 ^{−4}	3.5 × 10 ^{−9}	1.6 × 10 ^{−9}	1.0 × 10 ^{−9}	6.2 × 10 ^{−10}	5.2 × 10 ^{−10}		
		S	0.005	5.5 × 10 ^{−9}	5.0 × 10 ^{−4}	3.7 × 10 ^{−9}	1.7 × 10 ^{−9}	1.1 × 10 ^{−9}	6.6 × 10 ^{−10}	5.5 × 10 ^{−10}		
Pr-142m	0.243 h	M	0.005	6.7 × 10 ^{−11}	5.0 × 10 ^{−4}	4.5 × 10 ^{−11}	2.0 × 10 ^{−11}	1.3 × 10 ^{−11}	7.9 × 10 ^{−12}	6.6 × 10 ^{−12}		
		S	0.005	7.0 × 10 ^{−11}	5.0 × 10 ^{−4}	4.7 × 10 ^{−11}	2.2 × 10 ^{−11}	1.4 × 10 ^{−11}	8.4 × 10 ^{−12}	7.0 × 10 ^{−12}		
Pr-143	13.6 d	M	0.005	1.2 × 10 ^{−8}	5.0 × 10 ^{−4}	8.4 × 10 ^{−9}	4.6 × 10 ^{−9}	3.2 × 10 ^{−9}	2.7 × 10 ^{−9}	2.2 × 10 ^{−9}		
		S	0.005	1.3 × 10 ^{−8}	5.0 × 10 ^{−4}	9.2 × 10 ^{−9}	5.1 × 10 ^{−9}	3.6 × 10 ^{−9}	3.0 × 10 ^{−9}	2.4 × 10 ^{−9}		

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a							2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$
Pr-144	0.288 h	M	0.005	1.9×10^{-10}	5.0×10^{-4}	1.2×10^{-10}	5.0×10^{-11}	3.2×10^{-11}	2.1×10^{-11}	2.1×10^{-11}	1.8×10^{-11}	1.8×10^{-11}							
		S	0.005	1.9×10^{-10}	5.0×10^{-4}	1.2×10^{-10}	5.2×10^{-11}	3.4×10^{-11}	2.1×10^{-11}	2.1×10^{-11}	1.8×10^{-11}	1.8×10^{-11}							
Pr-145	5.98 h	M	0.005	1.6×10^{-9}	5.0×10^{-4}	1.0×10^{-9}	4.7×10^{-10}	3.0×10^{-10}	1.9×10^{-10}	1.9×10^{-10}	1.6×10^{-10}	1.6×10^{-10}							
		S	0.005	1.6×10^{-9}	5.0×10^{-4}	1.1×10^{-9}	4.9×10^{-10}	3.2×10^{-10}	2.0×10^{-10}	2.0×10^{-10}	1.7×10^{-10}	1.7×10^{-10}							
Pr-147	0.227 h	M	0.005	1.5×10^{-10}	5.0×10^{-4}	1.0×10^{-10}	4.8×10^{-11}	3.1×10^{-11}	2.1×10^{-11}	2.1×10^{-11}	1.8×10^{-11}	1.8×10^{-11}							
		S	0.005	1.6×10^{-10}	5.0×10^{-4}	1.1×10^{-10}	5.0×10^{-11}	3.3×10^{-11}	2.2×10^{-11}	2.2×10^{-11}	1.8×10^{-11}	1.8×10^{-11}							
Neodymium																			
Nd-136	0.844 h	M	0.005	4.6×10^{-10}	5.0×10^{-4}	3.2×10^{-10}	1.6×10^{-10}	9.8×10^{-11}	6.3×10^{-11}	6.3×10^{-11}	5.1×10^{-11}	5.1×10^{-11}							
		S	0.005	4.8×10^{-10}	5.0×10^{-4}	3.3×10^{-10}	1.6×10^{-10}	1.0×10^{-10}	6.6×10^{-11}	6.6×10^{-11}	5.4×10^{-11}	5.4×10^{-11}							
Nd-138	5.04 h	M	0.005	2.3×10^{-9}	5.0×10^{-4}	1.7×10^{-9}	7.7×10^{-10}	4.8×10^{-10}	2.8×10^{-10}	2.8×10^{-10}	2.3×10^{-10}	2.3×10^{-10}							
		S	0.005	2.4×10^{-9}	5.0×10^{-4}	1.8×10^{-9}	8.0×10^{-10}	5.0×10^{-10}	3.0×10^{-10}	3.0×10^{-10}	2.5×10^{-10}	2.5×10^{-10}							
Nd-139	0.495 h	M	0.005	9.0×10^{-11}	5.0×10^{-4}	6.2×10^{-11}	3.0×10^{-11}	1.9×10^{-11}	1.2×10^{-11}	1.2×10^{-11}	9.9×10^{-12}	9.9×10^{-12}							
		S	0.005	9.4×10^{-11}	5.0×10^{-4}	6.4×10^{-11}	3.1×10^{-11}	2.0×10^{-11}	1.3×10^{-11}	1.3×10^{-11}	1.0×10^{-11}	1.0×10^{-11}							
Nd-139m	5.50 h	M	0.005	1.1×10^{-9}	5.0×10^{-4}	8.8×10^{-10}	4.5×10^{-10}	2.9×10^{-10}	1.8×10^{-10}	1.8×10^{-10}	1.5×10^{-10}	1.5×10^{-10}							
		S	0.005	1.2×10^{-9}	5.0×10^{-4}	9.1×10^{-10}	4.6×10^{-10}	3.0×10^{-10}	1.9×10^{-10}	1.9×10^{-10}	1.5×10^{-10}	1.5×10^{-10}							
Nd-141	2.49 h	M	0.005	4.1×10^{-11}	5.0×10^{-4}	3.1×10^{-11}	1.5×10^{-11}	9.6×10^{-12}	6.0×10^{-12}	6.0×10^{-12}	4.8×10^{-12}	4.8×10^{-12}							
		S	0.005	4.3×10^{-11}	5.0×10^{-4}	3.2×10^{-11}	1.6×10^{-11}	1.0×10^{-11}	6.2×10^{-12}	6.2×10^{-12}	5.0×10^{-12}	5.0×10^{-12}							
Nd-147	11.0 d	M	0.005	1.1×10^{-8}	5.0×10^{-4}	8.0×10^{-9}	4.5×10^{-9}	3.2×10^{-9}	2.6×10^{-9}	2.6×10^{-9}	2.1×10^{-9}	2.1×10^{-9}							
		S	0.005	1.2×10^{-8}	5.0×10^{-4}	8.6×10^{-9}	4.9×10^{-9}	3.5×10^{-9}	3.0×10^{-9}	3.0×10^{-9}	2.4×10^{-9}	2.4×10^{-9}							
Nd-149	1.73 h	M	0.005	6.8×10^{-10}	5.0×10^{-4}	4.6×10^{-10}	2.2×10^{-10}	1.5×10^{-10}	1.0×10^{-10}	1.0×10^{-10}	8.4×10^{-11}	8.4×10^{-11}							
		S	0.005	7.1×10^{-10}	5.0×10^{-4}	4.8×10^{-10}	2.3×10^{-10}	1.5×10^{-10}	1.1×10^{-10}	1.1×10^{-10}	8.9×10^{-11}	8.9×10^{-11}							

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		> 17 a
		Type	f_1		$e(g)$	Type	$e(g)$	Type	$e(g)$	Type	$e(g)$		
Nd-151	0.207 h	M	0.005	1.5×10^{-10}	M	5.0×10^{-4}	9.9×10^{-11}	4.6×10^{-11}	3.0×10^{-11}	2.0×10^{-11}	1.7×10^{-11}	1.7×10^{-11}	
		S	0.005	1.5×10^{-10}	S	5.0×10^{-4}	1.0×10^{-10}	4.8×10^{-11}	3.1×10^{-11}	2.1×10^{-11}	1.7×10^{-11}		
Promethium													
Pm-141	0.348 h	M	0.005	1.4×10^{-10}	M	5.0×10^{-4}	9.4×10^{-11}	4.3×10^{-11}	2.7×10^{-11}	1.7×10^{-11}	1.4×10^{-11}	1.4×10^{-11}	
		S	0.005	1.5×10^{-10}	S	5.0×10^{-4}	9.7×10^{-11}	4.4×10^{-11}	2.8×10^{-11}	1.8×10^{-11}	1.5×10^{-11}		
Pm-143	265 d	M	0.005	6.2×10^{-9}	M	5.0×10^{-4}	5.4×10^{-9}	3.3×10^{-9}	2.2×10^{-9}	1.7×10^{-9}	1.5×10^{-9}	1.5×10^{-9}	
		S	0.005	5.5×10^{-9}	S	5.0×10^{-4}	4.8×10^{-9}	3.1×10^{-9}	2.1×10^{-9}	1.7×10^{-9}	1.4×10^{-9}		
Pm-144	363 d	M	0.005	3.1×10^{-8}	M	5.0×10^{-4}	2.8×10^{-8}	1.8×10^{-8}	1.2×10^{-8}	9.3×10^{-9}	8.2×10^{-9}	8.2×10^{-9}	
		S	0.005	2.6×10^{-8}	S	5.0×10^{-4}	2.4×10^{-8}	1.6×10^{-8}	1.1×10^{-8}	8.9×10^{-9}	7.5×10^{-9}		
Pm-145	17.7 a	M	0.005	1.1×10^{-8}	M	5.0×10^{-4}	9.8×10^{-9}	6.4×10^{-9}	4.3×10^{-9}	3.7×10^{-9}	3.6×10^{-9}	3.6×10^{-9}	
		S	0.005	7.1×10^{-9}	S	5.0×10^{-4}	6.5×10^{-9}	4.3×10^{-9}	2.9×10^{-9}	2.4×10^{-9}	2.3×10^{-9}		
Pm-146	5.53 a	M	0.005	6.4×10^{-8}	M	5.0×10^{-4}	5.9×10^{-8}	3.9×10^{-8}	2.6×10^{-8}	2.2×10^{-8}	2.1×10^{-8}	2.1×10^{-8}	
		S	0.005	5.3×10^{-8}	S	5.0×10^{-4}	4.9×10^{-8}	3.3×10^{-8}	2.2×10^{-8}	1.9×10^{-8}	1.7×10^{-8}		
Pm-147	2.62 a	M	0.005	2.1×10^{-8}	M	5.0×10^{-4}	1.8×10^{-8}	1.1×10^{-8}	7.0×10^{-9}	5.7×10^{-9}	5.0×10^{-9}	5.0×10^{-9}	
		S	0.005	1.9×10^{-8}	S	5.0×10^{-4}	1.6×10^{-8}	1.0×10^{-8}	6.8×10^{-9}	5.8×10^{-9}	4.9×10^{-9}		
Pm-148	5.37 d	M	0.005	1.5×10^{-8}	M	5.0×10^{-4}	1.0×10^{-8}	5.2×10^{-9}	3.4×10^{-9}	2.4×10^{-9}	2.0×10^{-9}	2.0×10^{-9}	
		S	0.005	1.5×10^{-8}	S	5.0×10^{-4}	1.1×10^{-8}	5.5×10^{-9}	3.7×10^{-9}	2.6×10^{-9}	2.2×10^{-9}		
Pm-148m	41.3 d	M	0.005	2.4×10^{-8}	M	5.0×10^{-4}	1.9×10^{-8}	1.1×10^{-8}	7.7×10^{-9}	6.3×10^{-9}	5.1×10^{-9}	5.1×10^{-9}	
		S	0.005	2.5×10^{-8}	S	5.0×10^{-4}	2.0×10^{-8}	1.2×10^{-8}	8.3×10^{-9}	7.1×10^{-9}	5.7×10^{-9}		
Pm-149	2.21 d	M	0.005	5.0×10^{-9}	M	5.0×10^{-4}	3.5×10^{-9}	1.7×10^{-9}	1.1×10^{-9}	8.3×10^{-10}	6.7×10^{-10}	6.7×10^{-10}	
		S	0.005	5.3×10^{-9}	S	5.0×10^{-4}	3.6×10^{-9}	1.8×10^{-9}	1.2×10^{-9}	9.0×10^{-10}	7.3×10^{-10}		

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Pm-150	2.68 h	M	0.005	1.2×10^{-9}	5.0×10^{-4}	7.9×10^{-10}	3.8×10^{-10}	2.4×10^{-10}	1.5×10^{-10}	1.2×10^{-10}							
		S	0.005	1.2×10^{-9}	5.0×10^{-4}	8.2×10^{-10}	3.9×10^{-10}	2.5×10^{-10}	1.6×10^{-10}	1.3×10^{-10}							
	1.18 d	M	0.005	3.3×10^{-9}	5.0×10^{-4}	2.5×10^{-9}	1.2×10^{-9}	8.3×10^{-10}	5.3×10^{-10}	4.3×10^{-10}							
		S	0.005	3.4×10^{-9}	5.0×10^{-4}	2.6×10^{-9}	1.3×10^{-9}	7.9×10^{-10}	5.7×10^{-10}	4.6×10^{-10}							
Samarium																	
Sm-141	0.170 h	M	0.005	1.5×10^{-10}	5.0×10^{-4}	1.0×10^{-10}	4.7×10^{-11}	2.9×10^{-11}	1.8×10^{-11}	1.5×10^{-11}							
Sm-141m	0.377 h	M	0.005	3.0×10^{-10}	5.0×10^{-4}	2.1×10^{-10}	9.7×10^{-11}	6.1×10^{-11}	3.9×10^{-11}	3.2×10^{-11}							
Sm-142	1.21 h	M	0.005	7.5×10^{-10}	5.0×10^{-4}	4.8×10^{-10}	2.2×10^{-10}	1.4×10^{-10}	8.5×10^{-11}	7.1×10^{-11}							
Sm-145	340 d	M	0.005	8.1×10^{-9}	5.0×10^{-4}	6.8×10^{-9}	4.0×10^{-9}	2.5×10^{-9}	1.9×10^{-9}	1.6×10^{-9}							
Sm-146	1.03×10^8 a	M	0.005	2.7×10^{-5}	5.0×10^{-4}	2.6×10^{-5}	1.7×10^{-5}	1.2×10^{-5}	1.1×10^{-5}	1.1×10^{-5}							
Sm-147	1.06×10^{11} a	M	0.005	2.5×10^{-5}	5.0×10^{-4}	2.3×10^{-5}	1.6×10^{-5}	1.1×10^{-5}	9.6×10^{-6}	9.6×10^{-6}							
Sm-151	90.0 a	M	0.005	1.1×10^{-8}	5.0×10^{-4}	1.0×10^{-8}	6.7×10^{-9}	4.5×10^{-9}	4.0×10^{-9}	4.0×10^{-9}							
Sm-153	1.95 d	M	0.005	4.2×10^{-9}	5.0×10^{-4}	2.9×10^{-9}	1.5×10^{-9}	1.0×10^{-9}	7.9×10^{-10}	6.3×10^{-10}							
Sm-155	0.368 h	M	0.005	1.5×10^{-10}	5.0×10^{-4}	9.9×10^{-11}	4.4×10^{-11}	2.9×10^{-11}	2.0×10^{-11}	1.7×10^{-11}							
Sm-156	9.40 h	M	0.005	1.6×10^{-9}	5.0×10^{-4}	1.1×10^{-9}	5.8×10^{-10}	3.5×10^{-10}	2.7×10^{-10}	2.2×10^{-10}							
Europium																	
Eu-145	5.94 d	M	0.005	3.6×10^{-9}	5.0×10^{-4}	2.9×10^{-9}	1.6×10^{-9}	1.0×10^{-9}	6.8×10^{-10}	5.5×10^{-10}							
Eu-146	4.61 d	M	0.005	5.5×10^{-9}	5.0×10^{-4}	4.4×10^{-9}	2.4×10^{-9}	1.5×10^{-9}	1.0×10^{-9}	8.0×10^{-10}							
Eu-147	24.0 d	M	0.005	4.9×10^{-9}	5.0×10^{-4}	3.7×10^{-9}	2.2×10^{-9}	1.6×10^{-9}	1.3×10^{-9}	1.1×10^{-9}							
Eu-148	54.5 d	M	0.005	1.4×10^{-8}	5.0×10^{-4}	1.2×10^{-8}	6.8×10^{-9}	4.6×10^{-9}	3.2×10^{-9}	2.6×10^{-9}							

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Eu-149	93.1 d	M	0.005	1.6×10^{-9}	5.0×10^{-4}	1.3×10^{-9}	7.3×10^{-10}	4.7×10^{-10}	3.5×10^{-10}	2.9×10^{-10}			
Eu-150	34.2 a	M	0.005	1.1×10^{-7}	5.0×10^{-4}	1.1×10^{-7}	7.8×10^{-8}	5.7×10^{-8}	5.3×10^{-8}	5.3×10^{-8}			
Eu-150m	12.6 h	M	0.005	1.6×10^{-9}	5.0×10^{-4}	1.1×10^{-9}	5.2×10^{-10}	3.4×10^{-10}	2.3×10^{-10}	1.9×10^{-10}			
Eu-152	13.3 a	M	0.005	1.1×10^{-7}	5.0×10^{-4}	1.0×10^{-7}	7.0×10^{-8}	4.9×10^{-8}	4.3×10^{-8}	4.2×10^{-8}			
Eu-152m	9.32 h	M	0.005	1.9×10^{-9}	5.0×10^{-4}	1.3×10^{-9}	6.6×10^{-10}	4.2×10^{-10}	2.4×10^{-10}	2.2×10^{-10}			
Eu-154	8.80 a	M	0.005	1.6×10^{-7}	5.0×10^{-4}	1.5×10^{-7}	9.7×10^{-8}	6.5×10^{-8}	5.6×10^{-8}	5.3×10^{-8}			
Eu-155	4.96 a	M	0.005	2.6×10^{-8}	5.0×10^{-4}	2.3×10^{-8}	1.4×10^{-8}	9.2×10^{-9}	7.6×10^{-9}	6.9×10^{-9}			
Eu-156	15.2 d	M	0.005	1.9×10^{-8}	5.0×10^{-4}	1.4×10^{-8}	7.7×10^{-9}	5.3×10^{-9}	4.2×10^{-9}	3.4×10^{-9}			
Eu-157	15.1 h	M	0.005	2.5×10^{-9}	5.0×10^{-4}	1.9×10^{-9}	8.9×10^{-10}	5.9×10^{-10}	3.5×10^{-10}	2.8×10^{-10}			
Eu-158	0.765 h	M	0.005	4.3×10^{-10}	5.0×10^{-4}	2.9×10^{-10}	1.3×10^{-10}	8.5×10^{-11}	5.6×10^{-11}	4.7×10^{-11}			
Gadolinium													
Gd-145	0.382 h	F	0.005	1.3×10^{-10}	5.0×10^{-4}	9.6×10^{-11}	4.7×10^{-11}	2.9×10^{-11}	1.7×10^{-11}	1.4×10^{-11}			
		M	0.005	1.8×10^{-10}	5.0×10^{-4}	1.3×10^{-10}	6.2×10^{-11}	3.9×10^{-11}	2.4×10^{-11}	2.0×10^{-11}			
Gd-146	48.3 d	F	0.005	2.9×10^{-8}	5.0×10^{-4}	2.3×10^{-8}	1.2×10^{-8}	7.8×10^{-9}	5.1×10^{-9}	4.4×10^{-9}			
		M	0.005	2.8×10^{-8}	5.0×10^{-4}	2.2×10^{-8}	1.3×10^{-8}	9.3×10^{-9}	7.9×10^{-9}	6.4×10^{-9}			
Gd-147	1.59 d	F	0.005	2.1×10^{-9}	5.0×10^{-4}	1.7×10^{-9}	8.4×10^{-10}	5.3×10^{-10}	3.1×10^{-10}	2.6×10^{-10}			
		M	0.005	2.8×10^{-9}	5.0×10^{-4}	2.2×10^{-9}	1.1×10^{-9}	7.5×10^{-10}	5.1×10^{-10}	4.0×10^{-10}			
Gd-148	93.0 a	F	0.005	8.3×10^{-5}	5.0×10^{-4}	7.6×10^{-5}	4.7×10^{-5}	3.2×10^{-5}	2.6×10^{-5}	2.6×10^{-5}			
		M	0.005	3.2×10^{-5}	5.0×10^{-4}	2.9×10^{-5}	1.9×10^{-5}	1.3×10^{-5}	1.2×10^{-5}	1.1×10^{-5}			
Gd-149	9.40 d	F	0.005	2.6×10^{-9}	5.0×10^{-4}	2.0×10^{-9}	8.0×10^{-10}	5.1×10^{-10}	3.1×10^{-10}	2.6×10^{-10}			
		M	0.005	3.6×10^{-9}	5.0×10^{-4}	3.0×10^{-9}	1.5×10^{-9}	1.1×10^{-9}	9.2×10^{-10}	7.3×10^{-10}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Gd-151	120 d	F	0.005	6.3×10^{-9}	5.0×10^{-4}	4.9×10^{-9}	2.5×10^{-9}	1.5×10^{-9}	9.2×10^{-10}	7.8×10^{-10}			
		M	0.005	4.5×10^{-9}	5.0×10^{-4}	3.5×10^{-9}	2.0×10^{-9}	1.3×10^{-9}	1.0×10^{-9}	8.6×10^{-10}			
Gd-152	1.08×10^{14} a	F	0.005	5.9×10^{-5}	5.0×10^{-4}	5.4×10^{-5}	3.4×10^{-5}	2.4×10^{-5}	1.9×10^{-5}	1.9×10^{-5}			
		M	0.005	2.1×10^{-5}	5.0×10^{-4}	1.9×10^{-5}	1.3×10^{-5}	8.9×10^{-6}	7.9×10^{-6}	8.0×10^{-6}			
Gd-153	242 d	F	0.005	1.5×10^{-8}	5.0×10^{-4}	1.2×10^{-8}	6.5×10^{-9}	3.9×10^{-9}	2.4×10^{-9}	2.1×10^{-9}			
		M	0.005	9.9×10^{-9}	5.0×10^{-4}	7.9×10^{-9}	4.8×10^{-9}	3.1×10^{-9}	2.5×10^{-9}	2.1×10^{-9}			
Gd-159	18.6 h	F	0.005	1.2×10^{-9}	5.0×10^{-4}	8.9×10^{-10}	3.8×10^{-10}	2.3×10^{-10}	1.2×10^{-10}	1.0×10^{-10}			
		M	0.005	2.2×10^{-9}	5.0×10^{-4}	1.5×10^{-9}	7.3×10^{-10}	4.9×10^{-10}	3.4×10^{-10}	2.7×10^{-10}			
Terbium													
Tb-147	1.65 h	M	0.005	6.7×10^{-10}	5.0×10^{-4}	4.8×10^{-10}	2.3×10^{-10}	1.5×10^{-10}	9.3×10^{-11}	7.6×10^{-11}			
Tb-149	4.15 h	M	0.005	2.1×10^{-8}	5.0×10^{-4}	1.5×10^{-8}	9.6×10^{-9}	6.6×10^{-9}	5.8×10^{-9}	4.9×10^{-9}			
Tb-150	3.27 h	M	0.005	1.0×10^{-9}	5.0×10^{-4}	7.4×10^{-10}	3.5×10^{-10}	2.2×10^{-10}	1.3×10^{-10}	1.1×10^{-10}			
Tb-151	17.6 h	M	0.005	1.6×10^{-9}	5.0×10^{-4}	1.2×10^{-9}	6.3×10^{-10}	4.2×10^{-10}	2.8×10^{-10}	2.3×10^{-10}			
Tb-153	2.34 d	M	0.005	1.4×10^{-9}	5.0×10^{-4}	1.0×10^{-9}	5.4×10^{-10}	3.6×10^{-10}	2.3×10^{-10}	1.9×10^{-10}			
Tb-154	21.4 h	M	0.005	2.7×10^{-9}	5.0×10^{-4}	2.1×10^{-9}	1.1×10^{-9}	7.1×10^{-10}	4.5×10^{-10}	3.6×10^{-10}			
Tb-155	5.32 d	M	0.005	1.4×10^{-9}	5.0×10^{-4}	1.0×10^{-9}	5.6×10^{-10}	3.4×10^{-10}	2.7×10^{-10}	2.2×10^{-10}			
Tb-156	5.34 d	M	0.005	7.0×10^{-9}	5.0×10^{-4}	5.4×10^{-9}	3.0×10^{-9}	2.0×10^{-9}	1.5×10^{-9}	1.2×10^{-9}			
Tb-156m	1.02 d	M	0.005	1.1×10^{-9}	5.0×10^{-4}	9.4×10^{-10}	4.7×10^{-10}	3.3×10^{-10}	2.7×10^{-10}	2.1×10^{-10}			
Tb-156m'	5.00 h	M	0.005	6.2×10^{-10}	5.0×10^{-4}	4.5×10^{-10}	2.4×10^{-10}	1.7×10^{-10}	1.2×10^{-10}	9.6×10^{-11}			
Tb-157	1.50×10^2 a	M	0.005	3.2×10^{-9}	5.0×10^{-4}	3.0×10^{-9}	2.0×10^{-9}	1.4×10^{-9}	1.2×10^{-9}	1.2×10^{-9}			
Tb-158	1.50×10^2 a	M	0.005	1.1×10^{-7}	5.0×10^{-4}	1.0×10^{-7}	7.0×10^{-8}	5.1×10^{-8}	4.7×10^{-8}	4.6×10^{-8}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Tb-160	72.3 d	M	0.005	3.2×10^{-8}	5.0×10^{-4}	2.5×10^{-8}	1.5×10^{-8}	1.0×10^{-8}	1.0×10^{-8}	8.6×10^{-9}	7.0×10^{-9}		
Tb-161	6.91 d	M	0.005	6.6×10^{-9}	5.0×10^{-4}	4.7×10^{-9}	2.6×10^{-9}	1.9×10^{-9}	1.9×10^{-9}	1.6×10^{-9}	1.3×10^{-9}		
Dysprosium													
Dy-155	10.0 h	M	0.005	5.6×10^{-10}	5.0×10^{-4}	4.4×10^{-10}	2.3×10^{-10}	1.5×10^{-10}	1.5×10^{-10}	9.6×10^{-11}	7.7×10^{-11}		
Dy-157	8.10 h	M	0.005	2.4×10^{-10}	5.0×10^{-4}	1.9×10^{-10}	9.9×10^{-11}	6.2×10^{-11}	6.2×10^{-11}	3.8×10^{-11}	3.0×10^{-11}		
Dy-159	144 d	M	0.005	2.1×10^{-9}	5.0×10^{-4}	1.7×10^{-9}	9.6×10^{-10}	6.0×10^{-10}	6.0×10^{-10}	4.4×10^{-10}	3.7×10^{-10}		
Dy-165	2.33 h	M	0.005	5.2×10^{-10}	5.0×10^{-4}	3.4×10^{-10}	1.6×10^{-10}	1.1×10^{-10}	1.1×10^{-10}	7.2×10^{-11}	6.0×10^{-11}		
Dy-166	3.40 d	M	0.005	1.2×10^{-8}	5.0×10^{-4}	8.3×10^{-9}	4.4×10^{-9}	3.0×10^{-9}	3.0×10^{-9}	2.3×10^{-9}	1.9×10^{-9}		
Holmium													
Ho-155	0.800 h	M	0.005	1.7×10^{-10}	5.0×10^{-4}	1.2×10^{-10}	5.8×10^{-11}	3.7×10^{-11}	3.7×10^{-11}	2.4×10^{-11}	2.0×10^{-11}		
Ho-157	0.210 h	M	0.005	3.4×10^{-11}	5.0×10^{-4}	2.5×10^{-11}	1.3×10^{-11}	8.0×10^{-12}	8.0×10^{-12}	5.1×10^{-12}	4.2×10^{-12}		
Ho-159	0.550 h	M	0.005	4.6×10^{-11}	5.0×10^{-4}	3.3×10^{-11}	1.7×10^{-11}	1.1×10^{-11}	1.1×10^{-11}	7.5×10^{-12}	6.1×10^{-12}		
Ho-161	2.50 h	M	0.005	5.7×10^{-11}	5.0×10^{-4}	4.0×10^{-11}	2.0×10^{-11}	1.2×10^{-11}	1.2×10^{-11}	7.5×10^{-12}	6.0×10^{-12}		
Ho-162	0.250 h	M	0.005	2.1×10^{-11}	5.0×10^{-4}	1.5×10^{-11}	7.2×10^{-12}	4.8×10^{-12}	4.8×10^{-12}	3.4×10^{-12}	2.8×10^{-12}		
Ho-162m	1.13 h	M	0.005	1.5×10^{-10}	5.0×10^{-4}	1.1×10^{-10}	5.8×10^{-11}	3.8×10^{-11}	3.8×10^{-11}	2.6×10^{-11}	2.1×10^{-11}		
Ho-164	0.483 h	M	0.005	6.8×10^{-11}	5.0×10^{-4}	4.5×10^{-11}	2.1×10^{-11}	1.4×10^{-11}	1.4×10^{-11}	9.9×10^{-12}	8.4×10^{-12}		
Ho-164m	0.625 h	M	0.005	9.1×10^{-11}	5.0×10^{-4}	5.9×10^{-11}	3.0×10^{-11}	2.0×10^{-11}	2.0×10^{-11}	1.3×10^{-11}	1.2×10^{-11}		
Ho-166	1.12 d	M	0.005	6.0×10^{-9}	5.0×10^{-4}	4.0×10^{-9}	1.9×10^{-9}	1.2×10^{-9}	1.2×10^{-9}	7.9×10^{-10}	6.5×10^{-10}		
Ho-166m	1.20×10^3 a	M	0.005	2.6×10^{-7}	5.0×10^{-4}	2.5×10^{-7}	1.8×10^{-7}	1.3×10^{-7}	1.3×10^{-7}	1.2×10^{-7}	1.2×10^{-7}		
Ho-167	3.10 h	M	0.005	5.2×10^{-10}	5.0×10^{-4}	3.6×10^{-10}	1.8×10^{-10}	1.2×10^{-10}	1.2×10^{-10}	8.7×10^{-11}	7.1×10^{-11}		

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Erbium													
Er-161	3.24 h	M	0.005	3.8×10^{-10}	5.0×10^{-4}	2.9×10^{-10}	1.5×10^{-10}	9.5×10^{-11}	6.0×10^{-11}	4.8×10^{-11}			
Er-165	10.4 h	M	0.005	7.2×10^{-11}	5.0×10^{-4}	5.3×10^{-11}	2.6×10^{-11}	1.6×10^{-11}	9.6×10^{-12}	7.9×10^{-12}			
Er-169	9.30 d	M	0.005	4.7×10^{-9}	5.0×10^{-4}	3.5×10^{-9}	2.0×10^{-9}	1.5×10^{-9}	1.3×10^{-9}	1.0×10^{-9}			
Er-171	7.52 h	M	0.005	1.8×10^{-9}	5.0×10^{-4}	1.2×10^{-9}	5.9×10^{-10}	3.9×10^{-10}	2.7×10^{-10}	2.2×10^{-10}			
Er-172	2.05 d	M	0.005	6.6×10^{-9}	5.0×10^{-4}	4.7×10^{-9}	2.5×10^{-9}	1.7×10^{-9}	1.4×10^{-9}	1.1×10^{-9}			
Thulium													
Tm-162	0.362 h	M	0.005	1.3×10^{-10}	5.0×10^{-4}	9.6×10^{-11}	4.7×10^{-11}	3.0×10^{-11}	1.9×10^{-11}	1.6×10^{-11}			
Tm-166	7.70 h	M	0.005	1.3×10^{-9}	5.0×10^{-4}	9.9×10^{-10}	5.2×10^{-10}	3.3×10^{-10}	2.2×10^{-10}	1.7×10^{-10}			
Tm-167	9.24 d	M	0.005	5.6×10^{-9}	5.0×10^{-4}	4.1×10^{-9}	2.3×10^{-9}	1.7×10^{-9}	1.4×10^{-9}	1.1×10^{-9}			
Tm-170	129 d	M	0.005	3.6×10^{-8}	5.0×10^{-4}	2.8×10^{-8}	1.6×10^{-8}	1.1×10^{-8}	8.5×10^{-9}	7.0×10^{-9}			
Tm-171	1.92 a	M	0.005	6.8×10^{-9}	5.0×10^{-4}	5.7×10^{-9}	3.4×10^{-9}	2.0×10^{-9}	1.6×10^{-9}	1.4×10^{-9}			
Tm-172	2.65 d	M	0.005	8.4×10^{-9}	5.0×10^{-4}	5.8×10^{-9}	2.9×10^{-9}	1.9×10^{-9}	1.4×10^{-9}	1.1×10^{-9}			
Tm-173	8.24 h	M	0.005	1.5×10^{-9}	5.0×10^{-4}	1.0×10^{-9}	5.0×10^{-10}	3.3×10^{-10}	2.2×10^{-10}	1.8×10^{-10}			
Tm-175	0.253 h	M	0.005	1.6×10^{-10}	5.0×10^{-4}	1.1×10^{-10}	5.0×10^{-11}	3.3×10^{-11}	2.2×10^{-11}	1.8×10^{-11}			
Ytterbium													
Yb-162	0.315 h	M	0.005	1.1×10^{-10}	5.0×10^{-4}	7.9×10^{-11}	3.9×10^{-11}	2.5×10^{-11}	1.6×10^{-11}	1.3×10^{-11}			
		S	0.005	1.2×10^{-10}	5.0×10^{-4}	8.2×10^{-11}	4.0×10^{-11}	2.6×10^{-11}	1.7×10^{-11}	1.4×10^{-11}			
Yb-166	2.36 d	M	0.005	4.7×10^{-9}	5.0×10^{-4}	3.5×10^{-9}	1.9×10^{-9}	1.3×10^{-9}	9.0×10^{-10}	7.2×10^{-10}			
		S	0.005	4.9×10^{-9}	5.0×10^{-4}	3.7×10^{-9}	2.0×10^{-9}	1.3×10^{-9}	9.6×10^{-10}	7.7×10^{-10}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Yb-167	0.292 h	M	0.005	4.4×10^{-11}	5.0×10^{-4}	3.1×10^{-11}	1.6×10^{-11}	1.1×10^{-11}	1.1×10^{-11}	7.9×10^{-12}	6.5×10^{-12}		
		S	0.005	4.6×10^{-11}	5.0×10^{-4}	3.2×10^{-11}	1.7×10^{-11}	1.1×10^{-11}	1.1×10^{-11}	8.4×10^{-12}	6.9×10^{-12}		
Yb-169	32.0 d	M	0.005	1.2×10^{-8}	5.0×10^{-4}	8.7×10^{-9}	5.1×10^{-9}	3.7×10^{-9}	3.7×10^{-9}	3.2×10^{-9}	2.5×10^{-9}		
		S	0.005	1.3×10^{-8}	5.0×10^{-4}	9.8×10^{-9}	5.9×10^{-9}	4.2×10^{-9}	4.2×10^{-9}	3.7×10^{-9}	3.0×10^{-9}		
Yb-175	4.19 d	M	0.005	3.5×10^{-9}	5.0×10^{-4}	2.5×10^{-9}	1.4×10^{-9}	9.8×10^{-10}	9.8×10^{-10}	8.3×10^{-10}	6.5×10^{-10}		
		S	0.005	3.7×10^{-9}	5.0×10^{-4}	2.7×10^{-9}	1.5×10^{-9}	1.1×10^{-9}	1.1×10^{-9}	9.2×10^{-10}	7.3×10^{-10}		
Yb-177	1.90 h	M	0.005	5.0×10^{-10}	5.0×10^{-4}	3.3×10^{-10}	1.6×10^{-10}	1.1×10^{-10}	1.1×10^{-10}	7.8×10^{-11}	6.4×10^{-11}		
		S	0.005	5.3×10^{-10}	5.0×10^{-4}	3.5×10^{-10}	1.7×10^{-10}	1.2×10^{-10}	1.2×10^{-10}	8.4×10^{-11}	6.9×10^{-11}		
Yb-178	1.23 h	M	0.005	5.9×10^{-10}	5.0×10^{-4}	3.9×10^{-10}	1.8×10^{-10}	1.2×10^{-10}	1.2×10^{-10}	8.5×10^{-11}	7.0×10^{-11}		
		S	0.005	6.2×10^{-10}	5.0×10^{-4}	4.1×10^{-10}	1.9×10^{-10}	1.3×10^{-10}	1.3×10^{-10}	9.1×10^{-11}	7.5×10^{-11}		
Lutetium													
Lu-169	1.42 d	M	0.005	2.3×10^{-9}	5.0×10^{-4}	1.8×10^{-9}	9.5×10^{-10}	6.3×10^{-10}	6.3×10^{-10}	4.4×10^{-10}	3.5×10^{-10}		
		S	0.005	2.4×10^{-9}	5.0×10^{-4}	1.9×10^{-9}	1.0×10^{-9}	6.7×10^{-10}	6.7×10^{-10}	4.8×10^{-10}	3.8×10^{-10}		
Lu-170	2.00 d	M	0.005	4.3×10^{-9}	5.0×10^{-4}	3.4×10^{-9}	1.8×10^{-9}	1.2×10^{-9}	1.2×10^{-9}	7.8×10^{-10}	6.3×10^{-10}		
		S	0.005	4.5×10^{-9}	5.0×10^{-4}	3.5×10^{-9}	1.8×10^{-9}	1.2×10^{-9}	1.2×10^{-9}	8.2×10^{-10}	6.6×10^{-10}		
Lu-171	8.22 d	M	0.005	5.0×10^{-9}	5.0×10^{-4}	3.7×10^{-9}	2.1×10^{-9}	1.2×10^{-9}	1.2×10^{-9}	9.8×10^{-10}	8.0×10^{-10}		
		S	0.005	4.7×10^{-9}	5.0×10^{-4}	3.9×10^{-9}	2.0×10^{-9}	1.4×10^{-9}	1.4×10^{-9}	1.1×10^{-9}	8.8×10^{-10}		
Lu-172	6.70 d	M	0.005	8.7×10^{-9}	5.0×10^{-4}	6.7×10^{-9}	3.8×10^{-9}	2.6×10^{-9}	2.6×10^{-9}	1.8×10^{-9}	1.4×10^{-9}		
		S	0.005	9.3×10^{-9}	5.0×10^{-4}	7.1×10^{-9}	4.0×10^{-9}	2.8×10^{-9}	2.8×10^{-9}	2.0×10^{-9}	1.6×10^{-9}		
Lu-173	1.37 a	M	0.005	1.0×10^{-8}	5.0×10^{-4}	8.5×10^{-9}	5.1×10^{-9}	3.2×10^{-9}	3.2×10^{-9}	2.5×10^{-9}	2.2×10^{-9}		
		S	0.005	1.0×10^{-8}	5.0×10^{-4}	8.7×10^{-9}	5.4×10^{-9}	3.6×10^{-9}	3.6×10^{-9}	2.9×10^{-9}	2.4×10^{-9}		

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$		$e(g)$		$e(g)$		$e(g)$		
Lu-174	3.31 a	M	0.005	1.7×10^{-8}	5.0×10^{-4}	1.5×10^{-8}	9.1×10^{-9}	5.8×10^{-9}	4.7×10^{-9}	4.2×10^{-9}			
		S	0.005	1.6×10^{-8}	5.0×10^{-4}	1.4×10^{-8}	8.9×10^{-9}	5.9×10^{-9}	4.9×10^{-9}	4.2×10^{-9}			
Lu-174m	142 d	M	0.005	1.9×10^{-8}	5.0×10^{-4}	1.4×10^{-8}	8.6×10^{-9}	5.4×10^{-9}	4.3×10^{-9}	3.7×10^{-9}			
		S	0.005	2.0×10^{-8}	5.0×10^{-4}	1.5×10^{-8}	9.2×10^{-9}	6.1×10^{-9}	5.0×10^{-9}	4.2×10^{-9}			
Lu-176	3.60×10^{10} a	M	0.005	1.8×10^{-7}	5.0×10^{-4}	1.7×10^{-7}	1.1×10^{-7}	7.8×10^{-8}	7.1×10^{-8}	7.0×10^{-8}			
		S	0.005	1.5×10^{-7}	5.0×10^{-4}	1.4×10^{-7}	9.4×10^{-8}	6.5×10^{-8}	5.9×10^{-8}	5.6×10^{-8}			
Lu-176m	3.68 h	M	0.005	8.9×10^{-10}	5.0×10^{-4}	5.9×10^{-10}	2.8×10^{-10}	1.9×10^{-10}	1.2×10^{-10}	1.1×10^{-10}			
		S	0.005	9.3×10^{-10}	5.0×10^{-4}	6.2×10^{-10}	3.0×10^{-10}	2.0×10^{-10}	1.2×10^{-10}	1.2×10^{-10}			
Lu-177	6.71 d	M	0.005	5.3×10^{-9}	5.0×10^{-4}	3.8×10^{-9}	2.2×10^{-9}	1.6×10^{-9}	1.4×10^{-9}	1.1×10^{-9}			
		S	0.005	5.7×10^{-9}	5.0×10^{-4}	4.1×10^{-9}	2.4×10^{-9}	1.7×10^{-9}	1.5×10^{-9}	1.2×10^{-9}			
Lu-177m	161 d	M	0.005	5.8×10^{-8}	5.0×10^{-4}	4.6×10^{-8}	2.8×10^{-8}	1.9×10^{-8}	1.6×10^{-8}	1.3×10^{-8}			
		S	0.005	6.5×10^{-8}	5.0×10^{-4}	5.3×10^{-8}	3.2×10^{-8}	2.3×10^{-8}	2.0×10^{-8}	1.6×10^{-8}			
Lu-178	0.473 h	M	0.005	2.3×10^{-10}	5.0×10^{-4}	1.5×10^{-10}	6.6×10^{-11}	4.3×10^{-11}	2.9×10^{-11}	2.4×10^{-11}			
		S	0.005	2.4×10^{-10}	5.0×10^{-4}	1.5×10^{-10}	6.9×10^{-11}	4.5×10^{-11}	3.0×10^{-11}	2.6×10^{-11}			
Lu-178m	0.378 h	M	0.005	2.6×10^{-10}	5.0×10^{-4}	1.8×10^{-10}	8.3×10^{-11}	5.6×10^{-11}	3.8×10^{-11}	3.2×10^{-11}			
		S	0.005	2.7×10^{-10}	5.0×10^{-4}	1.9×10^{-10}	8.7×10^{-11}	5.8×10^{-11}	4.0×10^{-11}	3.3×10^{-11}			
Lu-179	4.59 h	M	0.005	9.9×10^{-10}	5.0×10^{-4}	6.5×10^{-10}	3.0×10^{-10}	2.0×10^{-10}	1.2×10^{-10}	1.1×10^{-10}			
		S	0.005	1.0×10^{-9}	5.0×10^{-4}	6.8×10^{-10}	3.2×10^{-10}	2.1×10^{-10}	1.3×10^{-10}	1.2×10^{-10}			
Hafnium													
Hf-170	16.0 h	F	0.020	1.4×10^{-9}	0.002	1.1×10^{-9}	5.4×10^{-10}	3.4×10^{-10}	2.0×10^{-10}	1.6×10^{-10}			
		M	0.020	2.2×10^{-9}	0.002	1.7×10^{-9}	8.7×10^{-10}	5.8×10^{-10}	3.9×10^{-10}	3.2×10^{-10}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$				
Hf-172	1.87 a	F	0.020	1.5×10^{-7}	0.002	1.3×10^{-7}	7.8×10^{-8}	4.9×10^{-8}	3.5×10^{-8}	3.2×10^{-8}			
		M	0.020	8.1×10^{-8}	0.002	6.9×10^{-8}	4.3×10^{-8}	2.8×10^{-8}	2.3×10^{-8}	2.0×10^{-8}			
Hf-173	24.0 h	F	0.020	6.6×10^{-10}	0.002	5.0×10^{-10}	2.5×10^{-10}	1.5×10^{-10}	8.9×10^{-11}	7.4×10^{-11}			
		M	0.020	1.1×10^{-9}	0.002	8.2×10^{-10}	4.3×10^{-10}	2.9×10^{-10}	2.0×10^{-10}	1.6×10^{-10}			
Hf-175	70.0 d	F	0.020	5.4×10^{-9}	0.002	4.0×10^{-9}	2.1×10^{-9}	1.3×10^{-9}	8.5×10^{-10}	7.2×10^{-10}			
		M	0.020	5.8×10^{-9}	0.002	4.5×10^{-9}	2.6×10^{-9}	1.8×10^{-9}	1.4×10^{-9}	1.2×10^{-9}			
Hf-177m	0.856 h	F	0.020	3.9×10^{-10}	0.002	2.8×10^{-10}	1.3×10^{-10}	8.5×10^{-11}	5.2×10^{-11}	4.4×10^{-11}			
		M	0.020	6.5×10^{-10}	0.002	4.7×10^{-10}	2.3×10^{-10}	1.5×10^{-10}	1.1×10^{-10}	9.0×10^{-11}			
Hf-178m	31.0 a	F	0.020	6.2×10^{-7}	0.002	5.8×10^{-7}	4.0×10^{-7}	3.1×10^{-7}	2.7×10^{-7}	2.6×10^{-7}			
		M	0.020	2.6×10^{-7}	0.002	2.4×10^{-7}	1.7×10^{-7}	1.3×10^{-7}	1.2×10^{-7}	1.2×10^{-7}			
Hf-179m	25.1 d	F	0.020	9.7×10^{-9}	0.002	6.8×10^{-9}	3.4×10^{-9}	2.1×10^{-9}	1.2×10^{-9}	1.1×10^{-9}			
		M	0.020	1.7×10^{-8}	0.002	1.3×10^{-8}	7.6×10^{-9}	5.5×10^{-9}	4.8×10^{-9}	3.8×10^{-9}			
Hf-180m	5.50 h	F	0.020	5.4×10^{-10}	0.002	4.1×10^{-10}	2.0×10^{-10}	1.3×10^{-10}	7.2×10^{-11}	5.9×10^{-11}			
		M	0.020	9.1×10^{-10}	0.002	6.8×10^{-10}	3.6×10^{-10}	2.4×10^{-10}	1.7×10^{-10}	1.3×10^{-10}			
Hf-181	42.4 d	F	0.020	1.3×10^{-8}	0.002	9.6×10^{-9}	4.8×10^{-9}	2.8×10^{-9}	1.7×10^{-9}	1.4×10^{-9}			
		M	0.020	2.2×10^{-8}	0.002	1.7×10^{-8}	9.9×10^{-9}	7.1×10^{-9}	6.3×10^{-9}	5.0×10^{-9}			
Hf-182	9.00×10^6 a	F	0.020	6.5×10^{-7}	0.002	6.2×10^{-7}	4.4×10^{-7}	3.6×10^{-7}	3.1×10^{-7}	3.1×10^{-7}			
		M	0.020	2.4×10^{-7}	0.002	2.3×10^{-7}	1.7×10^{-7}	1.3×10^{-7}	1.3×10^{-7}	1.3×10^{-7}			
Hf-182m	1.02 h	F	0.020	1.9×10^{-10}	0.002	1.4×10^{-10}	6.6×10^{-11}	4.2×10^{-11}	2.6×10^{-11}	2.1×10^{-11}			
		M	0.020	3.2×10^{-10}	0.002	2.3×10^{-10}	1.2×10^{-10}	7.8×10^{-11}	5.6×10^{-11}	4.6×10^{-11}			
Hf-183	1.07 h	F	0.020	2.5×10^{-10}	0.002	1.7×10^{-10}	7.9×10^{-11}	4.9×10^{-11}	2.8×10^{-11}	2.4×10^{-11}			
		M	0.020	4.4×10^{-10}	0.002	3.0×10^{-10}	1.5×10^{-10}	9.8×10^{-11}	7.0×10^{-11}	5.7×10^{-11}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)			
Hf-184	4.12 h	F	0.020	1.4 × 10 ^{−9}	0.002	9.6 × 10 ^{−10}	4.3 × 10 ^{−10}	4.3 × 10 ^{−10}	2.7 × 10 ^{−10}	2.7 × 10 ^{−10}	1.4 × 10 ^{−10}	1.4 × 10 ^{−10}	1.2 × 10 ^{−10}
		M	0.020	2.6 × 10 ^{−9}	0.002	1.8 × 10 ^{−9}	8.9 × 10 ^{−10}	8.9 × 10 ^{−10}	5.9 × 10 ^{−10}	5.9 × 10 ^{−10}	4.0 × 10 ^{−10}	4.0 × 10 ^{−10}	3.3 × 10 ^{−10}
Tantalum													
Ta-172	0.613 h	M	0.010	2.8 × 10 ^{−10}	0.001	1.9 × 10 ^{−10}	9.3 × 10 ^{−11}	9.3 × 10 ^{−11}	6.0 × 10 ^{−11}	6.0 × 10 ^{−11}	4.0 × 10 ^{−11}	4.0 × 10 ^{−11}	3.3 × 10 ^{−11}
		S	0.010	2.9 × 10 ^{−10}	0.001	2.0 × 10 ^{−10}	9.8 × 10 ^{−11}	9.8 × 10 ^{−11}	6.3 × 10 ^{−11}	6.3 × 10 ^{−11}	4.2 × 10 ^{−11}	4.2 × 10 ^{−11}	3.5 × 10 ^{−11}
Ta-173	3.65 h	M	0.010	8.8 × 10 ^{−10}	0.001	6.2 × 10 ^{−10}	3.0 × 10 ^{−10}	3.0 × 10 ^{−10}	2.0 × 10 ^{−10}	2.0 × 10 ^{−10}	1.3 × 10 ^{−10}	1.3 × 10 ^{−10}	1.1 × 10 ^{−10}
		S	0.010	9.2 × 10 ^{−10}	0.001	6.5 × 10 ^{−10}	3.2 × 10 ^{−10}	3.2 × 10 ^{−10}	2.1 × 10 ^{−10}	2.1 × 10 ^{−10}	1.4 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}
Ta-174	1.20 h	M	0.010	3.2 × 10 ^{−10}	0.001	2.2 × 10 ^{−10}	1.1 × 10 ^{−10}	1.1 × 10 ^{−10}	7.1 × 10 ^{−11}	7.1 × 10 ^{−11}	5.0 × 10 ^{−11}	5.0 × 10 ^{−11}	4.1 × 10 ^{−11}
		S	0.010	3.4 × 10 ^{−10}	0.001	2.3 × 10 ^{−10}	1.1 × 10 ^{−10}	1.1 × 10 ^{−10}	7.5 × 10 ^{−11}	7.5 × 10 ^{−11}	5.3 × 10 ^{−11}	5.3 × 10 ^{−11}	4.3 × 10 ^{−11}
Ta-175	10.5 h	M	0.010	9.1 × 10 ^{−10}	0.001	7.0 × 10 ^{−10}	3.7 × 10 ^{−10}	3.7 × 10 ^{−10}	2.4 × 10 ^{−10}	2.4 × 10 ^{−10}	1.5 × 10 ^{−10}	1.5 × 10 ^{−10}	1.2 × 10 ^{−10}
		S	0.010	9.5 × 10 ^{−10}	0.001	7.3 × 10 ^{−10}	3.8 × 10 ^{−10}	3.8 × 10 ^{−10}	2.5 × 10 ^{−10}	2.5 × 10 ^{−10}	1.6 × 10 ^{−10}	1.6 × 10 ^{−10}	1.3 × 10 ^{−10}
Ta-176	8.08 h	M	0.010	1.4 × 10 ^{−9}	0.001	1.1 × 10 ^{−9}	5.7 × 10 ^{−10}	5.7 × 10 ^{−10}	3.7 × 10 ^{−10}	3.7 × 10 ^{−10}	2.4 × 10 ^{−10}	2.4 × 10 ^{−10}	1.9 × 10 ^{−10}
		S	0.010	1.4 × 10 ^{−9}	0.001	1.1 × 10 ^{−9}	5.9 × 10 ^{−10}	5.9 × 10 ^{−10}	3.8 × 10 ^{−10}	3.8 × 10 ^{−10}	2.5 × 10 ^{−10}	2.5 × 10 ^{−10}	2.0 × 10 ^{−10}
Ta-177	2.36 d	M	0.010	6.5 × 10 ^{−10}	0.001	4.7 × 10 ^{−10}	2.5 × 10 ^{−10}	2.5 × 10 ^{−10}	1.5 × 10 ^{−10}	1.5 × 10 ^{−10}	1.2 × 10 ^{−10}	1.2 × 10 ^{−10}	9.6 × 10 ^{−11}
		S	0.010	6.9 × 10 ^{−10}	0.001	5.0 × 10 ^{−10}	2.7 × 10 ^{−10}	2.7 × 10 ^{−10}	1.7 × 10 ^{−10}	1.7 × 10 ^{−10}	1.3 × 10 ^{−10}	1.3 × 10 ^{−10}	1.1 × 10 ^{−10}
Ta-178	2.20 h	M	0.010	4.4 × 10 ^{−10}	0.001	3.3 × 10 ^{−10}	1.7 × 10 ^{−10}	1.7 × 10 ^{−10}	1.1 × 10 ^{−10}	1.1 × 10 ^{−10}	8.0 × 10 ^{−11}	8.0 × 10 ^{−11}	6.5 × 10 ^{−11}
		S	0.010	4.6 × 10 ^{−10}	0.001	3.4 × 10 ^{−10}	1.8 × 10 ^{−10}	1.8 × 10 ^{−10}	1.2 × 10 ^{−10}	1.2 × 10 ^{−10}	8.5 × 10 ^{−11}	8.5 × 10 ^{−11}	6.8 × 10 ^{−11}
Ta-179	1.82 a	M	0.010	1.2 × 10 ^{−9}	0.001	9.6 × 10 ^{−10}	5.5 × 10 ^{−10}	5.5 × 10 ^{−10}	3.5 × 10 ^{−10}	3.5 × 10 ^{−10}	2.6 × 10 ^{−10}	2.6 × 10 ^{−10}	2.2 × 10 ^{−10}
		S	0.010	2.4 × 10 ^{−9}	0.001	2.1 × 10 ^{−9}	1.3 × 10 ^{−9}	1.3 × 10 ^{−9}	8.3 × 10 ^{−10}	8.3 × 10 ^{−10}	6.4 × 10 ^{−10}	6.4 × 10 ^{−10}	5.6 × 10 ^{−10}
Ta-180	1.00 × 10 ¹³ a	M	0.010	2.7 × 10 ^{−8}	0.001	2.2 × 10 ^{−8}	1.3 × 10 ^{−8}	1.3 × 10 ^{−8}	9.2 × 10 ^{−9}	9.2 × 10 ^{−9}	7.9 × 10 ^{−9}	7.9 × 10 ^{−9}	6.4 × 10 ^{−9}
		S	0.010	7.0 × 10 ^{−8}	0.001	6.5 × 10 ^{−8}	4.5 × 10 ^{−8}	4.5 × 10 ^{−8}	3.1 × 10 ^{−8}	3.1 × 10 ^{−8}	2.8 × 10 ^{−8}	2.8 × 10 ^{−8}	2.6 × 10 ^{−8}

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Ta-180m	8.10 h	M	0.010	3.1×10^{-10}	0.001	2.2×10^{-10}	1.1×10^{-10}	7.4×10^{-11}	4.8×10^{-11}	4.4×10^{-11}			
		S	0.010	3.3×10^{-10}	0.001	2.3×10^{-10}	1.2×10^{-10}	7.9×10^{-11}	5.2×10^{-11}	4.2×10^{-11}			
Ta-182	115 d	M	0.010	3.2×10^{-8}	0.001	2.6×10^{-8}	1.5×10^{-8}	1.1×10^{-8}	9.5×10^{-9}	7.6×10^{-9}			
		S	0.010	4.2×10^{-8}	0.001	3.4×10^{-8}	2.1×10^{-8}	1.5×10^{-8}	1.3×10^{-8}	1.0×10^{-8}			
Ta-182m	0.264 h	M	0.010	1.6×10^{-10}	0.001	1.1×10^{-10}	4.9×10^{-11}	3.4×10^{-11}	2.4×10^{-11}	2.0×10^{-11}			
		S	0.010	1.6×10^{-10}	0.001	1.1×10^{-10}	5.2×10^{-11}	3.6×10^{-11}	2.5×10^{-11}	2.1×10^{-11}			
Ta-183	5.10 d	M	0.010	1.0×10^{-8}	0.001	7.4×10^{-9}	4.1×10^{-9}	2.9×10^{-9}	2.4×10^{-9}	1.9×10^{-9}			
		S	0.010	1.1×10^{-8}	0.001	8.0×10^{-9}	4.5×10^{-9}	3.2×10^{-9}	2.7×10^{-9}	2.1×10^{-9}			
Ta-184	8.70 h	M	0.010	3.2×10^{-9}	0.001	2.3×10^{-9}	1.1×10^{-9}	7.5×10^{-10}	5.0×10^{-10}	4.1×10^{-10}			
		S	0.010	3.4×10^{-9}	0.001	2.4×10^{-9}	1.2×10^{-9}	7.9×10^{-10}	5.4×10^{-10}	4.3×10^{-10}			
Ta-185	0.816 h	M	0.010	3.8×10^{-10}	0.001	2.5×10^{-10}	1.2×10^{-10}	7.7×10^{-11}	5.4×10^{-11}	4.5×10^{-11}			
		S	0.010	4.0×10^{-10}	0.001	2.6×10^{-10}	1.2×10^{-10}	8.2×10^{-11}	5.7×10^{-11}	4.8×10^{-11}			
Ta-186	0.175 h	M	0.010	1.6×10^{-10}	0.001	1.1×10^{-10}	4.8×10^{-11}	3.1×10^{-11}	2.0×10^{-11}	1.7×10^{-11}			
		S	0.010	1.6×10^{-10}	0.001	1.1×10^{-10}	5.0×10^{-11}	3.2×10^{-11}	2.1×10^{-11}	1.8×10^{-11}			
Tungsten													
W-176	2.30 h	F	0.600	3.3×10^{-10}	0.300	2.7×10^{-10}	1.4×10^{-10}	8.6×10^{-11}	5.0×10^{-11}	4.1×10^{-11}			
W-177	2.25 h	F	0.600	2.0×10^{-10}	0.300	1.6×10^{-10}	8.2×10^{-11}	5.1×10^{-11}	3.0×10^{-11}	2.4×10^{-11}			
W-178	21.7 d	F	0.600	7.2×10^{-10}	0.300	5.4×10^{-10}	2.5×10^{-10}	1.6×10^{-10}	8.7×10^{-11}	7.2×10^{-11}			
W-179	0.625 h	F	0.600	9.3×10^{-12}	0.300	6.8×10^{-12}	3.3×10^{-12}	2.0×10^{-12}	1.2×10^{-12}	9.2×10^{-13}			
W-181	121 d	F	0.600	2.5×10^{-10}	0.300	1.9×10^{-10}	9.2×10^{-11}	5.7×10^{-11}	3.2×10^{-11}	2.7×10^{-11}			
W-185	75.1 d	F	0.600	1.4×10^{-9}	0.300	1.0×10^{-9}	4.4×10^{-10}	2.7×10^{-10}	1.4×10^{-10}	1.2×10^{-10}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1-2 a		2-7 a		7-12 a		12-17 a		>17 a
		Type	f_1		$e(g)$		$e(g)$		$e(g)$		$e(g)$		$e(g)$
W-187	23.9 h	F	0.600		2.0×10^{-9}		1.5×10^{-9}	7.0×10^{-10}	4.3×10^{-10}		2.3×10^{-10}		1.9×10^{-10}
W-188	69.4 d	F	0.600		7.1×10^{-9}		5.0×10^{-9}	2.2×10^{-9}	1.3×10^{-9}		6.8×10^{-10}		5.7×10^{-10}
Rhenium													
Re-177	0.233 h	F	1.000		9.4×10^{-11}		6.7×10^{-11}	3.2×10^{-11}	1.9×10^{-11}		1.2×10^{-11}		9.7×10^{-12}
		M	1.000		1.1×10^{-10}		7.9×10^{-11}	3.9×10^{-11}	2.5×10^{-11}		1.7×10^{-11}		1.4×10^{-11}
Re-178	0.220 h	F	1.000		9.9×10^{-11}		6.8×10^{-11}	3.1×10^{-11}	1.9×10^{-11}		1.2×10^{-11}		1.0×10^{-11}
		M	1.000		1.3×10^{-10}		8.5×10^{-11}	3.9×10^{-11}	2.6×10^{-11}		1.7×10^{-11}		1.4×10^{-11}
Re-181	20.0 h	F	1.000		2.0×10^{-9}		1.4×10^{-9}	6.7×10^{-10}	3.8×10^{-10}		2.3×10^{-10}		1.8×10^{-10}
		M	1.000		2.1×10^{-9}		1.5×10^{-9}	7.4×10^{-10}	4.6×10^{-10}		3.1×10^{-10}		2.5×10^{-10}
Re-182	2.67 d	F	1.000		6.5×10^{-9}		4.7×10^{-9}	2.2×10^{-9}	1.3×10^{-9}		8.0×10^{-10}		6.4×10^{-10}
		M	1.000		8.7×10^{-9}		6.3×10^{-9}	3.4×10^{-9}	2.2×10^{-9}		1.5×10^{-9}		1.2×10^{-9}
Re-182m	12.7 h	F	1.000		1.3×10^{-9}		1.0×10^{-9}	4.9×10^{-10}	2.8×10^{-10}		1.7×10^{-10}		1.4×10^{-10}
		M	1.000		1.4×10^{-9}		1.1×10^{-9}	5.7×10^{-10}	3.6×10^{-10}		2.5×10^{-10}		2.0×10^{-10}
Re-184	38.0 d	F	1.000		4.1×10^{-9}		2.9×10^{-9}	1.4×10^{-9}	8.6×10^{-10}		5.4×10^{-10}		4.4×10^{-10}
		M	1.000		9.1×10^{-9}		6.8×10^{-9}	4.0×10^{-9}	2.8×10^{-9}		2.4×10^{-9}		1.9×10^{-9}
Re-184m	165 d	F	1.000		6.6×10^{-9}		4.6×10^{-9}	2.0×10^{-9}	1.2×10^{-9}		7.3×10^{-10}		5.9×10^{-10}
		M	1.000		2.9×10^{-8}		2.2×10^{-8}	1.3×10^{-8}	9.3×10^{-9}		8.1×10^{-9}		6.5×10^{-9}
Re-186	3.78 d	F	1.000		7.3×10^{-9}		4.7×10^{-9}	2.0×10^{-9}	1.1×10^{-9}		6.6×10^{-10}		5.2×10^{-10}
		M	1.000		8.7×10^{-9}		5.7×10^{-9}	2.8×10^{-9}	1.8×10^{-9}		1.4×10^{-9}		1.1×10^{-9}
Re-186m	2.00×10^5 a	F	1.000		1.2×10^{-8}		7.0×10^{-9}	2.9×10^{-9}	1.7×10^{-9}		1.0×10^{-9}		8.3×10^{-10}
		M	1.000		5.9×10^{-8}		4.6×10^{-8}	2.7×10^{-8}	1.8×10^{-8}		1.4×10^{-8}		1.2×10^{-8}

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Re-187	5.00×10^{10} a	F	1.000	2.6×10^{-11}	0.800	1.6×10^{-11}	6.8×10^{-12}	3.8×10^{-12}	2.3×10^{-12}	2.3×10^{-12}	1.8×10^{-12}	1.8×10^{-12}	
		M	1.000	5.7×10^{-11}	0.800	4.1×10^{-11}	2.0×10^{-11}	1.2×10^{-11}	7.5×10^{-12}	7.5×10^{-12}	6.3×10^{-12}	6.3×10^{-12}	
Re-188	17.0 h	F	1.000	6.5×10^{-9}	0.800	4.4×10^{-9}	1.9×10^{-9}	1.0×10^{-9}	6.1×10^{-10}	6.1×10^{-10}	4.6×10^{-10}	4.6×10^{-10}	
		M	1.000	6.0×10^{-9}	0.800	4.0×10^{-9}	1.8×10^{-9}	1.0×10^{-9}	6.8×10^{-10}	6.8×10^{-10}	5.4×10^{-10}	5.4×10^{-10}	
Re-188m	0.310 h	F	1.000	1.4×10^{-10}	0.800	9.1×10^{-11}	4.0×10^{-11}	2.1×10^{-11}	1.3×10^{-11}	1.3×10^{-11}	1.0×10^{-11}	1.0×10^{-11}	
		M	1.000	1.3×10^{-10}	0.800	8.6×10^{-11}	4.0×10^{-11}	2.7×10^{-11}	1.6×10^{-11}	1.6×10^{-11}	1.3×10^{-11}	1.3×10^{-11}	
Re-189	1.01 d	F	1.000	3.7×10^{-9}	0.800	2.5×10^{-9}	1.1×10^{-9}	5.8×10^{-10}	3.5×10^{-10}	3.5×10^{-10}	2.7×10^{-10}	2.7×10^{-10}	
		M	1.000	3.9×10^{-9}	0.800	2.6×10^{-9}	1.2×10^{-9}	7.6×10^{-10}	5.5×10^{-10}	5.5×10^{-10}	4.3×10^{-10}	4.3×10^{-10}	
Osmium													
Os-180	0.366 h	F	0.020	7.1×10^{-11}	0.010	5.3×10^{-11}	2.6×10^{-11}	1.6×10^{-11}	1.0×10^{-11}	1.0×10^{-11}	8.2×10^{-12}	8.2×10^{-12}	
		M	0.020	1.1×10^{-10}	0.010	7.9×10^{-11}	3.9×10^{-11}	2.5×10^{-11}	1.7×10^{-11}	1.7×10^{-11}	1.4×10^{-11}	1.4×10^{-11}	
Os-181	1.75 h	S	0.020	1.1×10^{-10}	0.010	8.2×10^{-11}	4.1×10^{-11}	2.6×10^{-11}	1.8×10^{-11}	1.8×10^{-11}	1.5×10^{-11}	1.5×10^{-11}	
		F	0.020	3.0×10^{-10}	0.010	2.3×10^{-10}	1.1×10^{-10}	7.0×10^{-11}	4.1×10^{-11}	4.1×10^{-11}	3.3×10^{-11}	3.3×10^{-11}	
Os-182	22.0 h	M	0.020	4.5×10^{-10}	0.010	3.4×10^{-10}	1.8×10^{-10}	1.1×10^{-10}	7.6×10^{-11}	7.6×10^{-11}	6.2×10^{-11}	6.2×10^{-11}	
		S	0.020	4.7×10^{-10}	0.010	3.6×10^{-10}	1.8×10^{-10}	1.2×10^{-10}	8.1×10^{-11}	8.1×10^{-11}	6.5×10^{-11}	6.5×10^{-11}	
Os-185	94.0 d	F	0.020	1.6×10^{-9}	0.010	1.2×10^{-9}	6.0×10^{-10}	3.7×10^{-10}	2.1×10^{-10}	2.1×10^{-10}	1.7×10^{-10}	1.7×10^{-10}	
		M	0.020	2.5×10^{-9}	0.010	1.9×10^{-9}	1.0×10^{-9}	6.6×10^{-10}	4.5×10^{-10}	4.5×10^{-10}	3.6×10^{-10}	3.6×10^{-10}	
		S	0.020	2.6×10^{-9}	0.010	2.0×10^{-9}	1.0×10^{-9}	6.9×10^{-10}	4.8×10^{-10}	4.8×10^{-10}	3.8×10^{-10}	3.8×10^{-10}	
		F	0.020	7.2×10^{-9}	0.010	5.8×10^{-9}	3.1×10^{-9}	1.9×10^{-9}	1.2×10^{-9}	1.2×10^{-9}	1.1×10^{-9}	1.1×10^{-9}	
		M	0.020	6.6×10^{-9}	0.010	5.4×10^{-9}	2.9×10^{-9}	2.0×10^{-9}	1.5×10^{-9}	1.5×10^{-9}	1.3×10^{-9}	1.3×10^{-9}	
		S	0.020	7.0×10^{-9}	0.010	5.8×10^{-9}	3.6×10^{-9}	2.4×10^{-9}	1.9×10^{-9}	1.9×10^{-9}	1.6×10^{-9}	1.6×10^{-9}	

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a	
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	
Os-189m	6.00 h	F	0.020	3.8 × 10 ^{−11}	0.010	2.8 × 10 ^{−11}	1.2 × 10 ^{−11}	7.0 × 10 ^{−12}	3.5 × 10 ^{−12}	2.5 × 10 ^{−12}								
		M	0.020	6.5 × 10 ^{−11}	0.010	4.1 × 10 ^{−11}	1.8 × 10 ^{−11}	1.1 × 10 ^{−11}	6.0 × 10 ^{−12}	5.0 × 10 ^{−12}								
		S	0.020	6.8 × 10 ^{−11}	0.010	4.3 × 10 ^{−11}	1.9 × 10 ^{−11}	1.2 × 10 ^{−11}	6.3 × 10 ^{−12}	5.3 × 10 ^{−12}								
Os-191	15.4 d	F	0.020	2.8 × 10 ^{−9}	0.010	1.9 × 10 ^{−9}	8.5 × 10 ^{−10}	5.3 × 10 ^{−10}	3.0 × 10 ^{−10}	2.5 × 10 ^{−10}								
		M	0.020	8.0 × 10 ^{−9}	0.010	5.8 × 10 ^{−9}	3.4 × 10 ^{−9}	2.4 × 10 ^{−9}	2.0 × 10 ^{−9}	1.7 × 10 ^{−9}								
		S	0.020	9.0 × 10 ^{−9}	0.010	6.5 × 10 ^{−9}	3.9 × 10 ^{−9}	2.7 × 10 ^{−9}	2.3 × 10 ^{−9}	1.9 × 10 ^{−9}								
Os-191m	13.0 h	F	0.020	3.0 × 10 ^{−10}	0.010	2.0 × 10 ^{−10}	8.8 × 10 ^{−11}	5.4 × 10 ^{−11}	2.9 × 10 ^{−11}	2.4 × 10 ^{−11}								
		M	0.020	7.8 × 10 ^{−10}	0.010	5.4 × 10 ^{−10}	3.1 × 10 ^{−10}	2.1 × 10 ^{−10}	1.7 × 10 ^{−10}	1.4 × 10 ^{−10}								
		S	0.020	8.5 × 10 ^{−10}	0.010	6.0 × 10 ^{−10}	3.4 × 10 ^{−10}	2.4 × 10 ^{−10}	2.0 × 10 ^{−10}	1.6 × 10 ^{−10}								
Os-193	1.25 d	F	0.020	1.9 × 10 ^{−9}	0.010	1.2 × 10 ^{−9}	5.2 × 10 ^{−10}	3.2 × 10 ^{−10}	1.8 × 10 ^{−10}	1.6 × 10 ^{−10}								
		M	0.020	3.8 × 10 ^{−9}	0.010	2.6 × 10 ^{−9}	1.3 × 10 ^{−9}	8.4 × 10 ^{−10}	5.9 × 10 ^{−10}	4.8 × 10 ^{−10}								
		S	0.020	4.0 × 10 ^{−9}	0.010	2.7 × 10 ^{−9}	1.3 × 10 ^{−9}	9.0 × 10 ^{−10}	6.4 × 10 ^{−10}	5.2 × 10 ^{−10}								
Os-194	6.00 a	F	0.020	8.7 × 10 ^{−8}	0.010	6.8 × 10 ^{−8}	3.4 × 10 ^{−8}	2.1 × 10 ^{−8}	1.3 × 10 ^{−8}	1.1 × 10 ^{−8}								
		M	0.020	9.9 × 10 ^{−8}	0.010	8.3 × 10 ^{−8}	4.8 × 10 ^{−8}	3.1 × 10 ^{−8}	2.4 × 10 ^{−8}	2.1 × 10 ^{−8}								
		S	0.020	2.6 × 10 ^{−7}	0.010	2.4 × 10 ^{−7}	1.6 × 10 ^{−7}	1.1 × 10 ^{−7}	8.8 × 10 ^{−8}	8.5 × 10 ^{−8}								
Iridium																		
Ir-182	0.250 h	F	0.020	1.4 × 10 ^{−10}	0.010	9.8 × 10 ^{−11}	4.5 × 10 ^{−11}	2.8 × 10 ^{−11}	1.7 × 10 ^{−11}	1.4 × 10 ^{−11}								
		M	0.020	2.1 × 10 ^{−10}	0.010	1.4 × 10 ^{−10}	6.7 × 10 ^{−11}	4.3 × 10 ^{−11}	2.8 × 10 ^{−11}	2.3 × 10 ^{−11}								
		S	0.020	2.2 × 10 ^{−10}	0.010	1.5 × 10 ^{−10}	6.9 × 10 ^{−11}	4.4 × 10 ^{−11}	2.9 × 10 ^{−11}	2.4 × 10 ^{−11}								

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		> 17 a
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)			
Ir-184	3.02 h	F	0.020	5.7 × 10 ^{−10}	0.010	4.4 × 10 ^{−10}	2.1 × 10 ^{−10}	1.3 × 10 ^{−10}	7.6 × 10 ^{−11}	6.2 × 10 ^{−11}			
		M	0.020	8.6 × 10 ^{−10}	0.010	6.4 × 10 ^{−10}	3.2 × 10 ^{−10}	2.1 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}			
		S	0.020	8.9 × 10 ^{−10}	0.010	6.6 × 10 ^{−10}	3.4 × 10 ^{−10}	2.2 × 10 ^{−10}	1.4 × 10 ^{−10}	1.2 × 10 ^{−10}			
Ir-185	14.0 h	F	0.020	8.0 × 10 ^{−10}	0.010	6.1 × 10 ^{−10}	2.9 × 10 ^{−10}	1.8 × 10 ^{−10}	1.0 × 10 ^{−10}	8.2 × 10 ^{−11}			
		M	0.020	1.3 × 10 ^{−9}	0.010	9.7 × 10 ^{−10}	4.9 × 10 ^{−10}	3.2 × 10 ^{−10}	2.2 × 10 ^{−10}	1.8 × 10 ^{−10}			
		S	0.020	1.4 × 10 ^{−9}	0.010	1.0 × 10 ^{−9}	5.2 × 10 ^{−10}	3.4 × 10 ^{−10}	2.3 × 10 ^{−10}	1.9 × 10 ^{−10}			
Ir-186	15.8 h	F	0.020	1.5 × 10 ^{−9}	0.010	1.2 × 10 ^{−9}	5.9 × 10 ^{−10}	3.6 × 10 ^{−10}	2.1 × 10 ^{−10}	1.7 × 10 ^{−10}			
		M	0.020	2.2 × 10 ^{−9}	0.010	1.7 × 10 ^{−9}	8.8 × 10 ^{−10}	5.8 × 10 ^{−10}	3.8 × 10 ^{−10}	3.1 × 10 ^{−10}			
		S	0.020	2.3 × 10 ^{−9}	0.010	1.8 × 10 ^{−9}	9.2 × 10 ^{−10}	6.0 × 10 ^{−10}	4.0 × 10 ^{−10}	3.2 × 10 ^{−10}			
Ir-186m	1.75 h	F	0.020	2.1 × 10 ^{−10}	0.010	1.6 × 10 ^{−10}	7.7 × 10 ^{−11}	4.8 × 10 ^{−11}	2.8 × 10 ^{−11}	2.3 × 10 ^{−11}			
		M	0.020	3.3 × 10 ^{−10}	0.010	2.4 × 10 ^{−10}	1.2 × 10 ^{−10}	7.7 × 10 ^{−11}	5.1 × 10 ^{−11}	4.2 × 10 ^{−11}			
		S	0.020	3.4 × 10 ^{−10}	0.010	2.5 × 10 ^{−10}	1.2 × 10 ^{−10}	8.1 × 10 ^{−11}	5.4 × 10 ^{−11}	4.4 × 10 ^{−11}			
Ir-187	10.5 h	F	0.020	3.6 × 10 ^{−10}	0.010	2.8 × 10 ^{−10}	1.4 × 10 ^{−10}	8.2 × 10 ^{−11}	4.6 × 10 ^{−11}	3.7 × 10 ^{−11}			
		M	0.020	5.8 × 10 ^{−10}	0.010	4.3 × 10 ^{−10}	2.2 × 10 ^{−10}	1.4 × 10 ^{−10}	9.2 × 10 ^{−11}	7.4 × 10 ^{−11}			
		S	0.020	6.0 × 10 ^{−10}	0.010	4.5 × 10 ^{−10}	2.3 × 10 ^{−10}	1.5 × 10 ^{−10}	9.7 × 10 ^{−11}	7.9 × 10 ^{−11}			
Ir-188	1.73 d	F	0.020	2.0 × 10 ^{−9}	0.010	1.6 × 10 ^{−9}	8.0 × 10 ^{−10}	5.0 × 10 ^{−10}	2.9 × 10 ^{−10}	2.4 × 10 ^{−10}			
		M	0.020	2.7 × 10 ^{−9}	0.010	2.1 × 10 ^{−9}	1.1 × 10 ^{−9}	7.5 × 10 ^{−10}	5.0 × 10 ^{−10}	4.0 × 10 ^{−10}			
		S	0.020	2.8 × 10 ^{−9}	0.010	2.2 × 10 ^{−9}	1.2 × 10 ^{−9}	7.8 × 10 ^{−10}	5.2 × 10 ^{−10}	4.2 × 10 ^{−10}			
Ir-189	13.3 d	F	0.020	1.2 × 10 ^{−9}	0.010	8.2 × 10 ^{−10}	3.8 × 10 ^{−10}	2.4 × 10 ^{−10}	1.3 × 10 ^{−10}	1.1 × 10 ^{−10}			
		M	0.020	2.7 × 10 ^{−9}	0.010	1.9 × 10 ^{−9}	1.1 × 10 ^{−9}	7.7 × 10 ^{−10}	6.4 × 10 ^{−10}	5.2 × 10 ^{−10}			
		S	0.020	3.0 × 10 ^{−9}	0.010	2.2 × 10 ^{−9}	1.3 × 10 ^{−9}	8.7 × 10 ^{−10}	7.3 × 10 ^{−10}	6.0 × 10 ^{−10}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)		
Ir-190	12.1 d	F	0.020	6.2 × 10 ^{−9}	0.010	4.7 × 10 ^{−9}	2.4 × 10 ^{−9}	1.5 × 10 ^{−9}	9.1 × 10 ^{−10}	7.7 × 10 ^{−10}				
		M	0.020	1.1 × 10 ^{−8}	0.010	8.6 × 10 ^{−9}	4.4 × 10 ^{−9}	3.1 × 10 ^{−9}	2.7 × 10 ^{−9}	2.1 × 10 ^{−9}				
		S	0.020	1.1 × 10 ^{−8}	0.010	9.4 × 10 ^{−9}	4.8 × 10 ^{−9}	3.5 × 10 ^{−9}	3.0 × 10 ^{−9}	2.4 × 10 ^{−9}				
Ir-190m	3.10 h	F	0.020	4.2 × 10 ^{−10}	0.010	3.4 × 10 ^{−10}	1.7 × 10 ^{−10}	1.0 × 10 ^{−10}	6.0 × 10 ^{−11}	4.9 × 10 ^{−11}				
		M	0.020	6.0 × 10 ^{−10}	0.010	4.7 × 10 ^{−10}	2.4 × 10 ^{−10}	1.5 × 10 ^{−10}	9.9 × 10 ^{−11}	7.9 × 10 ^{−11}				
		S	0.020	6.2 × 10 ^{−10}	0.010	4.8 × 10 ^{−10}	2.5 × 10 ^{−10}	1.6 × 10 ^{−10}	1.0 × 10 ^{−10}	8.3 × 10 ^{−11}				
Ir-190m′	1.20 h	F	0.020	3.2 × 10 ^{−11}	0.010	2.4 × 10 ^{−11}	1.2 × 10 ^{−11}	7.2 × 10 ^{−12}	4.3 × 10 ^{−12}	3.6 × 10 ^{−12}				
		M	0.020	5.7 × 10 ^{−11}	0.010	4.2 × 10 ^{−11}	2.0 × 10 ^{−11}	1.4 × 10 ^{−11}	1.2 × 10 ^{−11}	9.3 × 10 ^{−12}				
		S	0.020	5.5 × 10 ^{−11}	0.010	4.5 × 10 ^{−11}	2.2 × 10 ^{−11}	1.6 × 10 ^{−11}	1.3 × 10 ^{−11}	1.0 × 10 ^{−11}				
Ir-192	74.0 d	F	0.020	1.5 × 10 ^{−8}	0.010	1.1 × 10 ^{−8}	5.7 × 10 ^{−9}	3.3 × 10 ^{−9}	2.1 × 10 ^{−9}	1.8 × 10 ^{−9}				
		M	0.020	2.3 × 10 ^{−8}	0.010	1.8 × 10 ^{−8}	1.1 × 10 ^{−8}	7.6 × 10 ^{−9}	6.4 × 10 ^{−9}	5.2 × 10 ^{−9}				
		S	0.020	2.8 × 10 ^{−8}	0.010	2.2 × 10 ^{−8}	1.3 × 10 ^{−8}	9.5 × 10 ^{−9}	8.1 × 10 ^{−9}	6.6 × 10 ^{−9}				
Ir-192m	2.41 × 10 ² a	F	0.020	2.7 × 10 ^{−8}	0.010	2.3 × 10 ^{−8}	1.4 × 10 ^{−8}	8.2 × 10 ^{−9}	5.4 × 10 ^{−9}	4.8 × 10 ^{−9}				
		M	0.020	2.3 × 10 ^{−8}	0.010	2.1 × 10 ^{−8}	1.3 × 10 ^{−8}	8.4 × 10 ^{−9}	6.6 × 10 ^{−9}	5.8 × 10 ^{−9}				
		S	0.020	9.2 × 10 ^{−8}	0.010	9.1 × 10 ^{−8}	6.5 × 10 ^{−8}	4.5 × 10 ^{−8}	4.0 × 10 ^{−8}	3.9 × 10 ^{−8}				
Ir-193m	11.9 d	F	0.020	1.2 × 10 ^{−9}	0.010	8.4 × 10 ^{−10}	3.7 × 10 ^{−10}	2.2 × 10 ^{−10}	1.2 × 10 ^{−10}	1.0 × 10 ^{−10}				
		M	0.020	4.8 × 10 ^{−9}	0.010	3.5 × 10 ^{−9}	2.1 × 10 ^{−9}	1.5 × 10 ^{−9}	1.4 × 10 ^{−9}	1.1 × 10 ^{−9}				
		S	0.020	5.4 × 10 ^{−9}	0.010	4.0 × 10 ^{−9}	2.4 × 10 ^{−9}	1.8 × 10 ^{−9}	1.6 × 10 ^{−9}	1.3 × 10 ^{−9}				
Ir-194	19.1 h	F	0.020	2.9 × 10 ^{−9}	0.010	1.9 × 10 ^{−9}	8.1 × 10 ^{−10}	4.9 × 10 ^{−10}	2.5 × 10 ^{−10}	2.1 × 10 ^{−10}				
		M	0.020	5.3 × 10 ^{−9}	0.010	3.5 × 10 ^{−9}	1.6 × 10 ^{−9}	1.0 × 10 ^{−9}	6.3 × 10 ^{−10}	5.2 × 10 ^{−10}				
		S	0.020	5.5 × 10 ^{−9}	0.010	3.7 × 10 ^{−9}	1.7 × 10 ^{−9}	1.1 × 10 ^{−9}	6.7 × 10 ^{−10}	5.6 × 10 ^{−10}				

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		> 17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)			
Ir-194m	171 d	F	0.020	3.4×10^{-8}	0.010	2.7×10^{-8}	1.4×10^{-8}	9.5×10^{-9}	6.2×10^{-9}	5.4×10^{-9}			
		M	0.020	3.9×10^{-8}	0.010	3.2×10^{-8}	1.9×10^{-8}	1.3×10^{-8}	1.1×10^{-8}	9.0×10^{-9}			
		S	0.020	5.0×10^{-8}	0.010	4.2×10^{-8}	2.6×10^{-8}	1.8×10^{-8}	1.5×10^{-8}	1.3×10^{-8}			
Ir-195	2.50 h	F	0.020	2.9×10^{-10}	0.010	1.9×10^{-10}	8.1×10^{-11}	5.1×10^{-11}	2.9×10^{-11}	2.4×10^{-11}			
		M	0.020	5.4×10^{-10}	0.010	3.6×10^{-10}	1.7×10^{-10}	1.1×10^{-10}	8.1×10^{-11}	6.7×10^{-11}			
		S	0.020	5.7×10^{-10}	0.010	3.8×10^{-10}	1.8×10^{-10}	1.2×10^{-10}	8.7×10^{-11}	7.1×10^{-11}			
Ir-195m	3.80 h	F	0.020	6.9×10^{-10}	0.010	4.8×10^{-10}	2.1×10^{-10}	1.3×10^{-10}	7.2×10^{-11}	6.0×10^{-11}			
		M	0.020	1.2×10^{-9}	0.010	8.6×10^{-10}	4.2×10^{-10}	2.7×10^{-10}	1.9×10^{-10}	1.6×10^{-10}			
		S	0.020	1.3×10^{-9}	0.010	9.0×10^{-10}	4.4×10^{-10}	2.9×10^{-10}	2.0×10^{-10}	1.7×10^{-10}			
Platinum													
Pt-186	2.00 h	F	0.020	3.0×10^{-10}	0.010	2.4×10^{-10}	1.2×10^{-10}	7.2×10^{-11}	4.1×10^{-11}	3.3×10^{-11}			
Pt-188	10.2 d	F	0.020	3.6×10^{-9}	0.010	2.7×10^{-9}	1.3×10^{-9}	8.4×10^{-10}	5.0×10^{-10}	4.2×10^{-10}			
Pt-189	10.9 h	F	0.020	3.8×10^{-10}	0.010	2.9×10^{-10}	1.4×10^{-10}	8.4×10^{-11}	4.7×10^{-11}	3.8×10^{-11}			
Pt-191	2.80 d	F	0.020	1.1×10^{-9}	0.010	7.9×10^{-10}	3.7×10^{-10}	2.3×10^{-10}	1.3×10^{-10}	1.1×10^{-10}			
Pt-193	50.0 a	F	0.020	2.2×10^{-10}	0.010	1.6×10^{-10}	7.2×10^{-11}	4.3×10^{-11}	2.5×10^{-11}	2.1×10^{-11}			
Pt-193m	4.33 d	F	0.020	1.6×10^{-9}	0.010	1.0×10^{-9}	4.5×10^{-10}	2.7×10^{-10}	1.4×10^{-10}	1.2×10^{-10}			
Pt-195m	4.02 d	F	0.020	2.2×10^{-9}	0.010	1.5×10^{-9}	6.4×10^{-10}	3.9×10^{-10}	2.1×10^{-10}	1.8×10^{-10}			
Pt-197	18.3 h	F	0.020	1.1×10^{-9}	0.010	7.3×10^{-10}	3.1×10^{-10}	1.9×10^{-10}	1.0×10^{-10}	8.5×10^{-11}			
Pt-197m	1.57 h	F	0.020	2.8×10^{-10}	0.010	1.8×10^{-10}	7.9×10^{-11}	4.9×10^{-11}	2.8×10^{-11}	2.4×10^{-11}			
Pt-199	0.513 h	F	0.020	1.3×10^{-10}	0.010	8.3×10^{-11}	3.6×10^{-11}	2.3×10^{-11}	1.4×10^{-11}	1.2×10^{-11}			
Pt-200	12.5 h	F	0.020	2.6×10^{-9}	0.010	1.7×10^{-9}	7.2×10^{-10}	5.1×10^{-10}	2.6×10^{-10}	2.2×10^{-10}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Gold													
Au-193	17.6 h	F	0.200	3.7×10^{-10}	0.100	2.8×10^{-10}	1.3×10^{-10}	7.9×10^{-11}	4.3×10^{-11}			3.6×10^{-11}	
		M	0.200	7.5×10^{-10}	0.100	5.6×10^{-10}	2.8×10^{-10}	1.9×10^{-10}	1.4×10^{-10}			1.1×10^{-10}	
		S	0.200	7.9×10^{-10}	0.100	5.9×10^{-10}	3.0×10^{-10}	2.0×10^{-10}	1.5×10^{-10}			1.2×10^{-10}	
Au-194	1.65 d	F	0.200	1.2×10^{-9}	0.100	9.6×10^{-10}	4.9×10^{-10}	3.0×10^{-10}	1.8×10^{-10}			1.4×10^{-10}	
		M	0.200	1.7×10^{-9}	0.100	1.4×10^{-9}	7.1×10^{-10}	4.6×10^{-10}	2.9×10^{-10}			2.3×10^{-10}	
		S	0.200	1.7×10^{-9}	0.100	1.4×10^{-9}	7.3×10^{-10}	4.7×10^{-10}	3.0×10^{-10}			2.4×10^{-10}	
Au-195	183 d	F	0.200	7.2×10^{-10}	0.100	5.3×10^{-10}	2.5×10^{-10}	1.5×10^{-10}	8.1×10^{-11}			6.6×10^{-11}	
		M	0.200	5.2×10^{-9}	0.100	4.1×10^{-9}	2.4×10^{-9}	1.6×10^{-9}	1.4×10^{-9}			1.1×10^{-9}	
		S	0.200	8.1×10^{-9}	0.100	6.6×10^{-9}	3.9×10^{-9}	2.6×10^{-9}	2.1×10^{-9}			1.7×10^{-9}	
Au-198	2.69 d	F	0.200	2.4×10^{-9}	0.100	1.7×10^{-9}	7.6×10^{-10}	4.7×10^{-10}	2.5×10^{-10}			2.1×10^{-10}	
		M	0.200	5.0×10^{-9}	0.100	4.1×10^{-9}	1.9×10^{-9}	1.3×10^{-9}	9.7×10^{-10}			7.8×10^{-10}	
		S	0.200	5.4×10^{-9}	0.100	4.4×10^{-9}	2.0×10^{-9}	1.4×10^{-9}	1.1×10^{-9}			8.6×10^{-10}	
Au-198m	2.30 d	F	0.200	3.3×10^{-9}	0.100	2.4×10^{-9}	1.1×10^{-9}	6.9×10^{-10}	3.7×10^{-10}			3.2×10^{-10}	
		M	0.200	8.7×10^{-9}	0.100	6.5×10^{-9}	3.6×10^{-9}	2.6×10^{-9}	2.2×10^{-9}			1.8×10^{-9}	
		S	0.200	9.5×10^{-9}	0.100	7.1×10^{-9}	4.0×10^{-9}	2.9×10^{-9}	2.5×10^{-9}			2.0×10^{-9}	
Au-199	3.14 d	F	0.200	1.1×10^{-9}	0.100	7.9×10^{-10}	3.5×10^{-10}	2.2×10^{-10}	1.1×10^{-10}			9.8×10^{-11}	
		M	0.200	3.4×10^{-9}	0.100	2.5×10^{-9}	1.4×10^{-9}	1.0×10^{-9}	9.0×10^{-10}			7.1×10^{-10}	
		S	0.200	3.8×10^{-9}	0.100	2.8×10^{-9}	1.6×10^{-9}	1.2×10^{-9}	1.0×10^{-9}			7.9×10^{-10}	
Au-200	0.807 h	F	0.200	1.9×10^{-10}	0.100	1.2×10^{-10}	5.2×10^{-11}	3.2×10^{-11}	1.9×10^{-11}			1.6×10^{-11}	
		M	0.200	3.2×10^{-10}	0.100	2.1×10^{-10}	9.3×10^{-11}	6.0×10^{-11}	4.0×10^{-11}			3.3×10^{-11}	
		S	0.200	3.4×10^{-10}	0.100	2.1×10^{-10}	9.8×10^{-11}	6.3×10^{-11}	4.2×10^{-11}			3.5×10^{-11}	

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)			
Au-200m	18.7 h	F	0.200	2.7 × 10 ^{−9}	0.100	2.1 × 10 ^{−9}	1.0 × 10 ^{−9}	6.4 × 10 ^{−10}	3.6 × 10 ^{−10}	2.9 × 10 ^{−10}			
		M	0.200	4.8 × 10 ^{−9}	0.100	3.7 × 10 ^{−9}	1.9 × 10 ^{−9}	1.2 × 10 ^{−9}	8.4 × 10 ^{−10}	6.8 × 10 ^{−10}			
		S	0.200	5.1 × 10 ^{−9}	0.100	3.9 × 10 ^{−9}	2.0 × 10 ^{−9}	1.3 × 10 ^{−9}	8.9 × 10 ^{−10}	7.2 × 10 ^{−10}			
Au-201	0.440 h	F	0.200	9.0 × 10 ^{−11}	0.100	5.7 × 10 ^{−11}	2.5 × 10 ^{−11}	1.6 × 10 ^{−11}	1.0 × 10 ^{−11}	8.7 × 10 ^{−12}			
		M	0.200	1.5 × 10 ^{−10}	0.100	9.6 × 10 ^{−11}	4.3 × 10 ^{−11}	2.9 × 10 ^{−11}	2.0 × 10 ^{−11}	1.7 × 10 ^{−11}			
		S	0.200	1.5 × 10 ^{−10}	0.100	1.0 × 10 ^{−10}	4.5 × 10 ^{−11}	3.0 × 10 ^{−11}	2.1 × 10 ^{−11}	1.7 × 10 ^{−11}			
Mercury													
Hg-193 (organic)	3.50 h	F	0.800	2.2 × 10 ^{−10}	0.400	1.8 × 10 ^{−10}	8.2 × 10 ^{−11}	5.0 × 10 ^{−11}	2.9 × 10 ^{−11}	2.4 × 10 ^{−11}			
Hg-193 (inorganic)	3.50 h	F	0.040	2.7 × 10 ^{−10}	0.020	2.0 × 10 ^{−10}	8.9 × 10 ^{−11}	5.5 × 10 ^{−11}	3.1 × 10 ^{−11}	2.6 × 10 ^{−11}			
		M	0.040	5.3 × 10 ^{−10}	0.020	3.8 × 10 ^{−10}	1.9 × 10 ^{−10}	1.3 × 10 ^{−10}	9.2 × 10 ^{−11}	7.5 × 10 ^{−11}			
Hg-193m (organic)	11.1 h	F	0.800	8.4 × 10 ^{−10}	0.400	7.6 × 10 ^{−10}	3.7 × 10 ^{−10}	2.2 × 10 ^{−10}	1.3 × 10 ^{−10}	1.0 × 10 ^{−10}			
Hg-193m (inorganic)	11.1 h	F	0.040	1.1 × 10 ^{−9}	0.020	8.5 × 10 ^{−10}	4.1 × 10 ^{−10}	2.5 × 10 ^{−10}	1.4 × 10 ^{−10}	1.1 × 10 ^{−10}			
		M	0.040	1.9 × 10 ^{−9}	0.020	1.4 × 10 ^{−9}	7.2 × 10 ^{−10}	4.7 × 10 ^{−10}	3.2 × 10 ^{−10}	2.6 × 10 ^{−10}			
Hg-194 (organic)	2.60 × 10 ² a	F	0.800	4.9 × 10 ^{−8}	0.400	3.7 × 10 ^{−8}	2.4 × 10 ^{−8}	1.9 × 10 ^{−8}	1.5 × 10 ^{−8}	1.4 × 10 ^{−8}			
Hg-194 (inorganic)	2.60 × 10 ² a	F	0.040	3.2 × 10 ^{−8}	0.020	2.9 × 10 ^{−8}	2.0 × 10 ^{−8}	1.6 × 10 ^{−8}	1.4 × 10 ^{−8}	1.3 × 10 ^{−8}			
		M	0.040	2.1 × 10 ^{−8}	0.020	1.9 × 10 ^{−8}	1.3 × 10 ^{−8}	1.0 × 10 ^{−8}	8.9 × 10 ^{−9}	8.3 × 10 ^{−9}			
Hg-195 (organic)	9.90 h	F	0.800	2.0 × 10 ^{−10}	0.400	1.8 × 10 ^{−10}	8.5 × 10 ^{−11}	5.1 × 10 ^{−11}	2.8 × 10 ^{−11}	2.3 × 10 ^{−11}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)			
Hg-195	9.90 h	F	0.040	2.7 × 10 ⁻¹⁰	0.020	2.0 × 10 ⁻¹⁰	9.5 × 10 ⁻¹¹	5.7 × 10 ⁻¹¹	3.1 × 10 ⁻¹¹	3.1 × 10 ⁻¹¹	2.5 × 10 ⁻¹¹		
(inorganic)		M	0.040	5.3 × 10 ⁻¹⁰	0.020	3.9 × 10 ⁻¹⁰	2.0 × 10 ⁻¹⁰	1.3 × 10 ⁻¹⁰	9.0 × 10 ⁻¹¹	9.0 × 10 ⁻¹¹	7.3 × 10 ⁻¹¹		
Hg-195m	1.73 d	F	0.800	1.1 × 10 ⁻⁹	0.400	9.7 × 10 ⁻¹⁰	4.4 × 10 ⁻¹⁰	2.7 × 10 ⁻¹⁰	1.4 × 10 ⁻¹⁰	1.4 × 10 ⁻¹⁰	1.2 × 10 ⁻¹⁰		
(organic)													
Hg-195m	1.73 d	F	0.040	1.6 × 10 ⁻⁹	0.020	1.1 × 10 ⁻⁹	5.1 × 10 ⁻¹⁰	3.1 × 10 ⁻¹⁰	1.7 × 10 ⁻¹⁰	1.7 × 10 ⁻¹⁰	1.4 × 10 ⁻¹⁰		
(inorganic)		M	0.040	3.7 × 10 ⁻⁹	0.020	2.6 × 10 ⁻⁹	1.4 × 10 ⁻⁹	8.5 × 10 ⁻¹⁰	6.7 × 10 ⁻¹⁰	6.7 × 10 ⁻¹⁰	5.3 × 10 ⁻¹⁰		
Hg-197	2.67 d	F	0.800	4.7 × 10 ⁻¹⁰	0.400	4.0 × 10 ⁻¹⁰	1.8 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰	5.8 × 10 ⁻¹¹	5.8 × 10 ⁻¹¹	4.7 × 10 ⁻¹¹		
(organic)													
Hg-197	2.67 d	F	0.040	6.8 × 10 ⁻¹⁰	0.020	4.7 × 10 ⁻¹⁰	2.1 × 10 ⁻¹⁰	1.3 × 10 ⁻¹⁰	6.8 × 10 ⁻¹¹	6.8 × 10 ⁻¹¹	5.6 × 10 ⁻¹¹		
(inorganic)		M	0.040	1.7 × 10 ⁻⁹	0.020	1.2 × 10 ⁻⁹	6.6 × 10 ⁻¹⁰	4.6 × 10 ⁻¹⁰	3.8 × 10 ⁻¹⁰	3.8 × 10 ⁻¹⁰	3.0 × 10 ⁻¹⁰		
Hg-197m	23.8 h	F	0.800	9.3 × 10 ⁻¹⁰	0.400	7.8 × 10 ⁻¹⁰	3.4 × 10 ⁻¹⁰	2.1 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰	9.6 × 10 ⁻¹¹		
(organic)													
Hg-197m	23.8 h	F	0.040	1.4 × 10 ⁻⁹	0.020	9.3 × 10 ⁻¹⁰	4.0 × 10 ⁻¹⁰	2.5 × 10 ⁻¹⁰	1.3 × 10 ⁻¹⁰	1.3 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰		
(inorganic)		M	0.040	3.5 × 10 ⁻⁹	0.020	2.5 × 10 ⁻⁹	1.1 × 10 ⁻⁹	8.2 × 10 ⁻¹⁰	6.7 × 10 ⁻¹⁰	6.7 × 10 ⁻¹⁰	5.3 × 10 ⁻¹⁰		
Hg-199m	0.710 h	F	0.800	1.4 × 10 ⁻¹⁰	0.400	9.6 × 10 ⁻¹¹	4.2 × 10 ⁻¹¹	2.7 × 10 ⁻¹¹	1.7 × 10 ⁻¹¹	1.7 × 10 ⁻¹¹	1.5 × 10 ⁻¹¹		
(organic)													
Hg-199m	0.710 h	F	0.040	1.4 × 10 ⁻¹⁰	0.020	9.6 × 10 ⁻¹¹	4.2 × 10 ⁻¹¹	2.7 × 10 ⁻¹¹	1.7 × 10 ⁻¹¹	1.7 × 10 ⁻¹¹	1.5 × 10 ⁻¹¹		
(inorganic)		M	0.040	2.5 × 10 ⁻¹⁰	0.020	1.7 × 10 ⁻¹⁰	7.9 × 10 ⁻¹¹	5.4 × 10 ⁻¹¹	3.8 × 10 ⁻¹¹	3.8 × 10 ⁻¹¹	3.2 × 10 ⁻¹¹		
Hg-203	46.6 d	F	0.800	5.7 × 10 ⁻⁹	0.400	3.7 × 10 ⁻⁹	1.7 × 10 ⁻⁹	1.1 × 10 ⁻⁹	6.6 × 10 ⁻¹⁰	6.6 × 10 ⁻¹⁰	5.6 × 10 ⁻¹⁰		
(organic)													
Hg-203	46.6 d	F	0.040	4.2 × 10 ⁻⁹	0.020	2.9 × 10 ⁻⁹	1.4 × 10 ⁻⁹	9.0 × 10 ⁻¹⁰	5.5 × 10 ⁻¹⁰	5.5 × 10 ⁻¹⁰	4.6 × 10 ⁻¹⁰		
(inorganic)		M	0.040	1.0 × 10 ⁻⁸	0.020	7.9 × 10 ⁻⁹	4.7 × 10 ⁻⁹	3.4 × 10 ⁻⁹	3.0 × 10 ⁻⁹	3.0 × 10 ⁻⁹	2.4 × 10 ⁻⁹		

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		$e(g)$
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$				
Thallium													
Tl-194	0.550 h	F	1.000	3.6×10^{-11}	1.000	3.0×10^{-11}	1.5×10^{-11}	9.2×10^{-12}	5.5×10^{-12}	4.4×10^{-12}			
Tl-194m	0.546 h	F	1.000	1.7×10^{-10}	1.000	1.2×10^{-10}	6.1×10^{-11}	3.8×10^{-11}	2.3×10^{-11}	1.9×10^{-11}			
Tl-195	1.16 h	F	1.000	1.3×10^{-10}	1.000	1.0×10^{-10}	5.3×10^{-11}	3.2×10^{-11}	1.9×10^{-11}	1.5×10^{-11}			
Tl-197	2.84 h	F	1.000	1.3×10^{-10}	1.000	9.7×10^{-11}	4.7×10^{-11}	2.9×10^{-11}	1.7×10^{-11}	1.4×10^{-11}			
Tl-198	5.30 h	F	1.000	4.7×10^{-10}	1.000	4.0×10^{-10}	2.1×10^{-10}	1.3×10^{-10}	7.5×10^{-11}	6.0×10^{-11}			
Tl-198m	1.87 h	F	1.000	3.2×10^{-10}	1.000	2.5×10^{-10}	1.2×10^{-10}	7.5×10^{-11}	4.5×10^{-11}	3.7×10^{-11}			
Tl-199	7.42 h	F	1.000	1.7×10^{-10}	1.000	1.3×10^{-10}	6.4×10^{-11}	3.9×10^{-11}	2.3×10^{-11}	1.9×10^{-11}			
Tl-200	1.09 d	F	1.000	1.0×10^{-9}	1.000	8.7×10^{-10}	4.6×10^{-10}	2.8×10^{-10}	1.6×10^{-10}	1.3×10^{-10}			
Tl-201	3.04 d	F	1.000	4.5×10^{-10}	1.000	3.3×10^{-10}	1.5×10^{-10}	9.4×10^{-11}	5.4×10^{-11}	4.4×10^{-11}			
Tl-202	12.2 d	F	1.000	1.5×10^{-9}	1.000	1.2×10^{-9}	5.9×10^{-10}	3.8×10^{-10}	2.3×10^{-10}	1.9×10^{-10}			
Tl-204	3.78 a	F	1.000	5.0×10^{-9}	1.000	3.3×10^{-9}	1.5×10^{-9}	8.8×10^{-10}	4.7×10^{-10}	3.9×10^{-10}			
Lead ^g													
Pb-195m	0.263 h	F	0.600	1.3×10^{-10}	0.200	1.0×10^{-10}	4.9×10^{-11}	3.1×10^{-11}	1.9×10^{-11}	1.6×10^{-11}			
		M	0.200	2.0×10^{-10}	0.100	1.5×10^{-10}	7.1×10^{-11}	4.6×10^{-11}	3.1×10^{-11}	2.5×10^{-11}			
		S	0.020	2.1×10^{-10}	0.010	1.5×10^{-10}	7.4×10^{-11}	4.8×10^{-11}	3.2×10^{-11}	2.7×10^{-11}			
Pb-198	2.40 h	F	0.600	3.4×10^{-10}	0.200	2.9×10^{-10}	1.5×10^{-10}	8.9×10^{-11}	5.2×10^{-11}	4.3×10^{-11}			
		M	0.200	5.0×10^{-10}	0.100	4.0×10^{-10}	2.1×10^{-10}	1.3×10^{-10}	8.3×10^{-11}	6.6×10^{-11}			
		S	0.020	5.4×10^{-10}	0.010	4.2×10^{-10}	2.2×10^{-10}	1.4×10^{-10}	8.7×10^{-11}	7.0×10^{-11}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$
Pb-199	1.50 h	F	0.600	1.9×10^{-10}	0.200	1.6×10^{-10}	8.2×10^{-11}	4.9×10^{-11}	2.9×10^{-11}	2.3×10^{-11}								
		M	0.200	2.8×10^{-10}	0.100	2.2×10^{-10}	1.1×10^{-10}	7.1×10^{-11}	4.5×10^{-11}	3.6×10^{-11}								
		S	0.020	2.9×10^{-10}	0.010	2.3×10^{-10}	1.2×10^{-10}	7.4×10^{-11}	4.7×10^{-11}	3.7×10^{-11}								
Pb-200	21.5 h	F	0.600	1.1×10^{-9}	0.200	9.3×10^{-10}	4.6×10^{-10}	2.8×10^{-10}	1.6×10^{-10}	1.4×10^{-10}								
		M	0.200	2.2×10^{-9}	0.100	1.7×10^{-9}	8.6×10^{-10}	5.7×10^{-10}	4.1×10^{-10}	3.3×10^{-10}								
		S	0.020	2.4×10^{-9}	0.010	1.8×10^{-9}	9.2×10^{-10}	6.2×10^{-10}	4.4×10^{-10}	3.5×10^{-10}								
Pb-201	9.40 h	F	0.600	4.8×10^{-10}	0.200	4.1×10^{-10}	2.0×10^{-10}	1.2×10^{-10}	7.1×10^{-11}	6.0×10^{-11}								
		M	0.200	8.0×10^{-10}	0.100	6.4×10^{-10}	3.3×10^{-10}	2.1×10^{-10}	1.4×10^{-10}	1.1×10^{-10}								
		S	0.020	8.8×10^{-10}	0.010	6.7×10^{-10}	3.5×10^{-10}	2.2×10^{-10}	1.5×10^{-10}	1.2×10^{-10}								
Pb-202	3.00×10^5 a	F	0.600	1.9×10^{-8}	0.200	1.3×10^{-8}	8.9×10^{-9}	1.3×10^{-8}	1.8×10^{-8}	1.1×10^{-8}								
		M	0.200	1.2×10^{-8}	0.100	8.9×10^{-9}	6.2×10^{-9}	6.7×10^{-9}	8.7×10^{-9}	6.3×10^{-9}								
		S	0.020	2.8×10^{-8}	0.010	2.8×10^{-8}	2.0×10^{-8}	1.4×10^{-8}	1.3×10^{-8}	1.2×10^{-8}								
Pb-202m	3.62 h	F	0.600	4.7×10^{-10}	0.200	4.0×10^{-10}	2.1×10^{-10}	1.3×10^{-10}	7.5×10^{-11}	6.2×10^{-11}								
		M	0.200	6.9×10^{-10}	0.100	5.6×10^{-10}	2.9×10^{-10}	1.9×10^{-10}	1.2×10^{-10}	9.5×10^{-11}								
		S	0.020	7.3×10^{-10}	0.010	5.8×10^{-10}	3.0×10^{-10}	1.9×10^{-10}	1.3×10^{-10}	1.0×10^{-10}								
Pb-203	2.17 d	F	0.600	7.2×10^{-10}	0.200	5.8×10^{-10}	2.8×10^{-10}	1.7×10^{-10}	9.9×10^{-11}	8.5×10^{-11}								
		M	0.200	1.3×10^{-9}	0.100	1.0×10^{-9}	5.4×10^{-10}	3.6×10^{-10}	2.5×10^{-10}	2.0×10^{-10}								
		S	0.020	1.5×10^{-9}	0.010	1.1×10^{-9}	5.8×10^{-10}	3.8×10^{-10}	2.8×10^{-10}	2.2×10^{-10}								
Pb-205	1.43×10^7 a	F	0.600	1.1×10^{-9}	0.200	6.9×10^{-10}	4.0×10^{-10}	4.1×10^{-10}	4.3×10^{-10}	3.3×10^{-10}								
		M	0.200	1.1×10^{-9}	0.100	7.7×10^{-10}	4.3×10^{-10}	3.2×10^{-10}	2.9×10^{-10}	2.5×10^{-10}								
		S	0.020	2.9×10^{-9}	0.010	2.7×10^{-9}	1.7×10^{-9}	1.1×10^{-9}	9.2×10^{-10}	8.5×10^{-10}								

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)			
Pb-209	3.25 h	F	0.600	1.8 × 10 ^{−10}	0.200	1.2 × 10 ^{−10}	5.3 × 10 ^{−11}	3.4 × 10 ^{−11}	1.9 × 10 ^{−11}	1.7 × 10 ^{−11}			
		M	0.200	4.0 × 10 ^{−10}	0.100	2.7 × 10 ^{−10}	1.3 × 10 ^{−10}	9.2 × 10 ^{−11}	6.9 × 10 ^{−11}	5.6 × 10 ^{−11}			
		S	0.020	4.4 × 10 ^{−10}	0.010	2.9 × 10 ^{−10}	1.4 × 10 ^{−10}	9.9 × 10 ^{−11}	7.5 × 10 ^{−11}	6.1 × 10 ^{−11}			
Pb-210	22.3 a	F	0.600	4.7 × 10 ^{−6}	0.200	2.9 × 10 ^{−6}	1.5 × 10 ^{−6}	1.4 × 10 ^{−6}	1.3 × 10 ^{−6}	9.0 × 10 ^{−7}			
		M	0.200	5.0 × 10 ^{−6}	0.100	3.7 × 10 ^{−6}	2.2 × 10 ^{−6}	1.5 × 10 ^{−6}	1.3 × 10 ^{−6}	1.1 × 10 ^{−6}			
		S	0.020	1.8 × 10 ^{−5}	0.010	1.8 × 10 ^{−5}	1.1 × 10 ^{−5}	7.2 × 10 ^{−6}	5.9 × 10 ^{−6}	5.6 × 10 ^{−6}			
Pb-211	0.601 h	F	0.600	2.5 × 10 ^{−8}	0.200	1.7 × 10 ^{−8}	8.7 × 10 ^{−9}	6.1 × 10 ^{−9}	4.6 × 10 ^{−9}	3.9 × 10 ^{−9}			
		M	0.200	6.2 × 10 ^{−8}	0.100	4.5 × 10 ^{−8}	2.5 × 10 ^{−8}	1.9 × 10 ^{−8}	1.4 × 10 ^{−8}	1.1 × 10 ^{−8}			
		S	0.020	6.6 × 10 ^{−8}	0.010	4.8 × 10 ^{−8}	2.7 × 10 ^{−8}	2.0 × 10 ^{−8}	1.5 × 10 ^{−8}	1.2 × 10 ^{−8}			
Pb-212	10.6 h	F	0.600	1.9 × 10 ^{−7}	0.200	1.2 × 10 ^{−7}	5.4 × 10 ^{−8}	3.5 × 10 ^{−8}	2.0 × 10 ^{−8}	1.8 × 10 ^{−8}			
		M	0.200	6.2 × 10 ^{−7}	0.100	4.6 × 10 ^{−7}	3.0 × 10 ^{−7}	2.2 × 10 ^{−7}	2.2 × 10 ^{−7}	1.7 × 10 ^{−7}			
		S	0.020	6.7 × 10 ^{−7}	0.010	5.0 × 10 ^{−7}	3.3 × 10 ^{−7}	2.5 × 10 ^{−7}	2.4 × 10 ^{−7}	1.9 × 10 ^{−7}			
Pb-214	0.447 h	F	0.600	2.2 × 10 ^{−8}	0.200	1.5 × 10 ^{−8}	6.9 × 10 ^{−9}	4.8 × 10 ^{−9}	3.3 × 10 ^{−9}	2.8 × 10 ^{−9}			
		M	0.200	6.4 × 10 ^{−8}	0.100	4.6 × 10 ^{−8}	2.6 × 10 ^{−8}	1.9 × 10 ^{−8}	1.4 × 10 ^{−8}	1.4 × 10 ^{−8}			
		S	0.020	6.9 × 10 ^{−8}	0.010	5.0 × 10 ^{−8}	2.8 × 10 ^{−8}	2.1 × 10 ^{−8}	1.5 × 10 ^{−8}	1.5 × 10 ^{−8}			
Bismuth													
Bi-200	0.606 h	F	0.100	1.9 × 10 ^{−10}	0.050	1.5 × 10 ^{−10}	7.4 × 10 ^{−11}	4.5 × 10 ^{−11}	2.7 × 10 ^{−11}	2.2 × 10 ^{−11}			
		M	0.100	2.5 × 10 ^{−10}	0.050	1.9 × 10 ^{−10}	9.9 × 10 ^{−11}	6.3 × 10 ^{−11}	4.1 × 10 ^{−11}	3.3 × 10 ^{−11}			
Bi-201	1.80 h	F	0.100	4.0 × 10 ^{−10}	0.050	3.1 × 10 ^{−10}	1.5 × 10 ^{−10}	9.3 × 10 ^{−11}	5.4 × 10 ^{−11}	4.4 × 10 ^{−11}			
		M	0.100	5.5 × 10 ^{−10}	0.050	4.1 × 10 ^{−10}	2.0 × 10 ^{−10}	1.3 × 10 ^{−10}	8.3 × 10 ^{−11}	6.6 × 10 ^{−11}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a						2–7 a		7–12 a		12–17 a		>17 a	
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	
Bi-202	1.67 h	F	0.100	3.4 × 10 ^{−10}	0.050	2.8 × 10 ^{−10}	1.5 × 10 ^{−10}	9.0 × 10 ^{−11}	5.3 × 10 ^{−11}	4.3 × 10 ^{−11}								
		M	0.100	4.2 × 10 ^{−10}	0.050	3.4 × 10 ^{−10}	1.8 × 10 ^{−10}	1.1 × 10 ^{−10}	6.9 × 10 ^{−11}	5.5 × 10 ^{−11}								
Bi-203	11.8 h	F	0.100	1.5 × 10 ^{−9}	0.050	1.2 × 10 ^{−9}	6.4 × 10 ^{−10}	4.0 × 10 ^{−10}	2.3 × 10 ^{−10}	1.9 × 10 ^{−10}								
		M	0.100	2.0 × 10 ^{−9}	0.050	1.6 × 10 ^{−9}	8.2 × 10 ^{−10}	5.3 × 10 ^{−10}	3.3 × 10 ^{−10}	2.6 × 10 ^{−10}								
Bi-205	15.3 d	F	0.100	3.0 × 10 ^{−9}	0.050	2.4 × 10 ^{−9}	1.3 × 10 ^{−9}	8.0 × 10 ^{−10}	4.7 × 10 ^{−10}	3.8 × 10 ^{−10}								
		M	0.100	5.5 × 10 ^{−9}	0.050	4.4 × 10 ^{−9}	2.5 × 10 ^{−9}	1.6 × 10 ^{−9}	1.2 × 10 ^{−9}	9.3 × 10 ^{−10}								
Bi-206	6.24 d	F	0.100	6.1 × 10 ^{−9}	0.050	4.8 × 10 ^{−9}	2.5 × 10 ^{−9}	1.6 × 10 ^{−9}	9.1 × 10 ^{−10}	7.4 × 10 ^{−10}								
		M	0.100	1.0 × 10 ^{−8}	0.050	8.0 × 10 ^{−9}	4.4 × 10 ^{−9}	2.9 × 10 ^{−9}	2.1 × 10 ^{−9}	1.7 × 10 ^{−9}								
Bi-207	38.0 a	F	0.100	4.3 × 10 ^{−9}	0.050	3.3 × 10 ^{−9}	1.7 × 10 ^{−9}	1.0 × 10 ^{−9}	6.0 × 10 ^{−10}	4.9 × 10 ^{−10}								
		M	0.100	2.3 × 10 ^{−8}	0.050	2.0 × 10 ^{−8}	1.2 × 10 ^{−8}	8.2 × 10 ^{−9}	6.5 × 10 ^{−9}	5.6 × 10 ^{−9}								
Bi-210	5.01 d	F	0.100	1.1 × 10 ^{−8}	0.050	6.9 × 10 ^{−9}	3.2 × 10 ^{−9}	2.1 × 10 ^{−9}	1.3 × 10 ^{−9}	1.1 × 10 ^{−9}								
		M	0.100	3.9 × 10 ^{−7}	0.050	3.0 × 10 ^{−7}	1.9 × 10 ^{−7}	1.3 × 10 ^{−7}	1.1 × 10 ^{−7}	9.3 × 10 ^{−8}								
Bi-210m	3.00 × 10 ⁶ a	F	0.100	4.1 × 10 ^{−7}	0.050	2.6 × 10 ^{−7}	1.3 × 10 ^{−7}	8.3 × 10 ^{−8}	5.6 × 10 ^{−8}	4.6 × 10 ^{−8}								
		M	0.100	1.5 × 10 ^{−5}	0.050	1.1 × 10 ^{−5}	7.0 × 10 ^{−6}	4.8 × 10 ^{−6}	4.1 × 10 ^{−6}	3.4 × 10 ^{−6}								
Bi-212	1.01 h	F	0.100	6.5 × 10 ^{−8}	0.050	4.5 × 10 ^{−8}	2.1 × 10 ^{−8}	1.5 × 10 ^{−8}	1.0 × 10 ^{−8}	9.1 × 10 ^{−9}								
		M	0.100	1.6 × 10 ^{−7}	0.050	1.1 × 10 ^{−7}	6.0 × 10 ^{−8}	4.4 × 10 ^{−8}	3.8 × 10 ^{−8}	3.1 × 10 ^{−8}								
Bi-213	0.761 h	F	0.100	7.7 × 10 ^{−8}	0.050	5.3 × 10 ^{−8}	2.5 × 10 ^{−8}	1.7 × 10 ^{−8}	1.2 × 10 ^{−8}	1.0 × 10 ^{−8}								
		M	0.100	1.6 × 10 ^{−7}	0.050	1.2 × 10 ^{−7}	6.0 × 10 ^{−8}	4.4 × 10 ^{−8}	3.6 × 10 ^{−8}	3.0 × 10 ^{−8}								
Bi-214	0.332 h	F	0.100	5.0 × 10 ^{−8}	0.050	3.5 × 10 ^{−8}	1.6 × 10 ^{−8}	1.1 × 10 ^{−8}	8.2 × 10 ^{−9}	7.1 × 10 ^{−9}								
		M	0.100	8.7 × 10 ^{−8}	0.050	6.1 × 10 ^{−8}	3.1 × 10 ^{−8}	2.2 × 10 ^{−8}	1.7 × 10 ^{−8}	1.4 × 10 ^{−8}								

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	<i>f</i> ₁		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)				
Polonium														
Po-203	0.612 h	F	0.200	1.9×10^{-10}	0.100	1.5×10^{-10}	7.7×10^{-11}	4.7×10^{-11}	2.8×10^{-11}	2.3×10^{-11}				
		M	0.200	2.7×10^{-10}	0.100	2.1×10^{-10}	1.1×10^{-10}	6.7×10^{-11}	4.3×10^{-11}	3.5×10^{-11}				
		S	0.020	2.8×10^{-10}	0.010	2.2×10^{-10}	1.1×10^{-10}	7.0×10^{-11}	4.5×10^{-11}	3.6×10^{-11}				
Po-205	1.80 h	F	0.200	2.6×10^{-10}	0.100	2.1×10^{-10}	1.1×10^{-10}	6.6×10^{-11}	4.1×10^{-11}	3.3×10^{-11}				
		M	0.200	4.0×10^{-10}	0.100	3.1×10^{-10}	1.7×10^{-10}	1.1×10^{-10}	8.1×10^{-11}	6.5×10^{-11}				
		S	0.020	4.2×10^{-10}	0.010	3.2×10^{-10}	1.8×10^{-10}	1.2×10^{-10}	8.5×10^{-11}	6.9×10^{-11}				
Po-207	5.83 h	F	0.200	4.8×10^{-10}	0.100	4.0×10^{-10}	2.1×10^{-10}	1.3×10^{-10}	7.3×10^{-11}	5.8×10^{-11}				
		M	0.200	6.2×10^{-10}	0.100	5.1×10^{-10}	2.6×10^{-10}	1.6×10^{-10}	9.9×10^{-11}	7.8×10^{-11}				
		S	0.020	6.6×10^{-10}	0.010	5.3×10^{-10}	2.7×10^{-10}	1.7×10^{-10}	1.0×10^{-10}	8.2×10^{-11}				
Po-210	138 d	F	0.200	7.4×10^{-6}	0.100	4.8×10^{-6}	2.2×10^{-6}	1.3×10^{-6}	7.7×10^{-7}	6.1×10^{-7}				
		M	0.200	1.5×10^{-5}	0.100	1.1×10^{-5}	6.7×10^{-6}	4.6×10^{-6}	4.0×10^{-6}	3.3×10^{-6}				
		S	0.020	1.8×10^{-5}	0.010	1.4×10^{-5}	8.6×10^{-6}	5.9×10^{-6}	5.1×10^{-6}	4.3×10^{-6}				
Astatine														
At-207	1.80 h	F	1.000	2.4×10^{-9}	1.000	1.7×10^{-9}	8.9×10^{-10}	5.9×10^{-10}	4.0×10^{-10}	3.3×10^{-10}				
		M	1.000	9.2×10^{-9}	1.000	6.7×10^{-9}	4.3×10^{-9}	3.1×10^{-9}	2.9×10^{-9}	2.3×10^{-9}				
At-211	7.21 h	F	1.000	1.4×10^{-7}	1.000	9.7×10^{-8}	4.3×10^{-8}	2.8×10^{-8}	1.7×10^{-8}	1.6×10^{-8}				
		M	1.000	5.2×10^{-7}	1.000	3.7×10^{-7}	1.9×10^{-7}	1.4×10^{-7}	1.3×10^{-7}	1.1×10^{-7}				

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)			
Francium													
Fr-222	0.240 h	F	1.000	9.1 × 10 ^{−8}	1.000	6.3 × 10 ^{−8}	3.0 × 10 ^{−8}	2.1 × 10 ^{−8}	1.6 × 10 ^{−8}	1.4 × 10 ^{−8}			
Fr-223	0.363 h	F	1.000	1.1 × 10 ^{−8}	1.000	7.3 × 10 ^{−9}	3.2 × 10 ^{−9}	1.9 × 10 ^{−9}	1.0 × 10 ^{−9}	8.9 × 10 ^{−10}			
Radium^h													
Ra-223	11.4 d	F	0.600	3.0 × 10 ^{−6}	0.200	1.0 × 10 ^{−6}	4.9 × 10 ^{−7}	4.0 × 10 ^{−7}	3.3 × 10 ^{−7}	1.2 × 10 ^{−7}			
		M	0.200	2.8 × 10 ^{−5}	0.100	2.1 × 10 ^{−5}	1.3 × 10 ^{−5}	9.9 × 10 ^{−6}	9.4 × 10 ^{−6}	7.4 × 10 ^{−6}			
		S	0.020	3.2 × 10 ^{−5}	0.010	2.4 × 10 ^{−5}	1.5 × 10 ^{−5}	1.1 × 10 ^{−5}	1.1 × 10 ^{−5}	8.7 × 10 ^{−6}			
Ra-224	3.66 d	F	0.600	1.5 × 10 ^{−6}	0.200	6.0 × 10 ^{−7}	2.9 × 10 ^{−7}	2.2 × 10 ^{−7}	1.7 × 10 ^{−7}	7.5 × 10 ^{−8}			
		M	0.200	1.1 × 10 ^{−5}	0.100	8.2 × 10 ^{−6}	5.3 × 10 ^{−6}	3.9 × 10 ^{−6}	3.7 × 10 ^{−6}	3.0 × 10 ^{−6}			
		S	0.020	1.2 × 10 ^{−5}	0.010	9.2 × 10 ^{−6}	5.9 × 10 ^{−6}	4.4 × 10 ^{−6}	4.2 × 10 ^{−6}	3.4 × 10 ^{−6}			
Ra-225	14.8 d	F	0.600	4.0 × 10 ^{−6}	0.200	1.2 × 10 ^{−6}	5.6 × 10 ^{−7}	4.6 × 10 ^{−7}	3.8 × 10 ^{−7}	1.3 × 10 ^{−7}			
		M	0.200	2.4 × 10 ^{−5}	0.100	1.8 × 10 ^{−5}	1.1 × 10 ^{−5}	8.4 × 10 ^{−6}	7.9 × 10 ^{−6}	6.3 × 10 ^{−6}			
		S	0.020	2.8 × 10 ^{−5}	0.010	2.2 × 10 ^{−5}	1.4 × 10 ^{−5}	1.0 × 10 ^{−5}	9.8 × 10 ^{−6}	7.7 × 10 ^{−6}			
Ra-226	1.60 × 10 ³ a	F	0.600	2.6 × 10 ^{−6}	0.200	9.4 × 10 ^{−7}	5.5 × 10 ^{−7}	7.2 × 10 ^{−7}	1.3 × 10 ^{−6}	3.6 × 10 ^{−7}			
		M	0.200	1.5 × 10 ^{−5}	0.100	1.1 × 10 ^{−5}	7.0 × 10 ^{−6}	4.9 × 10 ^{−6}	4.5 × 10 ^{−6}	3.5 × 10 ^{−6}			
		S	0.020	3.4 × 10 ^{−5}	0.010	2.9 × 10 ^{−5}	1.9 × 10 ^{−5}	1.2 × 10 ^{−5}	1.0 × 10 ^{−5}	9.5 × 10 ^{−6}			
Ra-227	0.703 h	F	0.600	1.5 × 10 ^{−9}	0.200	1.2 × 10 ^{−9}	7.8 × 10 ^{−10}	6.1 × 10 ^{−10}	5.3 × 10 ^{−10}	4.6 × 10 ^{−10}			
		M	0.200	8.0 × 10 ^{−10}	0.100	6.7 × 10 ^{−10}	4.4 × 10 ^{−10}	3.2 × 10 ^{−10}	2.9 × 10 ^{−10}	2.8 × 10 ^{−10}			
		S	0.020	1.0 × 10 ^{−9}	0.010	8.5 × 10 ^{−10}	4.4 × 10 ^{−10}	2.9 × 10 ^{−10}	2.4 × 10 ^{−10}	2.2 × 10 ^{−10}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Ra-228	5.75 a	F	0.600	1.7×10^{-5}	0.200	5.7×10^{-6}	3.1×10^{-6}	3.6×10^{-6}	4.6×10^{-6}	4.6×10^{-6}	4.6×10^{-6}	9.0×10^{-7}	
		M	0.200	1.5×10^{-5}	0.100	1.0×10^{-5}	6.3×10^{-6}	4.6×10^{-6}	4.4×10^{-6}	4.4×10^{-6}	2.6×10^{-6}		
		S	0.020	4.9×10^{-5}	0.010	4.8×10^{-5}	3.2×10^{-5}	2.0×10^{-5}	1.6×10^{-5}	1.6×10^{-5}	1.6×10^{-5}		
Actinium													
Ac-224	2.90 h	F	0.005	1.3×10^{-7}	5.0×10^{-4}	8.9×10^{-8}	4.7×10^{-8}	3.1×10^{-8}	1.4×10^{-8}	1.4×10^{-8}	1.4×10^{-8}	1.1×10^{-8}	
		M	0.005	4.2×10^{-7}	5.0×10^{-4}	3.2×10^{-7}	2.0×10^{-7}	1.5×10^{-7}	1.4×10^{-7}	1.4×10^{-7}	1.1×10^{-7}		
		S	0.005	4.6×10^{-7}	5.0×10^{-4}	3.5×10^{-7}	2.2×10^{-7}	1.7×10^{-7}	1.6×10^{-7}	1.6×10^{-7}	1.3×10^{-7}		
Ac-225	10.0 d	F	0.005	1.1×10^{-5}	5.0×10^{-4}	7.7×10^{-6}	4.0×10^{-6}	2.6×10^{-6}	1.1×10^{-6}	1.1×10^{-6}	1.1×10^{-6}	8.8×10^{-7}	
		M	0.005	2.8×10^{-5}	5.0×10^{-4}	2.1×10^{-5}	1.3×10^{-5}	1.0×10^{-5}	9.3×10^{-6}	9.3×10^{-6}	7.4×10^{-6}		
		S	0.005	3.1×10^{-5}	5.0×10^{-4}	2.3×10^{-5}	1.5×10^{-5}	1.1×10^{-5}	1.1×10^{-5}	1.1×10^{-5}	8.5×10^{-6}		
Ac-226	1.21 d	F	0.005	1.5×10^{-6}	5.0×10^{-4}	1.1×10^{-6}	4.0×10^{-7}	2.6×10^{-7}	1.2×10^{-7}	1.2×10^{-7}	1.2×10^{-7}	9.6×10^{-8}	
		M	0.005	4.3×10^{-6}	5.0×10^{-4}	3.2×10^{-6}	2.1×10^{-6}	1.5×10^{-6}	1.5×10^{-6}	1.5×10^{-6}	1.2×10^{-6}		
		S	0.005	4.7×10^{-6}	5.0×10^{-4}	3.5×10^{-6}	2.3×10^{-6}	1.7×10^{-6}	1.7×10^{-6}	1.6×10^{-6}	1.3×10^{-6}		
Ac-227	21.8 a	F	0.005	1.7×10^{-3}	5.0×10^{-4}	1.6×10^{-3}	1.0×10^{-3}	7.2×10^{-4}	5.6×10^{-4}	5.6×10^{-4}	5.5×10^{-4}		
		M	0.005	5.7×10^{-4}	5.0×10^{-4}	5.5×10^{-4}	3.9×10^{-4}	2.6×10^{-4}	2.3×10^{-4}	2.3×10^{-4}	2.2×10^{-4}		
		S	0.005	2.2×10^{-4}	5.0×10^{-4}	2.0×10^{-4}	1.3×10^{-4}	8.7×10^{-5}	7.6×10^{-5}	7.6×10^{-5}	7.2×10^{-5}		
Ac-228	6.13 h	F	0.005	1.8×10^{-7}	5.0×10^{-4}	1.6×10^{-7}	9.7×10^{-8}	5.7×10^{-8}	2.9×10^{-8}	2.9×10^{-8}	2.5×10^{-8}		
		M	0.005	8.4×10^{-8}	5.0×10^{-4}	7.3×10^{-8}	4.7×10^{-8}	2.9×10^{-8}	2.0×10^{-8}	2.0×10^{-8}	1.7×10^{-8}		
		S	0.005	6.4×10^{-8}	5.0×10^{-4}	5.3×10^{-8}	3.3×10^{-8}	2.2×10^{-8}	1.9×10^{-8}	1.9×10^{-8}	1.6×10^{-8}		

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$				
Thorium														
Th-226	0.515 h	F	0.005	5.0×10^{-4}	1.0×10^{-7}	4.8×10^{-8}	3.4×10^{-8}	2.5×10^{-8}	2.2×10^{-8}					
		M	0.005	5.0×10^{-4}	2.1×10^{-7}	1.1×10^{-7}	8.3×10^{-8}	7.0×10^{-8}	5.8×10^{-8}					
		S	0.005	5.0×10^{-4}	2.2×10^{-7}	1.2×10^{-7}	8.8×10^{-8}	7.5×10^{-8}	6.1×10^{-8}					
Th-227	18.7 d	F	0.005	5.0×10^{-4}	5.2×10^{-6}	2.6×10^{-6}	1.6×10^{-6}	1.0×10^{-6}	6.7×10^{-7}					
		M	0.005	5.0×10^{-4}	2.5×10^{-5}	1.6×10^{-5}	1.1×10^{-5}	1.1×10^{-5}	8.5×10^{-6}					
		S	0.005	5.0×10^{-4}	3.0×10^{-5}	1.9×10^{-5}	1.4×10^{-5}	1.3×10^{-5}	1.0×10^{-5}					
Th-228	1.91 a	F	0.005	5.0×10^{-4}	1.5×10^{-4}	8.3×10^{-5}	5.2×10^{-5}	3.6×10^{-5}	2.9×10^{-5}					
		M	0.005	5.0×10^{-4}	1.1×10^{-4}	6.8×10^{-5}	4.6×10^{-5}	3.9×10^{-5}	3.2×10^{-5}					
		S	0.005	5.0×10^{-4}	1.3×10^{-4}	8.2×10^{-5}	5.5×10^{-5}	4.7×10^{-5}	4.0×10^{-5}					
Th-229	7.34×10^3 a	F	0.005	5.0×10^{-4}	5.1×10^{-4}	3.6×10^{-4}	2.9×10^{-4}	2.4×10^{-4}	2.4×10^{-4}					
		M	0.005	5.0×10^{-4}	2.1×10^{-4}	1.6×10^{-4}	1.2×10^{-4}	1.1×10^{-4}	1.1×10^{-4}					
		S	0.005	5.0×10^{-4}	1.9×10^{-4}	1.3×10^{-4}	8.7×10^{-5}	7.6×10^{-5}	7.1×10^{-5}					
Th-230	7.70×10^4 a	F	0.005	5.0×10^{-4}	2.0×10^{-4}	1.4×10^{-4}	1.1×10^{-4}	9.9×10^{-5}	1.0×10^{-4}					
		M	0.005	5.0×10^{-4}	7.4×10^{-5}	5.5×10^{-5}	4.3×10^{-5}	4.2×10^{-5}	4.3×10^{-5}					
		S	0.005	5.0×10^{-4}	3.5×10^{-5}	2.4×10^{-5}	1.6×10^{-5}	1.5×10^{-5}	1.4×10^{-5}					
Th-231	1.06 d	F	0.005	5.0×10^{-4}	7.2×10^{-10}	2.6×10^{-10}	1.6×10^{-10}	9.2×10^{-11}	7.8×10^{-11}					
		M	0.005	5.0×10^{-4}	1.6×10^{-9}	8.0×10^{-10}	4.8×10^{-10}	3.8×10^{-10}	3.1×10^{-10}					
		S	0.005	5.0×10^{-4}	1.7×10^{-9}	7.6×10^{-10}	5.2×10^{-10}	4.1×10^{-10}	3.3×10^{-10}					
Th-232	1.40×10^{10} a	F	0.005	5.0×10^{-4}	2.2×10^{-4}	1.6×10^{-4}	1.3×10^{-4}	1.2×10^{-4}	1.1×10^{-4}					
		M	0.005	5.0×10^{-4}	8.1×10^{-5}	6.3×10^{-5}	5.0×10^{-5}	4.7×10^{-5}	4.5×10^{-5}					
		S	0.005	5.0×10^{-4}	5.0×10^{-5}	3.7×10^{-5}	2.6×10^{-5}	2.5×10^{-5}	2.5×10^{-5}					

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Th-234	24.1 d	F	0.005	4.0×10^{-8}	5.0×10^{-4}	2.5×10^{-8}	1.1×10^{-8}	6.1×10^{-9}	3.5×10^{-9}	3.5×10^{-9}	2.5×10^{-9}		
		M	0.005	3.9×10^{-8}	5.0×10^{-4}	2.9×10^{-8}	1.5×10^{-8}	1.0×10^{-8}	7.9×10^{-9}	7.9×10^{-9}	6.6×10^{-9}		
		S	0.005	4.1×10^{-8}	5.0×10^{-4}	3.1×10^{-8}	1.7×10^{-8}	1.1×10^{-8}	9.1×10^{-9}	9.1×10^{-9}	7.7×10^{-9}		
Protactinium													
Pa-227	0.638 h	M	0.005	3.6×10^{-7}	5.0×10^{-4}	2.6×10^{-7}	1.4×10^{-7}	1.0×10^{-7}	9.0×10^{-8}	9.0×10^{-8}	7.4×10^{-8}		
		S	0.005	3.8×10^{-7}	5.0×10^{-4}	2.8×10^{-7}	1.5×10^{-7}	1.1×10^{-7}	8.1×10^{-8}	8.1×10^{-8}	8.0×10^{-8}		
Pa-228	22.0 h	M	0.005	2.6×10^{-7}	5.0×10^{-4}	2.1×10^{-7}	1.3×10^{-7}	8.8×10^{-8}	7.7×10^{-8}	7.7×10^{-8}	6.4×10^{-8}		
		S	0.005	2.9×10^{-7}	5.0×10^{-4}	2.4×10^{-7}	1.5×10^{-7}	1.0×10^{-7}	9.1×10^{-8}	9.1×10^{-8}	7.5×10^{-8}		
Pa-230	17.4 d	M	0.005	2.4×10^{-6}	5.0×10^{-4}	1.8×10^{-6}	1.1×10^{-6}	8.3×10^{-7}	7.6×10^{-7}	7.6×10^{-7}	6.1×10^{-7}		
		S	0.005	2.9×10^{-6}	5.0×10^{-4}	2.2×10^{-6}	1.4×10^{-6}	1.0×10^{-6}	9.6×10^{-7}	9.6×10^{-7}	7.6×10^{-7}		
Pa-231	3.27×10^4 a	M	0.005	2.2×10^{-4}	5.0×10^{-4}	2.3×10^{-4}	1.9×10^{-4}	1.5×10^{-4}	1.5×10^{-4}	1.5×10^{-4}	1.4×10^{-4}		
		S	0.005	7.4×10^{-5}	5.0×10^{-4}	6.9×10^{-5}	5.2×10^{-5}	3.9×10^{-5}	3.6×10^{-5}	3.6×10^{-5}	3.4×10^{-5}		
Pa-232	1.31 d	M	0.005	1.9×10^{-8}	5.0×10^{-4}	1.8×10^{-8}	1.4×10^{-8}	1.1×10^{-8}	1.0×10^{-8}	1.0×10^{-8}	1.0×10^{-8}		
		S	0.005	1.0×10^{-8}	5.0×10^{-4}	8.7×10^{-9}	5.9×10^{-9}	4.1×10^{-9}	3.7×10^{-9}	3.7×10^{-9}	3.5×10^{-9}		
Pa-233	27.0 d	M	0.005	1.5×10^{-8}	5.0×10^{-4}	1.1×10^{-8}	6.5×10^{-9}	4.7×10^{-9}	4.1×10^{-9}	4.1×10^{-9}	3.3×10^{-9}		
		S	0.005	1.7×10^{-8}	5.0×10^{-4}	1.3×10^{-8}	7.5×10^{-9}	5.5×10^{-9}	4.9×10^{-9}	4.9×10^{-9}	3.9×10^{-9}		
Pa-234	6.70 h	M	0.005	2.8×10^{-9}	5.0×10^{-4}	2.0×10^{-9}	1.0×10^{-9}	6.8×10^{-10}	4.7×10^{-10}	4.7×10^{-10}	3.8×10^{-10}		
		S	0.005	2.9×10^{-9}	5.0×10^{-4}	2.1×10^{-9}	1.1×10^{-9}	7.1×10^{-10}	5.0×10^{-10}	5.0×10^{-10}	4.0×10^{-10}		

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)				
Uranium													
U-230	20.8 d	F	0.040	3.2 × 10 ^{−6}	0.020	1.5 × 10 ^{−6}	7.2 × 10 ^{−7}	5.4 × 10 ^{−7}	4.1 × 10 ^{−7}	3.8 × 10 ^{−7}			
		M	0.040	4.9 × 10 ^{−5}	0.020	3.7 × 10 ^{−5}	2.4 × 10 ^{−5}	1.8 × 10 ^{−5}	1.7 × 10 ^{−5}	1.3 × 10 ^{−5}			
		S	0.020	5.8 × 10 ^{−5}	0.002	4.4 × 10 ^{−5}	2.8 × 10 ^{−5}	2.1 × 10 ^{−5}	2.0 × 10 ^{−5}	1.6 × 10 ^{−5}			
U-231	4.20 d	F	0.040	8.9 × 10 ^{−10}	0.020	6.2 × 10 ^{−10}	3.1 × 10 ^{−10}	1.4 × 10 ^{−10}	1.0 × 10 ^{−10}	6.2 × 10 ^{−11}			
		M	0.040	2.4 × 10 ^{−9}	0.020	1.7 × 10 ^{−9}	9.4 × 10 ^{−10}	5.5 × 10 ^{−10}	4.6 × 10 ^{−10}	3.8 × 10 ^{−10}			
		S	0.020	2.6 × 10 ^{−9}	0.002	1.9 × 10 ^{−9}	9.0 × 10 ^{−10}	6.1 × 10 ^{−10}	4.9 × 10 ^{−10}	4.0 × 10 ^{−10}			
U-232	72.0 a	F	0.040	1.6 × 10 ^{−5}	0.020	1.0 × 10 ^{−5}	6.9 × 10 ^{−6}	6.8 × 10 ^{−6}	7.5 × 10 ^{−6}	4.0 × 10 ^{−6}			
		M	0.040	3.0 × 10 ^{−5}	0.020	2.4 × 10 ^{−5}	1.6 × 10 ^{−5}	1.1 × 10 ^{−5}	1.0 × 10 ^{−5}	7.8 × 10 ^{−6}			
		S	0.020	1.0 × 10 ^{−4}	0.002	9.7 × 10 ^{−5}	6.6 × 10 ^{−5}	4.3 × 10 ^{−5}	3.8 × 10 ^{−5}	3.7 × 10 ^{−5}			
U-233	1.58 × 10 ⁵ a	F	0.040	2.2 × 10 ^{−6}	0.020	1.4 × 10 ^{−6}	9.4 × 10 ^{−7}	8.4 × 10 ^{−7}	8.6 × 10 ^{−7}	5.8 × 10 ^{−7}			
		M	0.040	1.5 × 10 ^{−5}	0.020	1.1 × 10 ^{−5}	7.2 × 10 ^{−6}	4.9 × 10 ^{−6}	4.3 × 10 ^{−6}	3.6 × 10 ^{−6}			
		S	0.020	3.4 × 10 ^{−5}	0.002	3.0 × 10 ^{−5}	1.9 × 10 ^{−5}	1.2 × 10 ^{−5}	1.1 × 10 ^{−5}	9.6 × 10 ^{−6}			
U-234	2.44 × 10 ⁵ a	F	0.040	2.1 × 10 ^{−6}	0.020	1.4 × 10 ^{−6}	9.0 × 10 ^{−7}	8.0 × 10 ^{−7}	8.2 × 10 ^{−7}	5.6 × 10 ^{−7}			
		M	0.040	1.5 × 10 ^{−5}	0.020	1.1 × 10 ^{−5}	7.0 × 10 ^{−6}	4.8 × 10 ^{−6}	4.2 × 10 ^{−6}	3.5 × 10 ^{−6}			
		S	0.020	3.3 × 10 ^{−5}	0.002	2.9 × 10 ^{−5}	1.9 × 10 ^{−5}	1.2 × 10 ^{−5}	1.0 × 10 ^{−5}	9.4 × 10 ^{−6}			
U-235	7.04 × 10 ⁸ a	F	0.040	2.0 × 10 ^{−6}	0.020	1.3 × 10 ^{−6}	8.5 × 10 ^{−7}	7.5 × 10 ^{−7}	7.7 × 10 ^{−7}	5.2 × 10 ^{−7}			
		M	0.040	1.3 × 10 ^{−5}	0.020	1.0 × 10 ^{−5}	6.3 × 10 ^{−6}	4.3 × 10 ^{−6}	3.7 × 10 ^{−6}	3.1 × 10 ^{−6}			
		S	0.020	3.0 × 10 ^{−5}	0.002	2.6 × 10 ^{−5}	1.7 × 10 ^{−5}	1.1 × 10 ^{−5}	9.2 × 10 ^{−6}	8.5 × 10 ^{−6}			
U-236	2.34 × 10 ⁷ a	F	0.040	2.0 × 10 ^{−6}	0.020	1.3 × 10 ^{−6}	8.5 × 10 ^{−7}	7.5 × 10 ^{−7}	7.8 × 10 ^{−7}	5.3 × 10 ^{−7}			
		M	0.040	1.4 × 10 ^{−5}	0.020	1.0 × 10 ^{−5}	6.5 × 10 ^{−6}	4.5 × 10 ^{−6}	3.9 × 10 ^{−6}	3.2 × 10 ^{−6}			
		S	0.020	3.1 × 10 ^{−5}	0.002	2.7 × 10 ^{−5}	1.8 × 10 ^{−5}	1.1 × 10 ^{−5}	9.5 × 10 ^{−6}	8.7 × 10 ^{−6}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		<i>e</i> (g)
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)			
U-237	6.75 d	F	0.040	1.8 × 10 ^{−9}	0.020	1.5 × 10 ^{−9}	6.6 × 10 ^{−10}	4.2 × 10 ^{−10}	1.9 × 10 ^{−10}			1.8 × 10 ^{−10}	
		M	0.040	7.8 × 10 ^{−9}	0.020	5.7 × 10 ^{−9}	3.3 × 10 ^{−9}	2.4 × 10 ^{−9}	2.1 × 10 ^{−9}			1.7 × 10 ^{−9}	
		S	0.020	8.7 × 10 ^{−9}	0.002	6.4 × 10 ^{−9}	3.7 × 10 ^{−9}	2.7 × 10 ^{−9}	2.4 × 10 ^{−9}			1.9 × 10 ^{−9}	
U-238	4.47 × 10 ⁹ a	F	0.040	1.9 × 10 ^{−6}	0.020	1.3 × 10 ^{−6}	8.2 × 10 ^{−7}	7.3 × 10 ^{−7}	7.4 × 10 ^{−7}			5.0 × 10 ^{−7}	
		M	0.040	1.2 × 10 ^{−5}	0.020	9.4 × 10 ^{−6}	5.9 × 10 ^{−6}	4.0 × 10 ^{−6}	3.4 × 10 ^{−6}			2.9 × 10 ^{−6}	
		S	0.020	2.9 × 10 ^{−5}	0.002	2.5 × 10 ^{−5}	1.6 × 10 ^{−5}	1.0 × 10 ^{−5}	8.7 × 10 ^{−6}			8.0 × 10 ^{−6}	
U-239	0.392 h	F	0.040	1.0 × 10 ^{−10}	0.020	6.6 × 10 ^{−11}	2.9 × 10 ^{−11}	1.9 × 10 ^{−11}	1.2 × 10 ^{−11}			1.0 × 10 ^{−11}	
		M	0.040	1.8 × 10 ^{−10}	0.020	1.2 × 10 ^{−10}	5.6 × 10 ^{−11}	3.8 × 10 ^{−11}	2.7 × 10 ^{−11}			2.2 × 10 ^{−11}	
		S	0.020	1.9 × 10 ^{−10}	0.002	1.2 × 10 ^{−10}	5.9 × 10 ^{−11}	4.0 × 10 ^{−11}	2.9 × 10 ^{−11}			2.4 × 10 ^{−11}	
U-240	14.1 h	F	0.040	2.4 × 10 ^{−9}	0.020	1.6 × 10 ^{−9}	7.1 × 10 ^{−10}	4.5 × 10 ^{−10}	2.3 × 10 ^{−10}			2.0 × 10 ^{−10}	
		M	0.040	4.6 × 10 ^{−9}	0.020	3.1 × 10 ^{−9}	1.7 × 10 ^{−9}	1.1 × 10 ^{−9}	6.5 × 10 ^{−10}			5.3 × 10 ^{−10}	
		S	0.020	4.9 × 10 ^{−9}	0.002	3.3 × 10 ^{−9}	1.6 × 10 ^{−9}	1.1 × 10 ^{−9}	7.0 × 10 ^{−10}			5.8 × 10 ^{−10}	
Neptunium													
Np-232	0.245 h	F	0.005	2.0 × 10 ^{−10}	5.0 × 10 ^{−04}	1.9 × 10 ^{−10}	1.2 × 10 ^{−10}	1.1 × 10 ^{−10}	1.1 × 10 ^{−10}			1.2 × 10 ^{−10}	
		M	0.005	8.9 × 10 ^{−11}	5.0 × 10 ^{−04}	8.1 × 10 ^{−11}	5.5 × 10 ^{−11}	4.5 × 10 ^{−11}	4.7 × 10 ^{−11}			5.0 × 10 ^{−11}	
		S	0.005	1.2 × 10 ^{−10}	5.0 × 10 ^{−04}	9.7 × 10 ^{−11}	5.8 × 10 ^{−11}	3.9 × 10 ^{−11}	2.5 × 10 ^{−11}			2.4 × 10 ^{−11}	
Np-233	0.603 h	F	0.005	1.1 × 10 ^{−11}	5.0 × 10 ^{−04}	8.7 × 10 ^{−12}	4.2 × 10 ^{−12}	2.5 × 10 ^{−12}	1.4 × 10 ^{−12}			1.1 × 10 ^{−12}	
		M	0.005	1.5 × 10 ^{−11}	5.0 × 10 ^{−04}	1.1 × 10 ^{−11}	5.5 × 10 ^{−12}	3.3 × 10 ^{−12}	2.1 × 10 ^{−12}			1.6 × 10 ^{−12}	
		S	0.005	1.5 × 10 ^{−11}	5.0 × 10 ^{−04}	1.2 × 10 ^{−11}	5.7 × 10 ^{−12}	3.4 × 10 ^{−12}	2.1 × 10 ^{−12}			1.7 × 10 ^{−12}	

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Np-234	4.40 d	F	0.005	2.9×10^{-9}	5.0×10^{-4}	2.2×10^{-9}	1.1×10^{-9}	7.2×10^{-10}	4.3×10^{-10}	3.5×10^{-10}				
		M	0.005	3.8×10^{-9}	5.0×10^{-4}	3.0×10^{-9}	1.6×10^{-9}	1.0×10^{-9}	6.5×10^{-10}	5.3×10^{-10}				
		S	0.005	3.9×10^{-9}	5.0×10^{-4}	3.1×10^{-9}	1.6×10^{-9}	1.0×10^{-9}	6.8×10^{-10}	5.5×10^{-10}				
Np-235	1.08 a	F	0.005	4.2×10^{-9}	5.0×10^{-4}	3.5×10^{-9}	1.9×10^{-9}	1.1×10^{-9}	7.5×10^{-10}	6.3×10^{-10}				
		M	0.005	2.3×10^{-9}	5.0×10^{-4}	1.9×10^{-9}	1.1×10^{-9}	6.8×10^{-10}	5.1×10^{-10}	4.2×10^{-10}				
		S	0.005	2.6×10^{-9}	5.0×10^{-4}	2.2×10^{-9}	1.3×10^{-9}	8.3×10^{-10}	6.3×10^{-10}	5.2×10^{-10}				
Np-236	1.15×10^5 a	F	0.005	8.9×10^{-6}	5.0×10^{-4}	9.1×10^{-6}	7.2×10^{-6}	7.5×10^{-6}	7.9×10^{-6}	8.0×10^{-6}				
		M	0.005	3.0×10^{-6}	5.0×10^{-4}	3.1×10^{-6}	2.7×10^{-6}	2.7×10^{-6}	3.1×10^{-6}	3.2×10^{-6}				
		S	0.005	1.6×10^{-6}	5.0×10^{-4}	1.6×10^{-6}	1.3×10^{-6}	1.0×10^{-6}	1.0×10^{-6}	1.0×10^{-6}	1.0×10^{-6}			
Np-236m	22.5 h	F	0.005	2.8×10^{-8}	5.0×10^{-4}	2.6×10^{-8}	1.5×10^{-8}	1.1×10^{-8}	8.9×10^{-9}	9.0×10^{-9}				
		M	0.005	1.6×10^{-8}	5.0×10^{-4}	1.4×10^{-8}	8.9×10^{-9}	6.2×10^{-9}	5.6×10^{-9}	5.3×10^{-9}				
		S	0.005	1.6×10^{-8}	5.0×10^{-4}	1.3×10^{-8}	8.5×10^{-9}	5.7×10^{-9}	4.8×10^{-9}	4.2×10^{-9}				
Np-237	2.14×10^6 a	F	0.005	9.8×10^{-5}	5.0×10^{-4}	9.3×10^{-5}	6.0×10^{-5}	5.0×10^{-5}	4.7×10^{-5}	5.0×10^{-5}				
		M	0.005	4.4×10^{-5}	5.0×10^{-4}	4.0×10^{-5}	2.8×10^{-5}	2.2×10^{-5}	2.2×10^{-5}	2.3×10^{-5}				
		S	0.005	3.7×10^{-5}	5.0×10^{-4}	3.2×10^{-5}	2.1×10^{-5}	1.4×10^{-5}	1.3×10^{-5}	1.2×10^{-5}				
Np-238	2.12 d	F	0.005	9.0×10^{-9}	5.0×10^{-4}	7.9×10^{-9}	4.8×10^{-9}	3.7×10^{-9}	3.3×10^{-9}	3.5×10^{-9}				
		M	0.005	7.3×10^{-9}	5.0×10^{-4}	5.8×10^{-9}	3.4×10^{-9}	2.5×10^{-9}	2.2×10^{-9}	2.1×10^{-9}				
		S	0.005	8.1×10^{-9}	5.0×10^{-4}	6.2×10^{-9}	3.2×10^{-9}	2.1×10^{-9}	1.7×10^{-9}	1.5×10^{-9}				
Np-239	2.36 d	F	0.005	2.6×10^{-9}	5.0×10^{-4}	1.4×10^{-9}	6.3×10^{-10}	3.8×10^{-10}	2.1×10^{-10}	1.7×10^{-10}				
		M	0.005	5.9×10^{-9}	5.0×10^{-4}	4.2×10^{-9}	2.0×10^{-9}	1.4×10^{-9}	1.2×10^{-9}	9.3×10^{-10}				
		S	0.005	5.6×10^{-9}	5.0×10^{-4}	4.0×10^{-9}	2.2×10^{-9}	1.6×10^{-9}	1.3×10^{-9}	1.0×10^{-9}				

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Np-240	1.08 h	F	0.005	3.6×10^{-10}	5.0×10^{-4}	2.6×10^{-10}	1.2×10^{-10}	7.7×10^{-11}	4.7×10^{-11}	4.0×10^{-11}			
		M	0.005	6.3×10^{-10}	5.0×10^{-4}	4.4×10^{-10}	2.2×10^{-10}	1.4×10^{-10}	1.0×10^{-10}	8.5×10^{-11}			
		S	0.005	6.5×10^{-10}	5.0×10^{-4}	4.6×10^{-10}	2.3×10^{-10}	1.5×10^{-10}	1.1×10^{-10}	9.0×10^{-11}			
Plutonium													
Pu-234	8.80 h	F	0.005	3.0×10^{-8}	5.0×10^{-4}	2.0×10^{-8}	9.8×10^{-9}	5.7×10^{-9}	3.6×10^{-9}	3.0×10^{-9}			
		M	0.005	7.8×10^{-8}	5.0×10^{-4}	5.9×10^{-8}	3.7×10^{-8}	2.8×10^{-8}	2.6×10^{-8}	2.1×10^{-8}			
		S	1.0×10^{-4}	8.7×10^{-8}	1.0×10^{-5}	6.6×10^{-8}	4.2×10^{-8}	3.1×10^{-8}	3.0×10^{-8}	2.4×10^{-8}			
Pu-235	0.422 h	F	0.005	1.0×10^{-11}	5.0×10^{-4}	7.9×10^{-12}	3.9×10^{-12}	2.2×10^{-12}	1.3×10^{-12}	1.0×10^{-12}			
		M	0.005	1.3×10^{-11}	5.0×10^{-4}	1.0×10^{-11}	5.0×10^{-12}	2.9×10^{-12}	1.9×10^{-12}	1.4×10^{-12}			
		S	1.0×10^{-4}	1.3×10^{-11}	1.0×10^{-5}	1.0×10^{-11}	5.1×10^{-12}	3.0×10^{-12}	1.9×10^{-12}	1.5×10^{-12}			
Pu-236m	2.85 a	F	0.005	1.0×10^{-4}	5.0×10^{-4}	9.5×10^{-5}	6.1×10^{-5}	4.4×10^{-5}	3.7×10^{-5}	4.0×10^{-5}			
		M	0.005	4.8×10^{-5}	5.0×10^{-4}	4.3×10^{-5}	2.9×10^{-5}	2.1×10^{-5}	1.9×10^{-5}	2.0×10^{-5}			
		S	1.0×10^{-4}	3.6×10^{-5}	1.0×10^{-5}	3.1×10^{-5}	2.0×10^{-5}	1.4×10^{-5}	1.2×10^{-5}	1.0×10^{-5}			
Pu-237	45.3 d	F	0.005	2.2×10^{-9}	5.0×10^{-4}	1.6×10^{-9}	7.9×10^{-10}	4.8×10^{-10}	2.9×10^{-10}	2.6×10^{-10}			
		M	0.005	1.9×10^{-9}	5.0×10^{-4}	1.4×10^{-9}	8.2×10^{-10}	5.4×10^{-10}	4.3×10^{-10}	3.5×10^{-10}			
		S	1.0×10^{-4}	2.0×10^{-9}	1.0×10^{-5}	1.5×10^{-9}	8.8×10^{-10}	5.9×10^{-10}	4.8×10^{-10}	3.9×10^{-10}			
Pu-238	87.7 a	F	0.005	2.0×10^{-4}	5.0×10^{-4}	1.9×10^{-4}	1.4×10^{-4}	1.1×10^{-4}	1.0×10^{-4}	1.1×10^{-4}			
		M	0.005	7.8×10^{-5}	5.0×10^{-4}	7.4×10^{-5}	5.6×10^{-5}	4.4×10^{-5}	4.3×10^{-5}	4.6×10^{-5}			
		S	1.0×10^{-4}	4.5×10^{-5}	1.0×10^{-5}	4.0×10^{-5}	2.7×10^{-5}	1.9×10^{-5}	1.7×10^{-5}	1.6×10^{-5}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	<i>f</i> ₁		<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)	<i>e</i> (g)			
Pu-239	2.41 × 10 ⁴ a	F	0.005	2.1 × 10 ^{−4}	5.0 × 10 ^{−4}	2.0 × 10 ^{−4}	1.5 × 10 ^{−4}	1.2 × 10 ^{−4}	1.2 × 10 ^{−4}	1.1 × 10 ^{−4}	1.1 × 10 ^{−4}	1.2 × 10 ^{−4}	
		M	0.005	8.0 × 10 ^{−5}	5.0 × 10 ^{−4}	7.7 × 10 ^{−5}	6.0 × 10 ^{−5}	4.8 × 10 ^{−5}	4.8 × 10 ^{−5}	4.7 × 10 ^{−5}	4.7 × 10 ^{−5}	5.0 × 10 ^{−5}	
		S	1.0 × 10 ^{−4}	4.3 × 10 ^{−5}	1.0 × 10 ^{−5}	3.9 × 10 ^{−5}	2.7 × 10 ^{−5}	1.9 × 10 ^{−5}	1.9 × 10 ^{−5}	1.7 × 10 ^{−5}	1.7 × 10 ^{−5}	1.6 × 10 ^{−5}	
Pu-240	6.54 × 10 ³ a	F	0.005	2.1 × 10 ^{−4}	5.0 × 10 ^{−4}	2.0 × 10 ^{−4}	1.5 × 10 ^{−4}	1.2 × 10 ^{−4}	1.2 × 10 ^{−4}	1.1 × 10 ^{−4}	1.1 × 10 ^{−4}	1.2 × 10 ^{−4}	
		M	0.005	8.0 × 10 ^{−5}	5.0 × 10 ^{−4}	7.7 × 10 ^{−5}	6.0 × 10 ^{−5}	4.8 × 10 ^{−5}	4.8 × 10 ^{−5}	4.7 × 10 ^{−5}	4.7 × 10 ^{−5}	5.0 × 10 ^{−5}	
		S	1.0 × 10 ^{−4}	4.3 × 10 ^{−5}	1.0 × 10 ^{−5}	3.9 × 10 ^{−5}	2.7 × 10 ^{−5}	1.9 × 10 ^{−5}	1.9 × 10 ^{−5}	1.7 × 10 ^{−5}	1.7 × 10 ^{−5}	1.6 × 10 ^{−5}	
Pu-241	14.4 a	F	0.005	2.8 × 10 ^{−6}	5.0 × 10 ^{−4}	2.9 × 10 ^{−6}	2.6 × 10 ^{−6}	2.4 × 10 ^{−6}	2.4 × 10 ^{−6}	2.2 × 10 ^{−6}	2.2 × 10 ^{−6}	2.3 × 10 ^{−6}	
		M	0.005	9.1 × 10 ^{−7}	5.0 × 10 ^{−4}	9.7 × 10 ^{−7}	9.2 × 10 ^{−7}	8.3 × 10 ^{−7}	8.3 × 10 ^{−7}	8.6 × 10 ^{−7}	8.6 × 10 ^{−7}	9.0 × 10 ^{−7}	
		S	1.0 × 10 ^{−4}	2.2 × 10 ^{−7}	1.0 × 10 ^{−5}	2.3 × 10 ^{−7}	2.0 × 10 ^{−7}	1.7 × 10 ^{−7}	1.7 × 10 ^{−7}	1.7 × 10 ^{−7}	1.7 × 10 ^{−7}	1.7 × 10 ^{−7}	
Pu-242	3.76 × 10 ⁵ a	F	0.005	2.0 × 10 ^{−4}	5.0 × 10 ^{−4}	1.9 × 10 ^{−4}	1.4 × 10 ^{−4}	1.2 × 10 ^{−4}	1.2 × 10 ^{−4}	1.1 × 10 ^{−4}	1.1 × 10 ^{−4}	1.1 × 10 ^{−4}	
		M	0.005	7.6 × 10 ^{−5}	5.0 × 10 ^{−4}	7.3 × 10 ^{−5}	5.7 × 10 ^{−5}	4.5 × 10 ^{−5}	4.5 × 10 ^{−5}	4.5 × 10 ^{−5}	4.5 × 10 ^{−5}	4.8 × 10 ^{−5}	
		S	1.0 × 10 ^{−4}	4.0 × 10 ^{−5}	1.0 × 10 ^{−5}	3.6 × 10 ^{−5}	2.5 × 10 ^{−5}	1.7 × 10 ^{−5}	1.7 × 10 ^{−5}	1.6 × 10 ^{−5}	1.6 × 10 ^{−5}	1.5 × 10 ^{−5}	
Pu-243	4.95 h	F	0.005	2.7 × 10 ^{−10}	5.0 × 10 ^{−4}	1.9 × 10 ^{−10}	8.8 × 10 ^{−11}	5.7 × 10 ^{−11}	5.7 × 10 ^{−11}	3.5 × 10 ^{−11}	3.5 × 10 ^{−11}	3.2 × 10 ^{−11}	
		M	0.005	5.6 × 10 ^{−10}	5.0 × 10 ^{−4}	3.9 × 10 ^{−10}	1.9 × 10 ^{−10}	1.3 × 10 ^{−10}	1.3 × 10 ^{−10}	8.7 × 10 ^{−11}	8.7 × 10 ^{−11}	8.3 × 10 ^{−11}	
		S	1.0 × 10 ^{−4}	6.0 × 10 ^{−10}	1.0 × 10 ^{−5}	4.1 × 10 ^{−10}	2.0 × 10 ^{−10}	1.4 × 10 ^{−10}	1.4 × 10 ^{−10}	9.2 × 10 ^{−11}	9.2 × 10 ^{−11}	8.6 × 10 ^{−11}	
Pu-244	8.26 × 10 ⁷ a	F	0.005	2.0 × 10 ^{−4}	5.0 × 10 ^{−4}	1.9 × 10 ^{−4}	1.4 × 10 ^{−4}	1.2 × 10 ^{−4}	1.2 × 10 ^{−4}	1.1 × 10 ^{−4}	1.1 × 10 ^{−4}	1.1 × 10 ^{−4}	
		M	0.005	7.4 × 10 ^{−5}	5.0 × 10 ^{−4}	7.2 × 10 ^{−5}	5.6 × 10 ^{−5}	4.5 × 10 ^{−5}	4.5 × 10 ^{−5}	4.4 × 10 ^{−5}	4.4 × 10 ^{−5}	4.7 × 10 ^{−5}	
		S	1.0 × 10 ^{−4}	3.9 × 10 ^{−5}	1.0 × 10 ^{−5}	3.5 × 10 ^{−5}	2.4 × 10 ^{−5}	1.7 × 10 ^{−5}	1.7 × 10 ^{−5}	1.5 × 10 ^{−5}	1.5 × 10 ^{−5}	1.5 × 10 ^{−5}	
Pu-245	10.5 h	F	0.005	1.8 × 10 ^{−9}	5.0 × 10 ^{−4}	1.3 × 10 ^{−9}	5.6 × 10 ^{−10}	3.5 × 10 ^{−10}	3.5 × 10 ^{−10}	1.9 × 10 ^{−10}	1.9 × 10 ^{−10}	1.6 × 10 ^{−10}	
		M	0.005	3.6 × 10 ^{−9}	5.0 × 10 ^{−4}	2.5 × 10 ^{−9}	1.2 × 10 ^{−9}	8.0 × 10 ^{−10}	8.0 × 10 ^{−10}	5.0 × 10 ^{−10}	5.0 × 10 ^{−10}	4.0 × 10 ^{−10}	
		S	1.0 × 10 ^{−4}	3.8 × 10 ^{−9}	1.0 × 10 ^{−5}	2.6 × 10 ^{−9}	1.3 × 10 ^{−9}	8.5 × 10 ^{−10}	8.5 × 10 ^{−10}	5.4 × 10 ^{−10}	5.4 × 10 ^{−10}	4.3 × 10 ^{−10}	

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Pu-246	10.9 d	F	0.005	2.0×10^{-8}	5.0×10^{-4}	1.4×10^{-8}	7.0×10^{-9}	4.4×10^{-9}	2.8×10^{-9}	2.5×10^{-9}			
		M	0.005	3.5×10^{-8}	5.0×10^{-4}	2.6×10^{-8}	1.5×10^{-8}	1.1×10^{-8}	9.1×10^{-9}	7.4×10^{-9}			
		S	1.0×10^{-4}	3.8×10^{-8}	1.0×10^{-5}	2.8×10^{-8}	1.6×10^{-8}	1.2×10^{-8}	1.0×10^{-8}	8.0×10^{-9}			
Americium													
Am-237	1.22 h	F	0.005	9.8×10^{-11}	5.0×10^{-4}	7.3×10^{-11}	3.5×10^{-11}	2.2×10^{-11}	1.3×10^{-11}	1.1×10^{-11}			
		M	0.005	1.7×10^{-10}	5.0×10^{-4}	1.2×10^{-10}	6.2×10^{-11}	4.1×10^{-11}	3.0×10^{-11}	2.5×10^{-11}			
		S	0.005	1.7×10^{-10}	5.0×10^{-4}	1.3×10^{-10}	6.5×10^{-11}	4.3×10^{-11}	3.2×10^{-11}	2.6×10^{-11}			
Am-238	1.63 h	F	0.005	4.1×10^{-10}	5.0×10^{-4}	3.8×10^{-10}	2.5×10^{-10}	2.0×10^{-10}	1.8×10^{-10}	1.9×10^{-10}			
		M	0.005	3.1×10^{-10}	5.0×10^{-4}	2.6×10^{-10}	1.3×10^{-10}	9.6×10^{-11}	8.8×10^{-11}	9.0×10^{-11}			
		S	0.005	2.7×10^{-10}	5.0×10^{-4}	2.2×10^{-10}	1.3×10^{-10}	8.2×10^{-11}	6.1×10^{-11}	5.4×10^{-11}			
Am-239	11.9 h	F	0.005	8.1×10^{-10}	5.0×10^{-4}	5.8×10^{-10}	2.6×10^{-10}	1.6×10^{-10}	9.1×10^{-11}	7.6×10^{-11}			
		M	0.005	1.5×10^{-9}	5.0×10^{-4}	1.1×10^{-9}	5.6×10^{-10}	3.7×10^{-10}	2.7×10^{-10}	2.2×10^{-10}			
		S	0.005	1.6×10^{-9}	5.0×10^{-4}	1.1×10^{-9}	5.9×10^{-10}	4.0×10^{-10}	2.5×10^{-10}	2.4×10^{-10}			
Am-240	2.12 d	F	0.005	2.0×10^{-9}	5.0×10^{-4}	1.7×10^{-9}	8.8×10^{-10}	5.7×10^{-10}	3.6×10^{-10}	2.3×10^{-10}			
		M	0.005	2.9×10^{-9}	5.0×10^{-4}	2.2×10^{-9}	1.2×10^{-9}	7.7×10^{-10}	5.3×10^{-10}	4.3×10^{-10}			
		S	0.005	3.0×10^{-9}	5.0×10^{-4}	2.3×10^{-9}	1.2×10^{-9}	7.8×10^{-10}	5.3×10^{-10}	4.3×10^{-10}			
Am-241	4.32×10^2 a	F	0.005	1.8×10^{-4}	5.0×10^{-4}	1.8×10^{-4}	1.2×10^{-4}	1.0×10^{-4}	9.2×10^{-5}	9.6×10^{-5}			
		M	0.005	7.3×10^{-5}	5.0×10^{-4}	6.9×10^{-5}	5.1×10^{-5}	4.0×10^{-5}	4.0×10^{-5}	4.2×10^{-5}			
		S	0.005	4.6×10^{-5}	5.0×10^{-4}	4.0×10^{-5}	2.7×10^{-5}	1.9×10^{-5}	1.7×10^{-5}	1.6×10^{-5}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Am-242	16.0 h	F	0.005	9.2×10^{-8}	5.0×10^{-4}	7.1×10^{-8}	3.5×10^{-8}	2.1×10^{-8}	2.1×10^{-8}	1.4×10^{-8}	1.1×10^{-8}		
		M	0.005	7.6×10^{-8}	5.0×10^{-4}	5.9×10^{-8}	3.6×10^{-8}	2.4×10^{-8}	2.4×10^{-8}	2.1×10^{-8}	1.7×10^{-8}		
		S	0.005	8.0×10^{-8}	5.0×10^{-4}	6.2×10^{-8}	3.9×10^{-8}	2.7×10^{-8}	2.7×10^{-8}	2.4×10^{-8}	2.0×10^{-8}		
Am-242m	1.52×10^2 a	F	0.005	1.6×10^{-4}	5.0×10^{-4}	1.5×10^{-4}	1.1×10^{-4}	9.4×10^{-5}	9.4×10^{-5}	8.8×10^{-5}	9.2×10^{-5}		
		M	0.005	5.2×10^{-5}	5.0×10^{-4}	5.3×10^{-5}	4.1×10^{-5}	3.4×10^{-5}	3.4×10^{-5}	3.5×10^{-5}	3.7×10^{-5}		
		S	0.005	2.5×10^{-5}	5.0×10^{-4}	2.4×10^{-5}	1.7×10^{-5}	1.2×10^{-5}	1.2×10^{-5}	1.1×10^{-5}	1.1×10^{-5}		
Am-243	7.38×10^3 a	F	0.005	1.8×10^{-4}	5.0×10^{-4}	1.7×10^{-4}	1.2×10^{-4}	1.0×10^{-4}	1.0×10^{-4}	9.1×10^{-5}	9.6×10^{-5}		
		M	0.005	7.2×10^{-5}	5.0×10^{-4}	6.8×10^{-5}	5.0×10^{-5}	4.0×10^{-5}	4.0×10^{-5}	4.0×10^{-5}	4.1×10^{-5}		
		S	0.005	4.4×10^{-5}	5.0×10^{-4}	3.9×10^{-5}	2.6×10^{-5}	1.8×10^{-5}	1.8×10^{-5}	1.6×10^{-5}	1.5×10^{-5}		
Am-244	10.1 h	F	0.005	1.0×10^{-8}	5.0×10^{-4}	9.2×10^{-9}	5.6×10^{-9}	4.1×10^{-9}	4.1×10^{-9}	3.5×10^{-9}	3.7×10^{-9}		
		M	0.005	6.0×10^{-9}	5.0×10^{-4}	5.0×10^{-9}	3.2×10^{-9}	2.2×10^{-9}	2.2×10^{-9}	2.0×10^{-9}	2.0×10^{-9}		
		S	0.005	6.1×10^{-9}	5.0×10^{-4}	4.8×10^{-9}	2.4×10^{-9}	1.6×10^{-9}	1.6×10^{-9}	1.4×10^{-9}	1.2×10^{-9}		
Am-244m	0.433 h	F	0.005	4.6×10^{-10}	5.0×10^{-4}	4.0×10^{-10}	2.4×10^{-10}	1.8×10^{-10}	1.8×10^{-10}	1.5×10^{-10}	1.6×10^{-10}		
		M	0.005	3.3×10^{-10}	5.0×10^{-4}	2.1×10^{-10}	1.3×10^{-10}	9.2×10^{-11}	9.2×10^{-11}	8.3×10^{-11}	8.4×10^{-11}		
		S	0.005	3.0×10^{-10}	5.0×10^{-4}	2.2×10^{-10}	1.2×10^{-10}	8.1×10^{-11}	8.1×10^{-11}	5.5×10^{-11}	5.7×10^{-11}		
Am-245	2.05 h	F	0.005	2.1×10^{-10}	5.0×10^{-4}	1.4×10^{-10}	6.2×10^{-11}	4.0×10^{-11}	4.0×10^{-11}	2.4×10^{-11}	2.1×10^{-11}		
		M	0.005	3.9×10^{-10}	5.0×10^{-4}	2.6×10^{-10}	1.3×10^{-10}	8.7×10^{-11}	8.7×10^{-11}	6.4×10^{-11}	5.3×10^{-11}		
		S	0.005	4.1×10^{-10}	5.0×10^{-4}	2.8×10^{-10}	1.3×10^{-10}	9.2×10^{-11}	9.2×10^{-11}	6.8×10^{-11}	5.6×10^{-11}		
Am-246	0.650 h	F	0.005	3.0×10^{-10}	5.0×10^{-4}	2.0×10^{-10}	9.3×10^{-11}	6.1×10^{-11}	6.1×10^{-11}	3.8×10^{-11}	3.3×10^{-11}		
		M	0.005	5.0×10^{-10}	5.0×10^{-4}	3.4×10^{-10}	1.6×10^{-10}	1.1×10^{-10}	1.1×10^{-10}	7.9×10^{-11}	6.6×10^{-11}		
		S	0.005	5.3×10^{-10}	5.0×10^{-4}	3.6×10^{-10}	1.7×10^{-10}	1.2×10^{-10}	1.2×10^{-10}	8.3×10^{-11}	6.9×10^{-11}		

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Am-246m	0.417 h	F	0.005	1.3×10^{-10}	5.0×10^{-4}	8.9×10^{-11}	4.2×10^{-11}	2.6×10^{-11}	1.6×10^{-11}	1.4×10^{-11}	1.6×10^{-11}	2.2×10^{-11}	2.3×10^{-11}	2.3×10^{-11}
		M	0.005	1.9×10^{-10}	5.0×10^{-4}	1.3×10^{-10}	6.1×10^{-11}	4.0×10^{-11}	2.6×10^{-11}	2.2×10^{-11}	2.6×10^{-11}	2.2×10^{-11}	2.2×10^{-11}	2.2×10^{-11}
		S	0.005	2.0×10^{-10}	5.0×10^{-4}	1.4×10^{-10}	6.4×10^{-11}	4.1×10^{-11}	2.7×10^{-11}	2.3×10^{-11}	2.7×10^{-11}	2.3×10^{-11}	2.3×10^{-11}	2.3×10^{-11}
Curium														
Cm-238	2.40 h	F	0.005	7.7×10^{-9}	5.0×10^{-4}	5.4×10^{-9}	2.6×10^{-9}	1.8×10^{-9}	9.2×10^{-10}	7.8×10^{-10}	9.2×10^{-10}	4.5×10^{-9}	4.9×10^{-9}	4.9×10^{-9}
		M	0.005	2.1×10^{-8}	5.0×10^{-4}	1.5×10^{-8}	7.9×10^{-9}	5.9×10^{-9}	5.6×10^{-9}	4.5×10^{-9}	5.6×10^{-9}	4.5×10^{-9}	4.5×10^{-9}	4.5×10^{-9}
		S	0.005	2.2×10^{-8}	5.0×10^{-4}	1.6×10^{-8}	8.6×10^{-9}	6.4×10^{-9}	6.1×10^{-9}	4.9×10^{-9}	6.1×10^{-9}	4.9×10^{-9}	4.9×10^{-9}	4.9×10^{-9}
Cm-240	27.0 d	F	0.005	8.3×10^{-6}	5.0×10^{-4}	6.3×10^{-6}	3.2×10^{-6}	2.0×10^{-6}	1.5×10^{-6}	1.3×10^{-6}	1.5×10^{-6}	1.3×10^{-6}	1.3×10^{-6}	1.3×10^{-6}
		M	0.005	1.2×10^{-5}	5.0×10^{-4}	9.1×10^{-6}	5.8×10^{-6}	4.2×10^{-6}	3.8×10^{-6}	3.2×10^{-6}	3.8×10^{-6}	3.2×10^{-6}	3.2×10^{-6}	3.2×10^{-6}
		S	0.005	1.3×10^{-5}	5.0×10^{-4}	9.9×10^{-6}	6.4×10^{-6}	4.6×10^{-6}	4.3×10^{-6}	3.5×10^{-6}	4.3×10^{-6}	3.5×10^{-6}	3.5×10^{-6}	3.5×10^{-6}
Cm-241	32.8 d	F	0.005	1.1×10^{-7}	5.0×10^{-4}	8.9×10^{-8}	4.9×10^{-8}	3.5×10^{-8}	2.8×10^{-8}	2.7×10^{-8}	2.8×10^{-8}	2.7×10^{-8}	2.7×10^{-8}	2.7×10^{-8}
		M	0.005	1.3×10^{-7}	5.0×10^{-4}	1.0×10^{-7}	6.6×10^{-8}	4.8×10^{-8}	4.4×10^{-8}	3.7×10^{-8}	4.4×10^{-8}	3.7×10^{-8}	3.7×10^{-8}	3.7×10^{-8}
		S	0.005	1.4×10^{-7}	5.0×10^{-4}	1.1×10^{-7}	6.9×10^{-8}	4.9×10^{-8}	4.5×10^{-8}	3.7×10^{-8}	4.5×10^{-8}	3.7×10^{-8}	3.7×10^{-8}	3.7×10^{-8}
Cm-242	163 d	F	0.005	2.7×10^{-5}	5.0×10^{-4}	2.1×10^{-5}	1.0×10^{-5}	6.1×10^{-6}	4.0×10^{-6}	3.3×10^{-6}	4.0×10^{-6}	3.3×10^{-6}	3.3×10^{-6}	3.3×10^{-6}
		M	0.005	2.2×10^{-5}	5.0×10^{-4}	1.8×10^{-5}	1.1×10^{-5}	7.3×10^{-6}	6.4×10^{-6}	5.2×10^{-6}	6.4×10^{-6}	5.2×10^{-6}	5.2×10^{-6}	5.2×10^{-6}
		S	0.005	2.4×10^{-5}	5.0×10^{-4}	1.9×10^{-5}	1.2×10^{-5}	8.2×10^{-6}	7.3×10^{-6}	5.9×10^{-6}	7.3×10^{-6}	5.9×10^{-6}	5.9×10^{-6}	5.9×10^{-6}
Cm-243	28.5 a	F	0.005	1.6×10^{-4}	5.0×10^{-4}	1.5×10^{-4}	9.5×10^{-5}	7.3×10^{-5}	6.5×10^{-5}	6.9×10^{-5}	6.5×10^{-5}	6.9×10^{-5}	6.9×10^{-5}	6.9×10^{-5}
		M	0.005	6.7×10^{-5}	5.0×10^{-4}	6.1×10^{-5}	4.2×10^{-5}	3.1×10^{-5}	3.0×10^{-5}	3.1×10^{-5}	3.0×10^{-5}	3.1×10^{-5}	3.1×10^{-5}	3.1×10^{-5}
		S	0.005	4.6×10^{-5}	5.0×10^{-4}	4.0×10^{-5}	2.6×10^{-5}	1.8×10^{-5}	1.6×10^{-5}	1.5×10^{-5}	1.6×10^{-5}	1.5×10^{-5}	1.5×10^{-5}	1.5×10^{-5}

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a	
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$		
Cm-244	18.1 a	F	0.005	1.5×10^{-4}	5.0×10^{-4}	1.3×10^{-4}	8.3×10^{-5}	6.1×10^{-5}	5.3×10^{-5}	5.7×10^{-5}				
		M	0.005	6.2×10^{-5}	5.0×10^{-4}	5.7×10^{-5}	3.7×10^{-5}	2.7×10^{-5}	2.6×10^{-5}	2.7×10^{-5}				
		S	0.005	4.4×10^{-5}	5.0×10^{-4}	3.8×10^{-5}	2.5×10^{-5}	1.7×10^{-5}	1.5×10^{-5}	1.3×10^{-5}				
Cm-245	8.50×10^3 a	F	0.005	1.9×10^{-4}	5.0×10^{-4}	1.8×10^{-4}	1.2×10^{-4}	1.0×10^{-4}	9.4×10^{-5}	9.9×10^{-5}				
		M	0.005	7.3×10^{-5}	5.0×10^{-4}	6.9×10^{-5}	5.1×10^{-5}	4.1×10^{-5}	4.1×10^{-5}	4.2×10^{-5}				
		S	0.005	4.5×10^{-5}	5.0×10^{-4}	4.0×10^{-5}	2.7×10^{-5}	1.9×10^{-5}	1.7×10^{-5}	1.6×10^{-5}				
Cm-246	4.73×10^3 a	F	0.005	1.9×10^{-4}	5.0×10^{-4}	1.8×10^{-4}	1.2×10^{-4}	1.0×10^{-4}	9.4×10^{-5}	9.8×10^{-5}				
		M	0.005	7.3×10^{-5}	5.0×10^{-4}	6.9×10^{-5}	5.1×10^{-5}	4.1×10^{-5}	4.1×10^{-5}	4.2×10^{-5}				
		S	0.005	4.6×10^{-5}	5.0×10^{-4}	4.0×10^{-5}	2.7×10^{-5}	1.9×10^{-5}	1.7×10^{-5}	1.6×10^{-5}				
Cm-247	1.56×10^7 a	F	0.005	1.7×10^{-4}	5.0×10^{-4}	1.6×10^{-4}	1.1×10^{-4}	9.4×10^{-5}	8.6×10^{-5}	9.0×10^{-5}				
		M	0.005	6.7×10^{-5}	5.0×10^{-4}	6.3×10^{-5}	4.7×10^{-5}	3.7×10^{-5}	3.7×10^{-5}	3.9×10^{-5}				
		S	0.005	4.1×10^{-5}	5.0×10^{-4}	3.6×10^{-5}	2.4×10^{-5}	1.7×10^{-5}	1.5×10^{-5}	1.4×10^{-5}				
Cm-248	3.39×10^5 a	F	0.005	6.8×10^{-4}	5.0×10^{-4}	6.5×10^{-4}	4.5×10^{-4}	3.7×10^{-4}	3.4×10^{-4}	3.6×10^{-4}				
		M	0.005	2.5×10^{-4}	5.0×10^{-4}	2.4×10^{-4}	1.8×10^{-4}	1.4×10^{-4}	1.4×10^{-4}	1.5×10^{-4}				
		S	0.005	1.4×10^{-4}	5.0×10^{-4}	1.2×10^{-4}	8.2×10^{-5}	5.6×10^{-5}	5.0×10^{-5}	4.8×10^{-5}				
Cm-249	1.07 h	F	0.005	1.8×10^{-10}	5.0×10^{-4}	9.8×10^{-11}	5.9×10^{-11}	4.6×10^{-11}	4.0×10^{-11}	4.0×10^{-11}				
		M	0.005	2.4×10^{-10}	5.0×10^{-4}	1.6×10^{-10}	8.2×10^{-11}	5.8×10^{-11}	3.7×10^{-11}	3.3×10^{-11}				
		S	0.005	2.4×10^{-10}	5.0×10^{-4}	1.6×10^{-10}	7.8×10^{-11}	5.3×10^{-11}	3.9×10^{-11}	3.3×10^{-11}				
Cm-250	6.90×10^3 a	F	0.005	3.9×10^{-3}	5.0×10^{-4}	3.7×10^{-3}	2.6×10^{-3}	2.1×10^{-3}	2.0×10^{-3}	2.1×10^{-3}				
		M	0.005	1.4×10^{-3}	5.0×10^{-4}	1.3×10^{-3}	9.9×10^{-4}	7.9×10^{-4}	7.9×10^{-4}	8.4×10^{-4}				
		S	0.005	7.2×10^{-4}	5.0×10^{-4}	6.5×10^{-4}	4.4×10^{-4}	3.0×10^{-4}	2.7×10^{-4}	2.6×10^{-4}				

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Berkelium													
Bk-245	4.94 d	M	0.005	8.8×10^{-9}	5.0×10^{-4}	6.6×10^{-9}	4.0×10^{-9}	2.9×10^{-9}	2.6×10^{-9}	2.1×10^{-9}			
Bk-246	1.83 d	M	0.005	2.1×10^{-9}	5.0×10^{-4}	1.7×10^{-9}	9.3×10^{-10}	6.0×10^{-10}	4.0×10^{-10}	3.3×10^{-10}			
Bk-247	1.38×10^3 a	M	0.005	1.5×10^{-4}	5.0×10^{-4}	1.5×10^{-4}	1.1×10^{-4}	7.9×10^{-5}	7.2×10^{-5}	6.9×10^{-5}			
Bk-249	320 d	M	0.005	3.3×10^{-7}	5.0×10^{-4}	3.3×10^{-7}	2.4×10^{-7}	1.8×10^{-7}	1.6×10^{-7}	1.6×10^{-7}			
Bk-250	3.22 h	M	0.005	3.4×10^{-9}	5.0×10^{-4}	3.1×10^{-9}	2.0×10^{-9}	1.3×10^{-9}	1.1×10^{-9}	1.0×10^{-9}			
Californium													
Cf-244	0.323 h	M	0.005	7.6×10^{-8}	5.0×10^{-4}	5.4×10^{-8}	2.8×10^{-8}	2.0×10^{-8}	1.6×10^{-8}	1.4×10^{-8}			
Cf-246	1.49 d	M	0.005	1.7×10^{-6}	5.0×10^{-4}	1.3×10^{-6}	8.3×10^{-7}	6.1×10^{-7}	5.7×10^{-7}	4.5×10^{-7}			
Cf-248	334 d	M	0.005	3.8×10^{-5}	5.0×10^{-4}	3.2×10^{-5}	2.1×10^{-5}	1.4×10^{-5}	1.0×10^{-5}	8.8×10^{-6}			
Cf-249	3.50×10^2 a	M	0.005	1.6×10^{-4}	5.0×10^{-4}	1.5×10^{-4}	1.1×10^{-4}	8.0×10^{-5}	7.2×10^{-5}	7.0×10^{-5}			
Cf-250	13.1 a	M	0.005	1.1×10^{-4}	5.0×10^{-4}	9.8×10^{-5}	6.6×10^{-5}	4.2×10^{-5}	3.5×10^{-5}	3.4×10^{-5}			
Cf-251	8.98×10^2 a	M	0.005	1.6×10^{-4}	5.0×10^{-4}	1.5×10^{-4}	1.1×10^{-4}	8.1×10^{-5}	7.3×10^{-5}	7.1×10^{-5}			
Cf-252	2.64 a	M	0.005	9.7×10^{-5}	5.0×10^{-4}	8.7×10^{-5}	5.6×10^{-5}	3.2×10^{-5}	2.2×10^{-5}	2.0×10^{-5}			
Cf-253	17.8 d	M	0.005	5.4×10^{-6}	5.0×10^{-4}	4.2×10^{-6}	2.6×10^{-6}	1.9×10^{-6}	1.7×10^{-6}	1.3×10^{-6}			
Cf-254	60.5 d	M	0.005	2.5×10^{-4}	5.0×10^{-4}	1.9×10^{-4}	1.1×10^{-4}	7.0×10^{-5}	4.8×10^{-5}	4.1×10^{-5}			
Einsteinium													
Es-250	2.10 h	M	0.005	2.0×10^{-9}	5.0×10^{-4}	1.8×10^{-9}	1.2×10^{-9}	7.8×10^{-10}	6.4×10^{-10}	6.3×10^{-10}			
Es-251	1.38 d	M	0.005	7.9×10^{-9}	5.0×10^{-4}	6.0×10^{-9}	3.9×10^{-9}	2.8×10^{-9}	2.6×10^{-9}	2.1×10^{-9}			
Es-253	20.5 d	M	0.005	1.1×10^{-5}	5.0×10^{-4}	8.0×10^{-6}	5.1×10^{-6}	3.7×10^{-6}	3.4×10^{-6}	2.7×10^{-6}			

TABLE III.2E. MEMBERS OF THE PUBLIC: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ VIA INHALATION (cont.)

Radionuclide ^a	Physical half-life	Age ≤ 1 a		f_1 for $g > 1$ a	Age 1–2 a		2–7 a		7–12 a		12–17 a		>17 a
		Type	f_1		$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$	$e(g)$			
Es-254	276 d	M	0.005	3.7×10^{-5}	5.0×10^{-4}	3.1×10^{-5}	2.0×10^{-5}	1.3×10^{-5}	6.3×10^{-7}	1.0×10^{-5}	5.9×10^{-7}	8.6×10^{-6}	
Es-254m	1.64 d	M	0.005	1.7×10^{-6}	5.0×10^{-4}	1.3×10^{-6}	8.4×10^{-7}	6.3×10^{-7}				4.7×10^{-7}	
Fermium													
Fm-252	22.7 h	M	0.005	1.2×10^{-6}	5.0×10^{-4}	9.0×10^{-7}	5.8×10^{-7}	4.3×10^{-7}	4.0×10^{-7}	4.0×10^{-7}	4.0×10^{-7}	3.2×10^{-7}	
Fm-253	3.00 d	M	0.005	1.5×10^{-6}	5.0×10^{-4}	1.2×10^{-6}	7.3×10^{-7}	5.4×10^{-7}	5.0×10^{-7}	5.0×10^{-7}	5.0×10^{-7}	4.0×10^{-7}	
Fm-254	3.24 h	M	0.005	3.2×10^{-7}	5.0×10^{-4}	2.3×10^{-7}	1.3×10^{-7}	9.8×10^{-8}	7.6×10^{-8}	7.6×10^{-8}	7.6×10^{-8}	6.1×10^{-8}	
Fm-255	20.1 h	M	0.005	1.2×10^{-6}	5.0×10^{-4}	7.3×10^{-7}	4.7×10^{-7}	3.5×10^{-7}	3.4×10^{-7}	3.4×10^{-7}	3.4×10^{-7}	2.7×10^{-7}	
Fm-257	101 d	M	0.005	3.3×10^{-5}	5.0×10^{-4}	2.6×10^{-5}	1.6×10^{-5}	1.1×10^{-5}	8.8×10^{-6}	8.8×10^{-6}	8.8×10^{-6}	7.1×10^{-6}	
Mendelevium													
Md-257	5.20 h	M	0.005	1.0×10^{-7}	5.0×10^{-4}	8.2×10^{-8}	5.1×10^{-8}	3.6×10^{-8}	3.1×10^{-8}	3.1×10^{-8}	3.1×10^{-8}	2.5×10^{-8}	
Md-258	55.0 d	M	0.005	2.4×10^{-5}	5.0×10^{-4}	1.9×10^{-5}	1.2×10^{-5}	8.6×10^{-6}	7.3×10^{-6}	7.3×10^{-6}	7.3×10^{-6}	5.9×10^{-6}	

^a m and m' denote metastable states of the radionuclide. The metastable state m' is of higher energy than the metastable state m.

^b The f_1 value for calcium for 1–15 year olds for Type F is 0.4.

^c The f_1 value for iron for 1–15 year olds for Type F is 0.2.

^d The f_1 value for cobalt for 1–15 year olds for Type F is 0.3.

^e The f_1 value for strontium for 1–15 year olds for Type F is 0.4.

^f The f_1 value for barium for 1–15 year olds for Type F is 0.3.

^g The f_1 value for lead for 1–15 year olds for Type F is 0.4.

^h The f_1 value for radium for 1–15 year olds for Type F is 0.3.

Note: Types F, M and S denote fast, moderate and slow absorption from the lung, respectively. f_1 : gut transfer factor; $e(g)$: effective dose per unit intake by age group.

TABLE III.2F. LUNG ABSORPTION TYPES USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR EXPOSURE DUE TO PARTICULATE AEROSOLS OR DUE TO GASES AND VAPOURS FOR MEMBERS OF THE PUBLIC

Element	Absorption type(s) ^a	ICRP Publication No. for details of biokinetic model and absorption type(s)
Hydrogen	F, M ^b , S, G	Publications 56 [31], 67 [32] and 71 [33]
Beryllium	M, S	Publication 30, Part 3 [34]
Carbon	F, M ^b , S, G	Publications 56 [31], 67 [32] and 71 [33]
Fluorine	F, M, S	Publication 30, Part 2 [34]
Sodium	F	Publication 30, Part 2 [34]
Magnesium	F, M	Publication 30, Part 3 [34]
Aluminium	F, M	Publication 30, Part 3 [34]
Silicon	F, M, S	Publication 30, Part 3 [34]
Phosphorus	F, M	Publication 30, Part 1 [34]
Sulphur	F, M ^b , S, G	Publications 67 [32] and 71 [33]
Chlorine	F, M	Publication 30, Part 2 [34]
Potassium	F	Publication 30, Part 2 [34]
Calcium	F, M, S	Publication 71 [33]
Scandium	S	Publication 30, Part 3 [34]
Titanium	F, M, S	Publication 30, Part 3 [34]
Vanadium	F, M	Publication 30, Part 3 [34]
Chromium	F, M, S	Publication 30, Part 2 [34]
Manganese	F, M	Publication 30, Part 1 [34]
Iron	F, M ^b , S	Publications 69 [35] and 71 [33]
Cobalt	F, M ^b , S	Publications 67 [32] and 71 [33]
Nickel	F, M ^b , S, G	Publications 67 [32] and 71 [33]
Copper	F, M, S	Publication 30, Part 2 [34]
Zinc	F, M ^b , S	Publications 67 [32] and 71 [33]
Gallium	F, M	Publication 30, Part 3 [34]

TABLE III.2F. LUNG ABSORPTION TYPES USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR EXPOSURE DUE TO PARTICULATE AEROSOLS OR DUE TO GASES AND VAPOURS FOR MEMBERS OF THE PUBLIC (cont.)

Element	Absorption type(s) ^a	ICRP Publication No. for details of biokinetic model and absorption type(s)
Germanium	F, M	Publication 30, Part 3 [34]
Arsenic	M	Publication 30, Part 3 [34]
Selenium	F ^b , M, S	Publications 69 [35] and 71 [33]
Bromine	F, M	Publication 30, Part 2 [34]
Rubidium	F	Publication 30, Part 2 [34]
Strontium	F, M ^b , S	Publications 67 [32] and 71 [33]
Yttrium	M, S	Publication 30, Part 2 [34]
Zirconium	F, M ^b , S	Publications 56 [31], 67 [32] and 71 [33]
Niobium	F, M ^b , S	Publications 56 [31], 67 [32] and 71 [33]
Molybdenum	F, M ^b , S	Publications 67 [32] and 71 [33]
Technetium	F, M ^b , S	Publications 67 [32] and 71 [33]
Ruthenium	F, M ^b , S, G	Publications 56 [31], 67 [32] and 71 [33]
Rhodium	F, M, S	Publication 30, Part 2 [34]
Palladium	F, M, S	Publication 30, Part 3 [34]
Silver	F, M ^b , S	Publications 67 [32] and 71 [33]
Cadmium	F, M, S	Publication 30, Part 2 [34]
Indium	F, M	Publication 30, Part 2 [34]
Tin	F, M	Publication 30, Part 3 [34]
Antimony	F, M ^b , S	Publications 69 [35] and 71 [33]
Tellurium	F, M ^b , S, G	Publications 67 [32] and 71 [33]
Iodine	F ^b , M, S, G	Publications 56 [31], 67 [32] and 71 [33]
Caesium	F ^b , M, S	Publications 56 [31], 67 [32] and 71 [33]
Barium	F, M ^b , S	Publications 67 [32] and 71 [33]
Lanthanum	F, M	Publication 30, Part 3 [34]

TABLE III.2F. LUNG ABSORPTION TYPES USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR EXPOSURE DUE TO PARTICULATE AEROSOLS OR DUE TO GASES AND VAPOURS FOR MEMBERS OF THE PUBLIC (cont.)

Element	Absorption type(s) ^a	ICRP Publication No. for details of biokinetic model and absorption type(s)
Cerium	F, M ^b , S	Publications 56 [31], 67 [32] and 71 [33]
Praseodymium	M, S	Publication 30, Part 3 [34]
Neodymium	M, S	Publication 30, Part 3 [34]
Promethium	M, S	Publication 30, Part 3 [34]
Samarium	M	Publication 30, Part 3 [34]
Europium	M	Publication 30, Part 3 [34]
Gadolinium	F, M	Publication 30, Part 3 [34]
Terbium	M	Publication 30, Part 3 [34]
Dysprosium	M	Publication 30, Part 3 [34]
Holmium	M	Publication 30, Part 3 [34]
Erbium	M	Publication 30, Part 3 [34]
Thulium	M	Publication 30, Part 3 [34]
Ytterbium	M, S	Publication 30, Part 3 [34]
Lutetium	M, S	Publication 30, Part 3 [34]
Hafnium	F, M	Publication 30, Part 3 [34]
Tantalum	M, S	Publication 30, Part 3 [34]
Tungsten	F	Publication 30, Part 3 [34]
Rhenium	F, M	Publication 30, Part 2 [34]
Osmium	F, M, S	Publication 30, Part 2 [34]
Iridium	F, M, S	Publication 30, Part 2 [34]
Platinum	F	Publication 30, Part 3 [34]
Gold	F, M, S	Publication 30, Part 2 [34]
Mercury	F, M, G	Publication 30, Part 2 [34]
Thallium	F	Publication 30, Part 3 [34]

TABLE III.2F. LUNG ABSORPTION TYPES USED TO CALCULATE COMMITTED EFFECTIVE DOSE PER UNIT INTAKE VIA INHALATION FOR EXPOSURE DUE TO PARTICULATE AEROSOLS OR DUE TO GASES AND VAPOURS FOR MEMBERS OF THE PUBLIC (cont.)

Element	Absorption type(s) ^a	ICRP Publication No. for details of biokinetic model and absorption type(s)
Lead	F, M ^b , S, G	Publications 67 [32] and 71 [33]
Bismuth	F, M	Publication 30, Part 2 [34]
Polonium	F, M ^b , S, G	Publications 67 [32] and 71 [33]
Astatine	F, M	Publication 30, Part 3 [34]
Francium	F	Publication 30, Part 3 [34]
Radium	F, M ^b , S	Publications 67 [32] and 71 [33]
Actinium	F, M, S	Publication 30, Part 3 [34]
Thorium	F, M, S ^b	Publications 69 [35] and 71 [33]
Protactinium	M, S	Publication 30, Part 3 [34]
Uranium	F, M ^b , S	Publications 69 [35] and 71 [33]
Neptunium	F, M ^b , S	Publications 67 [32] and 71 [33]
Plutonium	F, M ^b , S	Publications 67 [32] and 71 [33]
Americium	F, M ^b , S	Publications 67 [32] and 71 [33]
Curium	F, M ^b , S	Publication 71 [33]
Berkelium	M	Publication 30, Part 4 [34]
Californium	M	Publication 30, Part 4 [34]
Einsteinium	M	Publication 30, Part 4 [34]
Fermium	M	Publication 30, Part 4 [34]
Mendelevium	M	Publication 30, Part 4 [34]

^a For particulates: F: fast; M: moderate; S: slow; G: gases and vapours.
^b Recommended default absorption type for particulate aerosol when no specific information is available (see ICRP Publication No. 71 [31]).

TABLE III.2G. INHALATION: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ (Sv/Bq) FOR SOLUBLE OR REACTIVE GASES AND VAPOURS

Radionuclide	Physical half-life	Absorption ^a	% deposits	Age ≤ 1 a		f_1 for $g > 1$ a	Age ≤ 1 a		$e(g)$	7–12 a		12–17 a		$e(g)^b$
				f_1	$e(g)$		$e(g)$	$e(g)$		$e(g)$	$e(g)$	$e(g)$	$e(g)$	
Tritiated water	12.3 a	V	100	1.000	6.4×10^{-11}	1.000	4.8×10^{-11}	3.1×10^{-11}	2.3×10^{-11}	1.8×10^{-11}	1.8×10^{-11}	1.8×10^{-11}	1.8×10^{-11}	
Elemental hydrogen	12.3 a	V	0.01	1.000	6.4×10^{-15}	1.000	4.8×10^{-15}	3.1×10^{-15}	2.3×10^{-15}	1.8×10^{-15}	1.8×10^{-15}	1.8×10^{-15}	1.8×10^{-15}	
Tritiated methane	12.3 a	V	1	1.000	6.4×10^{-13}	1.000	4.8×10^{-13}	3.1×10^{-13}	2.3×10^{-13}	1.8×10^{-13}	1.8×10^{-13}	1.8×10^{-13}	1.8×10^{-13}	
Organically bound tritium	12.3 a	V	100	1.000	1.1×10^{-10}	1.000	1.1×10^{-10}	7.0×10^{-11}	5.5×10^{-11}	4.1×10^{-11}	4.1×10^{-11}	4.1×10^{-11}	4.1×10^{-11}	
Carbon-11 vapour	0.340 h	V	100	1.000	2.8×10^{-11}	1.000	1.8×10^{-11}	9.7×10^{-12}	6.1×10^{-12}	3.8×10^{-12}	3.8×10^{-12}	3.2×10^{-12}	3.2×10^{-12}	
Carbon-11 dioxide	0.340 h	V	100	1.000	1.8×10^{-11}	1.000	1.2×10^{-11}	6.5×10^{-12}	4.1×10^{-12}	2.5×10^{-12}	2.5×10^{-12}	2.2×10^{-12}	2.2×10^{-12}	
Carbon-11 monoxide	0.340 h	V	40	1.000	1.0×10^{-11}	1.000	6.7×10^{-12}	3.5×10^{-12}	2.2×10^{-12}	1.4×10^{-12}	1.4×10^{-12}	1.2×10^{-12}	1.2×10^{-12}	
Carbon-14 vapour	5.73×10^3 a	V	100	1.000	1.3×10^{-9}	1.000	1.6×10^{-9}	9.7×10^{-10}	7.9×10^{-10}	5.7×10^{-10}	5.7×10^{-10}	5.8×10^{-10}	5.8×10^{-10}	
Carbon-14 dioxide	5.73×10^3 a	V	100	1.000	1.9×10^{-11}	1.000	1.9×10^{-11}	1.1×10^{-11}	8.9×10^{-12}	6.3×10^{-12}	6.3×10^{-12}	6.2×10^{-12}	6.2×10^{-12}	
Carbon-14 monoxide	5.73×10^3 a	V	40	1.000	9.1×10^{-12}	1.000	5.7×10^{-12}	2.8×10^{-12}	1.7×10^{-12}	9.9×10^{-13}	9.9×10^{-13}	8.0×10^{-13}	8.0×10^{-13}	
Carbon disulphide-35	87.4 d	F	100	1.000	6.9×10^{-9}	0.800	4.8×10^{-9}	2.4×10^{-9}	1.4×10^{-9}	8.6×10^{-10}	8.6×10^{-10}	7.0×10^{-10}	7.0×10^{-10}	

TABLE III.2G. INHALATION: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ (Sv/Bq) FOR SOLUBLE OR REACTIVE GASES AND VAPOURS (cont.)

Radionuclide	Physical half-life	Absorption ^a	% deposits	Age $\leq$ 1 a		f_1 for $g > 1$ a	Age $\leq$ 1 a		$e(g)$	7–12 a		$e(g)$	12–17 a		$e(g)^b$
				f_1	$e(g)$		$e(g)$	$e(g)$		$e(g)$	$e(g)$		$e(g)$	$e(g)$	
Ruthenium-105 tetroxide	4.44 h	F	100	0.100	1.6×10^{-9}	0.050	1.0×10^{-9}	5.3×10^{-10}	3.2×10^{-10}	2.2×10^{-10}	2.2×10^{-10}	1.8×10^{-10}	1.8×10^{-10}	1.8×10^{-10}	1.8×10^{-10}
Ruthenium-106 tetroxide	1.01 a	F	100	0.100	1.6×10^{-7}	0.050	1.1×10^{-7}	6.1×10^{-8}	3.7×10^{-8}	2.2×10^{-8}	2.2×10^{-8}	1.8×10^{-8}	1.8×10^{-8}	1.8×10^{-8}	1.8×10^{-8}
Tellurium-116 vapour	2.49 h	F	100	0.600	5.9×10^{-10}	0.300	4.4×10^{-10}	2.5×10^{-10}	1.6×10^{-10}	1.1×10^{-10}	1.1×10^{-10}	8.7×10^{-11}	8.7×10^{-11}	8.7×10^{-11}	8.7×10^{-11}
Tellurium-121 vapour	17.0 d	F	100	0.600	3.0×10^{-9}	0.300	2.4×10^{-9}	1.4×10^{-9}	9.6×10^{-10}	6.7×10^{-10}	6.7×10^{-10}	5.1×10^{-10}	5.1×10^{-10}	5.1×10^{-10}	5.1×10^{-10}
Tellurium-121m vapour	154 d	F	100	0.600	3.5×10^{-8}	0.300	2.7×10^{-8}	1.6×10^{-8}	9.8×10^{-9}	6.6×10^{-9}	6.6×10^{-9}	5.5×10^{-9}	5.5×10^{-9}	5.5×10^{-9}	5.5×10^{-9}
Tellurium-123 vapour	1.00×10^{13} a	F	100	0.600	2.8×10^{-8}	0.300	2.5×10^{-8}	1.9×10^{-8}	1.5×10^{-8}	1.3×10^{-8}	1.3×10^{-8}	1.2×10^{-8}	1.2×10^{-8}	1.2×10^{-8}	1.2×10^{-8}
Tellurium-123m vapour	120 d	F	100	0.600	2.5×10^{-8}	0.300	1.8×10^{-8}	1.0×10^{-8}	5.7×10^{-9}	3.5×10^{-9}	3.5×10^{-9}	2.9×10^{-9}	2.9×10^{-9}	2.9×10^{-9}	2.9×10^{-9}
Tellurium-125m vapour	58.0 d	F	100	0.600	1.5×10^{-8}	0.300	1.1×10^{-8}	5.9×10^{-9}	3.2×10^{-9}	1.9×10^{-9}	1.9×10^{-9}	1.5×10^{-9}	1.5×10^{-9}	1.5×10^{-9}	1.5×10^{-9}
Tellurium-127 vapour	9.35 h	F	100	0.600	6.1×10^{-10}	0.300	4.4×10^{-10}	2.3×10^{-10}	1.4×10^{-10}	9.2×10^{-11}	9.2×10^{-11}	7.7×10^{-11}	7.7×10^{-11}	7.7×10^{-11}	7.7×10^{-11}
Tellurium-127m vapour	109 d	F	100	0.600	5.3×10^{-8}	0.300	3.7×10^{-8}	1.9×10^{-8}	1.0×10^{-8}	6.1×10^{-9}	6.1×10^{-9}	4.6×10^{-9}	4.6×10^{-9}	4.6×10^{-9}	4.6×10^{-9}

TABLE III.2G. INHALATION: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ (Sv/Bq) FOR SOLUBLE OR REACTIVE GASES AND VAPOURS (cont.)

Radionuclide	Physical half-life	Absorption ^a	% deposits	Age $\leq$ 1 a		f_1 for $g > 1$ a	Age $\leq$ 1 a		$e(g)$	7–12 a		$e(g)$	12–17 a		$e(g)^b$
				f_1	$e(g)$		$e(g)$	$e(g)$		$e(g)$	$e(g)$		$e(g)$	$e(g)$	
Tellurium-129 vapour	1.16 h	F	100	0.600	2.5×10^{-10}	0.300	1.7×10^{-10}	9.4×10^{-11}	6.2×10^{-11}	6.2×10^{-11}	4.3×10^{-11}	4.3×10^{-11}	3.7×10^{-11}	3.7×10^{-11}	
Tellurium-129m vapour	33.6 d	F	100	0.600	4.8×10^{-8}	0.300	3.2×10^{-8}	1.6×10^{-8}	8.5×10^{-9}	8.5×10^{-9}	5.1×10^{-9}	5.1×10^{-9}	3.7×10^{-9}	3.7×10^{-9}	
Tellurium-131 vapour	0.417 h	F	100	0.600	5.1×10^{-10}	0.300	4.5×10^{-10}	2.6×10^{-10}	1.4×10^{-10}	1.4×10^{-10}	9.5×10^{-11}	9.5×10^{-11}	6.8×10^{-11}	6.8×10^{-11}	
Tellurium-131m vapour	1.25 d	F	100	0.600	2.1×10^{-8}	0.300	1.9×10^{-8}	1.1×10^{-8}	5.6×10^{-9}	5.6×10^{-9}	3.7×10^{-9}	3.7×10^{-9}	2.4×10^{-9}	2.4×10^{-9}	
Tellurium-132 vapour	3.26 d	F	100	0.600	5.4×10^{-8}	0.300	4.5×10^{-8}	2.4×10^{-8}	1.2×10^{-8}	1.2×10^{-8}	7.6×10^{-9}	7.6×10^{-9}	5.1×10^{-9}	5.1×10^{-9}	
Tellurium-133 vapour	0.207 h	F	100	0.600	5.5×10^{-10}	0.300	4.7×10^{-10}	2.5×10^{-10}	1.2×10^{-10}	1.2×10^{-10}	8.1×10^{-11}	8.1×10^{-11}	5.6×10^{-11}	5.6×10^{-11}	
Tellurium-133m vapour	0.923 h	F	100	0.600	2.3×10^{-9}	0.300	2.0×10^{-9}	1.1×10^{-9}	5.0×10^{-10}	5.0×10^{-10}	3.3×10^{-10}	3.3×10^{-10}	2.2×10^{-10}	2.2×10^{-10}	
Tellurium-134 vapour	0.696 h	F	100	0.600	6.8×10^{-10}	0.300	5.5×10^{-10}	3.0×10^{-10}	1.6×10^{-10}	1.6×10^{-10}	1.1×10^{-10}	1.1×10^{-10}	8.4×10^{-11}	8.4×10^{-11}	
Elemental iodine-120	1.35 h	V	100	1.000	3.0×10^{-9}	1.000	2.4×10^{-9}	1.3×10^{-9}	6.4×10^{-10}	6.4×10^{-10}	4.3×10^{-10}	4.3×10^{-10}	3.0×10^{-1}	3.0×10^{-1}	
Elemental iodine-120m	0.883 h	V	100	1.000	1.5×10^{-9}	1.000	1.2×10^{-9}	6.4×10^{-10}	3.4×10^{-10}	3.4×10^{-10}	2.3×10^{-10}	2.3×10^{-10}	1.8×10^{-10}	1.8×10^{-10}	

TABLE III.2G. INHALATION: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ (Sv/Bq) FOR SOLUBLE OR REACTIVE GASES AND VAPOURS (cont.)

Radionuclide	Physical half-life	Absorption ^a	% deposits	Age $\leq$ 1 a		f_1 for $g > 1$ a	Age $\leq$ 1 a		2–7 a	7–12 a	12–17 a	>17 a
				f_1	$e(g)$		$e(g)$	$e(g)^b$				
Elemental iodine-121	2.12 h	V	100	1.000	5.7×10^{-10}	1.000	5.1×10^{-10}	3.0×10^{-10}	1.7×10^{-10}	1.2×10^{-10}	8.6×10^{-11}	
Elemental iodine-123	13.2 h	V	100	1.000	2.1×10^{-9}	1.000	1.8×10^{-9}	1.0×10^{-9}	4.7×10^{-10}	3.2×10^{-10}	2.1×10^{-10}	
Elemental iodine-124	4.18 d	V	100	1.000	1.1×10^{-7}	1.000	1.0×10^{-7}	5.8×10^{-8}	2.8×10^{-8}	1.8×10^{-8}	1.2×10^{-8}	
Elemental iodine-125	60.1 d	V	100	1.000	4.7×10^{-8}	1.000	5.2×10^{-8}	3.7×10^{-8}	2.8×10^{-8}	2.0×10^{-8}	1.4×10^{-8}	
Elemental iodine-126	13.0 d	V	100	1.000	1.9×10^{-7}	1.000	1.9×10^{-7}	1.1×10^{-7}	6.2×10^{-8}	4.1×10^{-8}	2.6×10^{-8}	
Elemental iodine-128	0.416 h	V	100	1.000	4.2×10^{-10}	1.000	2.8×10^{-10}	1.6×10^{-10}	1.0×10^{-10}	7.5×10^{-11}	6.5×10^{-11}	
Elemental iodine-129	1.57×10^7 a	V	100	1.000	1.7×10^{-7}	1.000	2.0×10^{-7}	1.6×10^{-7}	1.7×10^{-7}	1.3×10^{-7}	9.6×10^{-8}	
Elemental iodine-130	12.4 h	V	100	1.000	1.9×10^{-8}	1.000	1.7×10^{-8}	9.2×10^{-9}	4.3×10^{-9}	2.8×10^{-9}	1.9×10^{-9}	
Elemental iodine-131	8.04 d	V	100	1.000	1.7×10^{-7}	1.000	1.6×10^{-7}	9.4×10^{-8}	4.8×10^{-8}	3.1×10^{-8}	2.0×10^{-8}	
Elemental iodine-132	2.30 h	V	100	1.000	2.8×10^{-9}	1.000	2.3×10^{-9}	1.3×10^{-9}	6.4×10^{-10}	4.3×10^{-10}	3.1×10^{-10}	

TABLE III.2G. INHALATION: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ (Sv/Bq) FOR SOLUBLE OR REACTIVE GASES AND VAPOURS (cont.)

Radionuclide	Physical half-life	Absorption ^a	% deposits	Age ≤ 1 a		<i>f</i> ₁ for <i>g</i> > 1 a	Age ≤ 1 a		2–7 a	7–12 a	12–17 a	>17 a
				<i>f</i> ₁	<i>e</i> (<i>g</i>)		<i>e</i> (<i>g</i>)	<i>e</i> (<i>g</i>)				
Elemental iodine-132m	1.39 h	V	100	1.000	2.4 × 10 ^{−9}	1.000	2.1 × 10 ^{−9}	1.1 × 10 ^{−9}	5.6 × 10 ^{−10}	3.8 × 10 ^{−10}	2.7 × 10 ^{−10}	
Elemental iodine-133	20.8 h	V	100	1.000	4.5 × 10 ^{−8}	1.000	4.1 × 10 ^{−8}	2.1 × 10 ^{−8}	9.7 × 10 ^{−9}	6.3 × 10 ^{−9}	4.0 × 10 ^{−9}	
Elemental iodine-134	0.876 h	V	100	1.000	8.7 × 10 ^{−10}	1.000	6.9 × 10 ^{−10}	3.9 × 10 ^{−10}	2.2 × 10 ^{−10}	1.6 × 10 ^{−10}	1.5 × 10 ^{−10}	
Elemental iodine-135	6.61 h	V	100	1.000	9.7 × 10 ^{−9}	1.000	8.5 × 10 ^{−9}	4.5 × 10 ^{−9}	2.1 × 10 ^{−9}	1.4 × 10 ^{−9}	9.2 × 10 ^{−10}	
Methyl iodide-120	1.35 h	V	70	1.000	2.3 × 10 ^{−9}	1.000	1.9 × 10 ^{−9}	1.0 × 10 ^{−9}	4.8 × 10 ^{−10}	3.1 × 10 ^{−10}	2.0 × 10 ^{−10}	
Methyl iodide-120m	0.883 h	V	70	1.000	1.0 × 10 ^{−9}	1.000	8.7 × 10 ^{−10}	4.6 × 10 ^{−10}	2.2 × 10 ^{−10}	1.5 × 10 ^{−10}	1.0 × 10 ^{−10}	
Methyl iodide-121	2.12 h	V	70	1.000	4.2 × 10 ^{−10}	1.000	3.8 × 10 ^{−10}	2.2 × 10 ^{−10}	1.2 × 10 ^{−10}	8.3 × 10 ^{−11}	5.6 × 10 ^{−11}	
Methyl iodide-123	13.2 h	V	70	1.000	1.6 × 10 ^{−9}	1.000	1.4 × 10 ^{−9}	7.7 × 10 ^{−10}	3.6 × 10 ^{−10}	2.4 × 10 ^{−10}	1.5 × 10 ^{−10}	
Methyl iodide-124	4.18 d	V	70	1.000	8.5 × 10 ^{−8}	1.000	8.0 × 10 ^{−8}	4.5 × 10 ^{−8}	2.2 × 10 ^{−8}	1.4 × 10 ^{−8}	9.2 × 10 ^{−9}	
Methyl iodide-125	60.1 d	V	70	1.000	3.7 × 10 ^{−8}	1.000	4.0 × 10 ^{−8}	2.9 × 10 ^{−8}	2.2 × 10 ^{−8}	1.6 × 10 ^{−8}	1.1 × 10 ^{−8}	
Methyl iodide-126	13.0 d	V	70	1.000	1.5 × 10 ^{−7}	1.000	1.5 × 10 ^{−7}	9.0 × 10 ^{−8}	4.8 × 10 ^{−8}	3.2 × 10 ^{−8}	2.0 × 10 ^{−8}	
Methyl iodide-128	0.416 h	V	70	1.000	1.5 × 10 ^{−10}	1.000	1.2 × 10 ^{−10}	6.3 × 10 ^{−11}	3.0 × 10 ^{−11}	1.9 × 10 ^{−11}	1.3 × 10 ^{−11}	
Methyl iodide-129	1.57 × 10 ⁷ a	V	70	1.000	1.3 × 10 ^{−7}	1.000	1.5 × 10 ^{−7}	1.2 × 10 ^{−7}	1.3 × 10 ^{−7}	9.9 × 10 ^{−8}	7.4 × 10 ^{−8}	

TABLE III.2G. INHALATION: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ (Sv/Bq) FOR SOLUBLE OR REACTIVE GASES AND VAPOURS (cont.)

Radionuclide	Physical half-life	Absorption ^a	% deposits	Age ≤ 1 a		<i>f</i> ₁ for g > 1 a	Age ≤ 1 a		<i>e</i> (g)	7–12 a		12–17 a		<i>e</i> (g) ^b
				<i>f</i> ₁	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)		<i>e</i> (g)	<i>e</i> (g)			
Methyl iodide-130	12.4 h	V	70	1.000	1.5 × 10 ⁻⁸	1.000	1.3 × 10 ⁻⁸	7.2 × 10 ⁻⁹	3.3 × 10 ⁻⁹	3.3 × 10 ⁻⁹	2.2 × 10 ⁻⁹	2.2 × 10 ⁻⁹	1.4 × 10 ⁻⁹	
Methyl iodide-131	8.04 d	V	70	1.000	1.3 × 10 ⁻⁷	1.000	1.3 × 10 ⁻⁷	7.4 × 10 ⁻⁸	3.7 × 10 ⁻⁸	3.7 × 10 ⁻⁸	2.4 × 10 ⁻⁸	2.4 × 10 ⁻⁸	1.5 × 10 ⁻⁸	
Methyl iodide-132	2.30 h	V	70	1.000	2.0 × 10 ⁻⁹	1.000	1.8 × 10 ⁻⁹	9.5 × 10 ⁻¹⁰	4.4 × 10 ⁻¹⁰	4.4 × 10 ⁻¹⁰	2.9 × 10 ⁻¹⁰	2.9 × 10 ⁻¹⁰	1.9 × 10 ⁻¹⁰	
Methyl iodide-132m	1.39 h	V	70	1.000	1.8 × 10 ⁻⁹	1.000	1.6 × 10 ⁻⁹	8.3 × 10 ⁻¹⁰	3.9 × 10 ⁻¹⁰	3.9 × 10 ⁻¹⁰	2.5 × 10 ⁻¹⁰	2.5 × 10 ⁻¹⁰	1.6 × 10 ⁻¹⁰	
Methyl iodide-133	20.8 h	V	70	1.000	3.5 × 10 ⁻⁸	1.000	3.2 × 10 ⁻⁸	1.7 × 10 ⁻⁸	7.6 × 10 ⁻⁹	7.6 × 10 ⁻⁹	4.9 × 10 ⁻⁹	4.9 × 10 ⁻⁹	3.1 × 10 ⁻⁹	
Methyl iodide-134	0.876 h	V	70	1.000	5.1 × 10 ⁻¹⁰	1.000	4.3 × 10 ⁻¹⁰	2.3 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰	1.1 × 10 ⁻¹⁰	7.4 × 10 ⁻¹¹	7.4 × 10 ⁻¹¹	5.0 × 10 ⁻¹¹	
Methyl iodide-135	6.61 h	V	70	1.000	7.5 × 10 ⁻⁹	1.000	6.7 × 10 ⁻⁹	3.5 × 10 ⁻⁹	1.6 × 10 ⁻⁹	1.6 × 10 ⁻⁹	1.1 × 10 ⁻⁹	1.1 × 10 ⁻⁹	6.8 × 10 ⁻¹⁰	
Mercury-193 vapour	3.50 h	^d	70	1.000	4.2 × 10 ⁻⁹	1.000	3.4 × 10 ⁻⁹	2.2 × 10 ⁻⁹	1.6 × 10 ⁻⁹	1.6 × 10 ⁻⁹	1.2 × 10 ⁻⁹	1.2 × 10 ⁻⁹	1.1 × 10 ⁻⁹	
Mercury-193m vapour	11.1 h	^d	70	1.000	1.2 × 10 ⁻⁸	1.000	9.4 × 10 ⁻⁹	6.1 × 10 ⁻⁹	4.5 × 10 ⁻⁹	4.5 × 10 ⁻⁹	3.4 × 10 ⁻⁹	3.4 × 10 ⁻⁹	3.1 × 10 ⁻⁹	
Mercury-194 vapour	2.60 × 10 ² a	^d	70	1.000	9.4 × 10 ⁻⁸	1.000	8.3 × 10 ⁻⁸	6.2 × 10 ⁻⁸	5.0 × 10 ⁻⁸	5.0 × 10 ⁻⁸	4.3 × 10 ⁻⁸	4.3 × 10 ⁻⁸	4.0 × 10 ⁻⁸	
Mercury-195 vapour	9.90 h	^d	70	1.000	5.3 × 10 ⁻⁹	1.000	4.3 × 10 ⁻⁹	2.8 × 10 ⁻⁹	2.1 × 10 ⁻⁹	2.1 × 10 ⁻⁹	1.6 × 10 ⁻⁹	1.6 × 10 ⁻⁹	1.4 × 10 ⁻⁹	
Mercury-195m vapour	1.73 d	^d	70	1.000	3.0 × 10 ⁻⁸	1.000	2.5 × 10 ⁻⁸	1.6 × 10 ⁻⁸	1.2 × 10 ⁻⁸	1.2 × 10 ⁻⁸	8.8 × 10 ⁻⁹	8.8 × 10 ⁻⁹	8.2 × 10 ⁻⁹	

TABLE III.2G. INHALATION: COMMITTED EFFECTIVE DOSE PER UNIT INTAKE $e(g)$ (Sv/Bq) FOR SOLUBLE OR REACTIVE GASES AND VAPOURS (cont.)

Radionuclide	Physical half-life	Absorption ^a	% deposits	Age $\leq$ 1 a		f_1 for $g > 1$ a	Age $\leq$ 1 a		2–7 a		7–12 a		12–17 a		>17 a	
				f_1	$e(g)$		$e(g)$		$e(g)$		$e(g)$		$e(g)$		$e(g)^b$	
Mercury-197 vapour	2.67 d	^d	70	1.000	1.6×10^{-8}	1.000	1.3×10^{-8}		8.4×10^{-9}		6.3×10^{-9}		4.7×10^{-9}		4.4×10^{-9}	
Mercury-197m vapour	23.8 h	^d	70	1.000	2.1×10^{-8}	1.000	1.7×10^{-8}		1.1×10^{-8}		8.2×10^{-9}		6.2×10^{-9}		5.8×10^{-9}	
Mercury-199m vapour	0.710 h	^d	70	1.000	6.5×10^{-10}	1.000	5.3×10^{-10}		3.4×10^{-10}		2.5×10^{-10}		1.9×10^{-10}		1.8×10^{-10}	
Mercury-203 vapour	46.6 d	^d	70	1.000	3.0×10^{-8}	1.000	2.3×10^{-8}		1.5×10^{-8}		1.0×10^{-8}		7.7×10^{-9}		7.0×10^{-9}	

^a F: fast; V: material is taken to be completely and instantaneously transferred to body fluids.

^b Applicable to both workers and adult members of the public.

^c Deposition: 30%:10%:20%:40% (extrathoracic: bronchial: bronchiolar: alveolar-interstitial; 0.1 d retention half-time (see Ref. [36]).

^d Deposition: 10%:20%:40% (bronchial: bronchiolar: alveolar-interstitial; 1.7 d retention half-time (see Ref. [36]).

TABLE III.2H. EFFECTIVE DOSE RATE FOR EXPOSURE DUE TO INERT GASES FOR ADULTS^a

Radionuclide	Physical half-life	Effective dose rate per unit integrated air concentration (Sv · d ⁻¹ /Bq · m ⁻³) ^a
Argon		
Ar-37	35.0 d	4.1×10^{-15}
Ar-39	269 a	1.1×10^{-11}
Ar-41	1.83 h	5.3×10^{-9}
Krypton		
Kr-74	0.192 h	4.5×10^{-9}
Kr-76	14.8 h	1.6×10^{-9}
Kr-77	1.245 h	3.9×10^{-9}
Kr-79	1.46 d	9.7×10^{-10}
Kr-81	2.10×10^5 a	2.1×10^{-11}
Kr-83m	1.83 h	2.1×10^{-13}
Kr-85	10.7 a	2.2×10^{-11}
Kr-85m	4.48 h	5.9×10^{-10}
Kr-87	1.27 h	3.4×10^{-9}
Kr-88	2.84 h	8.4×10^{-9}
Xenon		
Xe-120	0.667 h	1.5×10^{-9}
Xe-121	0.668 h	7.5×10^{-9}
Xe-122	20.1 h	1.9×10^{-10}
Xe-123	2.08 h	2.4×10^{-9}
Xe-125	17.0 h	9.3×10^{-10}
Xe-127	36.4 d	9.7×10^{-10}
Xe-129m	8.0 d	8.1×10^{-11}
Xe-131m	11.9 d	3.2×10^{-11}
Xe-133	5.24 d	1.2×10^{-10}
Xe-133m	2.19 d	1.1×10^{-10}
Xe-135	9.10 h	9.6×10^{-10}
Xe-135m	0.255 h	1.6×10^{-9}
Xe-138	0.237 h	4.7×10^{-9}

^a Applicable to both workers and adult members of the public.

Schedule IV

CRITERIA FOR USE IN EMERGENCY PREPAREDNESS AND RESPONSE

IV.1. Table IV.1 gives generic criteria for doses received within a short period of time for which protective actions and other response actions are expected to be undertaken under any circumstances to avoid or to minimize severe deterministic effects.

IV.2. Table IV.2 gives guidance values for restricting exposure of emergency workers.

TABLE IV.1. GENERIC CRITERIA FOR DOSES RECEIVED WITHIN A SHORT PERIOD OF TIME FOR WHICH PROTECTIVE ACTIONS AND OTHER RESPONSE ACTIONS ARE EXPECTED TO BE UNDERTAKEN UNDER ANY CIRCUMSTANCES TO AVOID OR TO MINIMIZE SEVERE DETERMINISTIC EFFECTS

Acute external exposure (<10 h)		If the dose is projected:
$AD_{\text{red marrow}}^a$	1 Gy	— Take precautionary urgent protective actions immediately (even under difficult conditions) to keep doses below the generic criteria — Provide public information and warnings — Carry out urgent decontamination
AD_{fetus}	0.1 Gy	
AD_{tissue}^b	25 Gy at 0.5 cm	
AD_{skin}^c	10 Gy to 100 cm ²	
Acute internal exposure due to an intake ($\Delta = 30$ d)^d		
$AD(\Delta)_{\text{red marrow}}$	0.2 Gy for radionuclides with atomic number $Z \geq 90^e$ 2 Gy for radionuclides with an atomic number $Z \leq 89^e$	If the dose has been received: — Perform immediate medical examination, consultation and indicated medical treatment
$AD(\Delta)_{\text{thyroid}}$	2 Gy	— Carry out contamination control
$AD(\Delta)_{\text{lung}}^g$	30 Gy	— Carry out immediate decorporation^f (if applicable)
$AD(\Delta)_{\text{colon}}$	20 Gy	— Carry out registration for longer term medical follow-up
$AD(\Delta')_{\text{fetus}}^h$	0.1 Gy	— Provide comprehensive psychological counselling

^a $AD_{\text{red marrow}}$ represents the average relative biological effectiveness (RBE) weighted absorbed dose to internal tissues or organs (e.g. red marrow, lung, small intestine, gonads, thyroid) and to the lens of the eye from exposure in a uniform field of strongly penetrating radiation.

^b Dose delivered to 100 cm² at a depth of 0.5 cm under the body surface in tissue due to close contact with a radioactive source (e.g. source carried in the hand or pocket).

^c The dose is to the 100 cm² dermis (skin structures at a depth of 40 mg/cm² (or 0.4 mm) below the surface).

^d $AD(\Delta)$ is the RBE weighted absorbed dose delivered over a period of time Δ by the intake (I_{05}) that will result in a severe deterministic effect in 5% of exposed individuals. This dose is calculated as described in appendix I of Ref. [29].

^e Different generic criteria are used to take account of the significant difference in RBE weighted absorbed dose from exposure at the intake threshold values specific for these two groups of radionuclides.

- ^f Decorporation is the action of the biological processes, facilitated by chemical or biological agents, by means of which incorporated radionuclides are removed from the human body. The generic criterion for decorporation is based on the projected dose without decorporation.
- ^g For the purposes of these generic criteria, ‘lung’ means the alveolar-interstitial region of the respiratory tract.
- ^h For this particular case, ‘Δ’ means the period of in utero development of the embryo and fetus.

TABLE IV.2. GUIDANCE VALUES FOR RESTRICTING EXPOSURE OF EMERGENCY WORKERS

Tasks	Guidance value ^a
Life saving actions	$H_p(10)^b < 500 \text{ mSv}$ This value may be exceeded under circumstances in which the expected benefits to others clearly outweigh the emergency worker’s own health risks, and the emergency worker volunteers to take the action and understands and accepts these health risks
Actions to prevent severe deterministic effects and actions to prevent the development of catastrophic conditions that could significantly affect people and the environment	$H_p(10) < 500 \text{ mSv}$
Actions to avert a large collective dose	$H_p(10) < 100 \text{ mSv}$

^a These values apply only for the dose from external exposure to strongly penetrating radiation. Doses from external exposure to weakly penetrating radiation and from intake or skin contamination need to be prevented by all possible means. If this is not feasible, the effective dose and the equivalent dose to a tissue or organ that are received have to be limited to minimize the health risk to the individual in line with the risk associated with the guidance values given here.

^b $H_p(10)$ is the personal dose equivalent $H_p(d)$ where $d = 10 \text{ mm}$.

REFERENCES

References are to editions that are current as of the time of publication of these Standards. New editions that supersede these editions may be adopted under national legislation. In the event that the publications referenced here are superseded, please refer to the most recent editions. See also: <http://www-ns.iaea.org/standards/>

- [1] EUROPEAN ATOMIC ENERGY COMMUNITY, FOOD AND AGRICULTURE ORGANIZATION OF THE UNITED NATIONS, INTERNATIONAL ATOMIC ENERGY AGENCY, INTERNATIONAL LABOUR ORGANIZATION, INTERNATIONAL MARITIME ORGANIZATION, OECD NUCLEAR ENERGY AGENCY, PAN AMERICAN HEALTH ORGANIZATION, UNITED NATIONS ENVIRONMENT PROGRAMME, WORLD HEALTH ORGANIZATION, Fundamental Safety Principles, IAEA Safety Standards Series No. SF-1, IAEA, Vienna (2006).
- [2] INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, The 2007 Recommendations of the International Commission on Radiological Protection, Publication 103, Elsevier (2007).
- [3] UNITED NATIONS, Effects of Ionizing Radiation. Vol. I: Report to the General Assembly, Scientific Annexes A and B; Vol. II: Scientific Annexes C, D and E, United Nations Scientific Committee on the Effects of Atomic Radiation (UNSCEAR), UNSCEAR 2006 Report, E.08.IX.6 (2008) and E.09.IX.5 (2009), UN, New York.
- [4] UNITED NATIONS, Sources and Effects of Ionizing Radiation (Report to the General Assembly), Scientific Committee on the Effects of Atomic Radiation (UNSCEAR), UN, New York (2000).
- [5] INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Lung Cancer Risk from Radon and Progeny and Statement on Radon, ICRP Publication 115, Elsevier (2010).
- [6] WORLD HEALTH ORGANIZATION, WHO Handbook on Indoor Radon: A Public Health Perspective, WHO, Geneva (2009).
- [7] INTERNATIONAL COMMISSION ON RADIATION UNITS AND MEASUREMENTS, Quantities and Units in Radiation Protection Dosimetry (Report 51), ICRU, Bethesda, MD (1993).
- [8] INTERNATIONAL ATOMIC ENERGY AGENCY, Governmental, Legal and Regulatory Framework for Safety, IAEA Safety Standards Series No. GSR Part 1, IAEA, Vienna (2010).
- [9] INTERNATIONAL ATOMIC ENERGY AGENCY, Decommissioning of Facilities, IAEA Safety Standards Series No. GSR Part 6, IAEA, Vienna (2014).
- [10] INTERNATIONAL ATOMIC ENERGY AGENCY, Predisposal Management of Radioactive Waste, IAEA Safety Standards Series No. GSR Part 5, IAEA, Vienna (2009).
- [11] INTERNATIONAL ATOMIC ENERGY AGENCY, Disposal of Radioactive Waste, IAEA Safety Standards Series No. SSR-5, IAEA, Vienna (2011).

- [12] INTERNATIONAL ATOMIC ENERGY AGENCY, Regulations for the Safe Transport of Radioactive Material, 2012 Edition, IAEA Safety Standards Series No. SSR-6, IAEA, Vienna (2012).
- [13] INTERNATIONAL ATOMIC ENERGY AGENCY, The Management System for Facilities and Activities, IAEA Safety Standards Series No. GS-R-3, IAEA, Vienna (2006).
- [14] INTERNATIONAL ATOMIC ENERGY AGENCY, Safety Assessment for Facilities and Activities, IAEA Safety Standards Series No. GSR Part 4, IAEA, Vienna (2009).
- [15] FOOD AND AGRICULTURE ORGANIZATION OF THE UNITED NATIONS, INTERNATIONAL ATOMIC ENERGY AGENCY, INTERNATIONAL LABOUR ORGANIZATION, OECD NUCLEAR ENERGY AGENCY, PAN AMERICAN HEALTH ORGANIZATION, UNITED NATIONS OFFICE FOR THE CO-ORDINATION OF HUMANITARIAN AFFAIRS, WORLD HEALTH ORGANIZATION, Preparedness and Response for a Nuclear or Radiological Emergency, IAEA Safety Standards Series No. GS-R-2, IAEA, Vienna (2002).
- [16] INTERNATIONAL ORGANIZATION FOR STANDARDIZATION, Basic Ionizing Radiation Symbol, ISO 361, ISO, Geneva (1975).
- [17] INTERNATIONAL ORGANIZATION FOR STANDARDIZATION, Ionizing-radiation Warning — Supplementary Symbol, ISO 21482, ISO, Geneva (2007).
- [18] Council Directive 96/29 Euratom of 13 May 1996, Laying Down Basic Safety Standards for the Protection of Health of Workers and the General Public against the Dangers Arising from Ionizing Radiation, Official Journal of the European Communities No. L 159, Office for Official Publications of the European Communities, Luxembourg (1996).
- [19] INTERNATIONAL LABOUR ORGANIZATION, Technical and Ethical Guidelines for Workers' Health Surveillance; Occupational Safety and Health Series 72, ILO, Geneva (1998).
- [20] WORLD MEDICAL ASSOCIATION, 18th World Medical Assembly, Helsinki, 1974, as amended by the 59th World Medical Assembly, Seoul, 2008.
- [21] COUNCIL FOR INTERNATIONAL ORGANIZATIONS OF MEDICAL SCIENCES, WORLD HEALTH ORGANIZATION, International Ethical Guidelines for Biomedical Research Involving Human Subjects, CIOMS, Geneva (2002).
- [22] INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Radiological Protection in Biomedical Research, ICRP Publication 62, Pergamon Press, Oxford and New York (1991).
- [23] JOINT FAO/WHO FOOD STANDARDS PROGRAMME, CODEX ALIMENTARIUS COMMISSION, Codex General Standard for Contaminants and Toxins in Foods, Schedule 1 — Radionuclides, CODEX STAN 193-1995, CAC, Rome (2006).
- [24] WORLD HEALTH ORGANIZATION, Guidelines for Drinking-water Quality — 4th edn, WHO, Geneva (2011).
- [25] EUROPEAN COMMISSION, Principles and Methods for Establishing Concentrations and Quantities (Exemption Values) below which Reporting Is not Required in the European Directive, Radiation Protection 65, Office for Official Publications of the European Communities, Luxembourg (1993).

- [26] NATIONAL RADIOLOGICAL PROTECTION BOARD, Exempt Concentrations and Quantities for Radionuclides not Included in the European Basic Safety Standards Directive (MOBBS, S.F., HARVEY, M.P.), NRPB-R306, Chilton (1998).
- [27] INTERNATIONAL ATOMIC ENERGY AGENCY, Derivation of Activity Concentration Values for Exclusion, Exemption and Clearance, Safety Reports Series No. 44, IAEA, Vienna (2005).
- [28] INTERNATIONAL ATOMIC ENERGY AGENCY, Categorization of Radioactive Sources, IAEA Safety Standards Series No. RS-G-1.9, IAEA, Vienna (2005).
- [29] INTERNATIONAL ATOMIC ENERGY AGENCY, Dangerous Quantities of Radioactive Material (D-values), Emergency Preparedness and Response, EPR-D-VALUES 2006, IAEA, Vienna (2006).
- [30] INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Conversion Coefficients for Use in Radiological Protection against External Radiation, ICRP Publication 74, Pergamon Press, Oxford and New York (1997).
- [31] INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Age-dependent Doses to Members of the Public from Intake of Radionuclides — Part 1, Publication 56, Pergamon Press, Oxford and New York (1990).
- [32] INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Age-dependent Doses to Members of the Public from Intake of Radionuclides — Part 2 Ingestion Dose Coefficients, Publication 67, Pergamon Press, Oxford and New York (1992).
- [33] INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Age-dependent Doses to Members of the Public from Intake of Radionuclides — Part 4 Inhalation Dose Coefficients, Publication 71, Pergamon Press, Oxford and New York (1995).
- [34] INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Limits for Intakes of Radionuclides by Workers, Publication 30 (Suppl. B to Part 3), Pergamon Press, Oxford and New York (1982).
- [35] INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Age-dependent Doses to Members of the Public from Intake of Radionuclides — Part 3 Ingestion Dose Coefficients, Publication 69, Pergamon Press, Oxford and New York (1995).
- [36] INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Dose Coefficients for Intakes of Radionuclides by Workers, Publication 68, Pergamon Press, Oxford and New York (1994).
- [37] INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Compendium of Dose Coefficients Based on ICRP Publication 60, ICRP Publication 119, Elsevier (2012).
- [38] INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Individual Monitoring for Internal Exposure of Workers, Publication 78, Pergamon Press, Oxford and New York (1998).

Annex

GENERIC CRITERIA FOR PROTECTIVE ACTIONS AND OTHER RESPONSE ACTIONS TO REDUCE THE RISK OF STOCHASTIC EFFECTS

A-1. Table A-1 provides a set of generic criteria (expressed in terms of projected dose and dose that has been received) for use in the protection strategy that are compatible with reference levels (expressed in terms of residual dose) within a range of 20–100 mSv, and provides details of specific protective actions and other response actions in different time frames.

A-2. For the thyroid, iodine thyroid blocking is an urgent protective action that is prescribed: (i) if exposure due to radioactive iodine is involved, (ii) before or shortly after a release of radioactive iodine and (iii) within a short period after an intake of radioactive iodine.

A-3. In the absence of national guidance, the generic criteria could be used as a basis for the development of criteria at the national level. In exceptional situations, the use of higher values for the generic criteria may be necessary, such as when no replacement food or water is available.

TABLE A-1. GENERIC CRITERIA FOR PROTECTIVE ACTIONS AND OTHER RESPONSE ACTIONS IN AN EMERGENCY TO REDUCE THE RISK OF STOCHASTIC EFFECTS

Generic criteria		Examples of protective actions and other response actions
Projected dose that exceeds the following generic criteria: Take urgent protective actions and other response actions		
H_{thyroid}	50 mSv in the first 7 days	Iodine thyroid blocking
E	100 mSv in the first 7 days	Sheltering; evacuation; decontamination; restrictions on food, milk and drinking water; contamination control; reassurance of the public
H_{fetus}	100 mSv in the first 7 days	
Projected dose that exceeds the following generic criteria: Take early protective actions and other response actions		
E	100 mSv in the first year	Temporary relocation; decontamination; restrictions on food, milk and drinking water; reassurance of the public
H_{fetus}	100 mSv for the full period of in utero development	
Dose that has been received and that exceeds the following generic criteria: Take longer term medical actions to detect and to effectively treat radiation induced health effects		
E	100 mSv in a month	Health screening based on equivalent doses to specific radiosensitive organs (as a basis for medical follow-up); counselling
H_{fetus}	100 mSv for the full period of in utero development	Counselling to allow informed decisions to be made in individual circumstances

DEFINITIONS

The following definitions apply for the purposes of these Standards.

Further definitions are provided in the IAEA Safety Glossary: Terminology Used in Nuclear Safety and Radiation Protection (2007 Edition), IAEA, Vienna (2007): <http://www-ns.iaea.org/standards/safety-glossary.asp>

The symbol ‘①’ denotes an information note.

The symbol ‘!’ denotes a cautionary note.

Notes do not constitute part of the definition.

absorbed dose, D

The fundamental dosimetric quantity D , defined as:

$$D = \frac{d\bar{\epsilon}}{dm}$$

where $d\bar{\epsilon}$ is the mean energy imparted by ionizing radiation to matter in a volume element and dm is the mass of matter in the volume element.

- ① The SI unit for absorbed dose is joule per kilogram (J/kg), termed the gray (Gy).
- ① The energy can be averaged over any defined volume, the average dose being equal to the total energy imparted in the volume divided by the mass in the volume.
- ① Absorbed dose is defined at a point; the mean absorbed dose in a specified tissue or organ of the human body is termed the organ dose.

accident

Any unintended event, including operating errors, equipment failures and other mishaps, the consequences or potential consequences of which are not negligible from the point of view of protection and safety.

activation

The process of inducing radioactivity in matter by irradiation of that matter.

activity

1. The quantity A for an amount of radionuclide in a given energy state at a given time, defined as:

$$A(t) = \frac{dN}{dt}$$

where dN is the expectation value of the number of spontaneous nuclear transformations from the given energy state in the time interval dt .

① The SI unit for activity is reciprocal second (s^{-1}), termed the becquerel (Bq).

2. See facilities and activities.

ambient dose equivalent, $H^*(d)$

The dose equivalent that would be produced by the corresponding aligned and expanded field in the International Commission on Radiation Units and Measurements ICRU sphere at a depth d on the radius vector opposing the direction of the aligned field.

① Parameter defined at a point in a radiation field. Used as a directly measurable proxy (i.e. substitute) for effective dose for use in monitoring of external exposure.

① The recommended value of d for strongly penetrating radiation is 10 mm.

annual dose

The dose from external exposure in a year plus the committed dose from intakes of radionuclides in that year.

approval

The granting of consent by a regulatory body.

area monitoring

A form of workplace monitoring in which an area is monitored by taking measurements at different points in that area.

① As opposed to measurements by a static monitor.

assessment

The process, and the result, of analysing systematically and evaluating the hazards associated with sources and practices, and associated protection and safety measures.

authorization

The granting by a regulatory body or other governmental body of written permission for a person or organization (the operator) to conduct specified activities.

bioassay

Any procedure used to determine the nature, activity, location or retention of radionuclides in the body by direct (in vivo) measurement or by in vitro analysis of material excreted or otherwise removed from the body.

carers and comforters

Persons who willingly and voluntarily help (other than in their occupation) in the care, support and comfort of patients undergoing radiological procedures for medical diagnosis or medical treatment.

clearance

The removal of regulatory control by the regulatory body from radioactive material or radioactive objects within notified or authorized practices.

- ① Removal of regulatory control in this context refers to regulatory control applied for radiation protection purposes.

clearance level

A value, established by a regulatory body and expressed in terms of activity concentration, at or below which regulatory control may be removed from a source of radiation within a notified or authorized practice.

committed dose

The lifetime dose expected to result from an intake.

committed effective dose, $E(\tau)$

The quantity $E(\tau)$, defined as:

$$E(\tau) = \sum_T w_T \cdot H_T(\tau)$$

where $H_T(\tau)$ is the committed equivalent dose to tissue or organ T over the integration time τ elapsed after an intake of radioactive substances and w_T is the tissue weighting factor for tissue or organ T. When τ is not specified, it will be taken to be 50 years for adults and the time to age 70 years for intakes by children.

committed equivalent dose, $H_T(\tau)$

The quantity $H_T(\tau)$, defined as:

$$H_T(\tau) = \int_{t_0}^{t_0 + \tau} \dot{H}_T(t) dt$$

where t_0 is the time of intake, $\dot{H}_T(t)$ is the equivalent dose rate at time t in tissue or organ T and τ is the integration time elapsed after an intake of radioactive substances. When τ is not specified, it will be taken to be 50 years for adults and the time to age 70 years for intakes by children.

confinement

Prevention or control of releases of radioactive material to the environment in operation or in accidents.

constraint

A prospective and source related value of individual dose (dose constraint) or of individual risk (risk constraint) that is used in planned exposure situations as a parameter for the optimization of protection and safety for the source, and that serves as a boundary in defining the range of options in optimization.

- ① For occupational exposure, a constraint on individual dose to workers established and used by registrants and licensees to set the range of options in optimizing protection and safety for the source.

- ① For public exposure, the dose constraint is a source related value established or approved by the government or the regulatory body, with account taken of the doses from planned operations of all sources under control.
- ① The dose constraint for each particular source is intended, among other things, to ensure that the sum of doses from planned operations for all sources under control remains within the dose limit.
- ① For medical exposure, the dose constraint is a source related value used in optimizing the protection of carers and comforters of patients undergoing radiological procedures, and the protection of volunteers subject to exposure as part of a programme of biomedical research.
- ① The risk constraint is a source related value that provides a basic level of protection for the individuals most at risk from a source. This risk is a function of the probability of an unintended event causing a dose and the probability of the detriment due to such a dose. Risk constraints correspond to dose constraints but apply to potential exposure.

consumer product

A device or manufactured item into which radionuclides have deliberately been incorporated or produced by activation, or which generates ionizing radiation, and which can be sold or made available to members of the public without special surveillance or regulatory control after sale.

- ① This includes items such as smoke detectors and luminous dials into which radionuclides have deliberately been incorporated and ion generating tubes. It does not include building materials, ceramic tiles, spa waters, minerals and foodstuffs, and it excludes products and appliances installed in public places (e.g. exit signs).

containment

Methods or physical structures designed to prevent or control the release and the dispersion of radioactive substances.

contamination

Radioactive substances on surfaces, or within solids, liquids or gases (including the human body), where their presence is unintended or undesirable, or the process giving rise to their presence in such places.

- ① Contamination does not include residual radioactive material remaining at a site after the completion of decommissioning.

- ① The term ‘contamination’ may have a connotation that is not intended. The term ‘contamination’ refers only to the presence of radioactivity, and gives no indication of the magnitude of the hazard involved.

control

The function or power or (usually as controls) means of directing, regulating or restraining.

- ① It should be noted that the usual meaning of the English word ‘control’ in safety related contexts is somewhat ‘stronger’ (more active) than that of its usual translations and other similar words in some other languages. For example, ‘control’ typically implies not only checking or monitoring something but also ensuring that corrective or enforcement measures are taken if the results of the checking or monitoring indicate such a need. This is in contrast, for example, to the more limited usage of the equivalent word in French and Spanish.

regulatory control. Any form of control or regulation applied to facilities and activities by a regulatory body for reasons relating to nuclear safety and radiation protection or to nuclear security.

controlled area

A defined area in which specific protection measures and safety provisions are or could be required for controlling exposures or preventing the spread of contamination in normal working conditions, and preventing or limiting the extent of potential exposures.

decontamination

The complete or partial removal of contamination by a deliberate physical, chemical or biological process.

- ① This definition is intended to include a wide range of processes for removing contamination from people, equipment and buildings, but to exclude the removal of radionuclides from within the human body or the removal of radionuclides by natural weathering or migration processes, which are not considered to be decontamination.

decorporation

The biological processes, facilitated by a chemical or biological agent, by which incorporated radionuclides are removed from the human body.

deterministic effect

A radiation induced health effect for which generally a threshold level of dose exists above which the severity of the effect is greater for a higher dose.

severe deterministic effect. A deterministic effect that is fatal or life threatening or results in a permanent injury that reduces quality of life.

- ① The level of the threshold dose is characteristic of the particular health effect but may also depend, to a limited extent, on the exposed individual. Examples of deterministic effects include erythema and acute radiation syndrome (radiation sickness).
- ① Deterministic effects are also referred to as ‘harmful tissue reactions’.

diagnostic reference level

A level used in medical imaging to indicate whether, in routine conditions, the dose to the patient or the amount of radiopharmaceuticals administered in a specified radiological procedure for medical imaging is unusually high or unusually low for that procedure.

directional dose equivalent, $H'(d, \Omega)$

The dose equivalent that would be produced by the corresponding expanded field in the International Commission on Radiation Units and Measurements ICRU sphere at a depth d on a radius in a specified direction Ω .

- ① Parameter defined at a point in a radiation field. Used as a directly measurable proxy (i.e. substitute) for equivalent dose in the skin for use in monitoring of external exposure.
- ① The recommended value of d for weakly penetrating radiation is 0.07 mm.

disposal

Emplacement of waste in an appropriate facility without the intention of retrieval.

dose

1. A measure of the energy deposited by radiation in a target.
2. Absorbed dose, committed equivalent dose, committed effective dose, effective dose, equivalent dose or organ dose, as indicated by the context.

committed dose. committed equivalent dose or committed effective dose.

dose assessment

Assessment of the dose(s) to an individual or group of people.

dose constraint

See constraint.

dose limit

The value of the effective dose or the equivalent dose to individuals in planned exposure situations that is not to be exceeded.

effective dose, E

The quantity E , defined as a summation of the tissue or organ equivalent doses, each multiplied by the appropriate tissue weighting factor:

$$E = \sum_T w_T \cdot H_T$$

where H_T is the equivalent dose in tissue or organ T and w_T is the tissue weighting factor for tissue or organ T. From the definition of equivalent dose, it follows that:

$$E = \sum_T w_T \cdot \sum_R w_R \cdot D_{T,R}$$

where w_R is the radiation weighting factor for radiation type R and $D_{T,R}$ is the average absorbed dose in the tissue or organ T delivered by radiation type R.

- ① The SI unit for effective dose is joule per kilogram (J/kg), termed the sievert (Sv). An explanation of the quantity is given in Annex B of International Commission on Radiological Protection Publication 103¹.

¹ INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, The 2007 Recommendations of the International Commission on Radiological Protection, Publication 103, Elsevier (2007).

- ① Effective dose is a measure of dose designed to reflect the amount of radiation detriment likely to result from the dose.
- ① Effective dose cannot be used to quantify higher doses or to make decisions on the need for any medical treatment relating to deterministic effects.
- ① Values of effective dose from exposure for any type(s) of radiation and any mode(s) of exposure can be compared directly.

emergency

A non-routine situation that necessitates prompt action, primarily to mitigate a hazard or adverse consequences for human health and safety, quality of life, property or the environment. This includes nuclear or radiological emergencies and conventional emergencies such as fires, release of hazardous chemicals, storms or earthquakes. It includes situations for which prompt action is warranted to mitigate the effects of a perceived hazard.

nuclear or radiological emergency. An emergency in which there is, or is perceived to be, a hazard due to:

- (a) The energy resulting from a nuclear chain reaction or from the decay of the products of a chain reaction; or
- (b) Radiation exposure.

emergency action level (EAL)

A specific, predetermined, observable criterion used to detect, recognize and determine the emergency class.

emergency arrangements

The integrated set of infrastructural elements necessary to provide the capability for performing a specified function or task required in response to a nuclear or radiological emergency. These elements may include authorities and responsibilities, organization, coordination, personnel, plans, procedures, facilities, equipment or training.

emergency class

A set of conditions that warrant a similar immediate emergency response.

- ① This is the term used for communicating to the response organizations and to the public the level of response needed. The events that belong to a given emergency class are defined by criteria specific to the installation, source or practice, which, if exceeded, indicate classification at the prescribed level. For each emergency class, the initial actions of the response organizations are predefined.

emergency exposure situation

A situation of exposure that arises as a result of an accident, a malicious act or other unexpected event, and requires prompt action in order to avoid or reduce adverse consequences.

- ① Emergency exposures can be reduced only by protective actions and other response actions.

emergency plan

A description of the objectives, policy and concept of operations for the response to an emergency and of the structure, authorities and responsibilities for a systematic, coordinated and effective response. The emergency plan serves as the basis for the development of other plans, procedures and checklists.

emergency preparedness

The capability to take actions that will effectively mitigate the consequences of an emergency for human health and safety, quality of life, property and the environment.

emergency procedures

A set of instructions describing in detail the actions to be taken by response personnel in an emergency.

emergency response

The performance of actions to mitigate the consequences of an emergency for human health and safety, quality of life, property and the environment. It may also provide a basis for the resumption of normal social and economic activity.

emergency worker

A person having specified duties as a worker in response to an emergency.

- ① Emergency workers may include workers employed by registrants and licensees, as well as personnel of response organizations, such as police officers, firefighters, medical personnel, and drivers and crews of evacuation vehicles.
- ① An emergency worker may or may not be designated as such in advance of an emergency. An emergency worker not designated as such in advance of an emergency is not necessarily a worker prior to the emergency.

employer

A person or organization with recognized responsibilities, commitments and duties towards a worker in the employment of the person or organization by virtue of a mutually agreed relationship.

- ! A self-employed person is regarded as being both an employer and a worker.

environment

The conditions under which people, animals and plants live or develop and which sustain all life and development; especially such conditions as affected by human activities.

- ① Protection of the environment includes the protection and conservation of: non-human species, both animal and plant, and their biodiversity; environmental goods and services, such as the production of food and feed; resources used in agriculture, forestry, fisheries and tourism; amenities used in spiritual, cultural and recreational activities; media, such as soil, water and air; and natural processes, such as carbon, nitrogen and water cycles.

environmental monitoring

The measurement of external dose rates due to sources in the environment or of radionuclide concentrations in environmental media.

- ① Contrasted with source monitoring.

equilibrium equivalent concentration (EEC)

The activity concentration of ^{222}Rn or ^{220}Rn in radioactive equilibrium with its short lived progeny that would have the same potential alpha energy concentration as the actual (non-equilibrium) mixture.

- ① The equilibrium equivalent concentration of ^{222}Rn is given by:
 $\text{EEC } ^{222}\text{Rn} = (0.104 \times C(^{218}\text{Po})) + (0.514 \times C(^{214}\text{Pb})) + (0.382 \times C(^{214}\text{Bi}))$
 where $C(x)$ is the activity concentration of nuclide x in air.
 1 Bq/m³ EEC ^{222}Rn corresponds to 5.56×10^{-6} mJ/m³.
- ① The equilibrium equivalent concentration of ^{220}Rn is given by:
 $\text{EEC } ^{220}\text{Rn} = (0.913 \times C(^{212}\text{Pb})) + (0.087 \times C(^{212}\text{Bi}))$
 where $C(x)$ is the activity concentration of nuclide x in air.
 1 Bq/m³ EEC ^{220}Rn corresponds to 7.57×10^{-5} mJ/m³.

equilibrium factor

The ratio of the equilibrium equivalent activity concentration of ^{222}Rn to the actual ^{222}Rn activity concentration.

equivalent dose, H_T

The quantity $H_{T,R}$, defined as:

$$H_{T,R} = w_R \cdot D_{T,R}$$

where $D_{T,R}$ is the absorbed dose delivered by radiation type R averaged over a tissue or organ T and w_R is the radiation weighting factor for radiation type R. When the radiation field is composed of different radiation types with different values of w_R , the equivalent dose is:

$$H_T = \sum_R w_R \cdot D_{T,R}$$

- ① The SI unit for equivalent dose is joule per kilogram (J/kg), termed the sievert (Sv). An explanation of the quantity is given in Annex B of International Commission on Radiological Protection Publication 103².
- ① Equivalent dose is a measure of the dose to a tissue or organ designed to reflect the amount of harm caused.
- ① Equivalent dose cannot be used to quantify higher doses or to make decisions on the need for any medical treatment relating to deterministic effects.
- ① Values of equivalent dose to a specified tissue or organ from any type(s) of radiation can be compared directly.

² INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, The 2007 Recommendations of the International Commission on Radiological Protection, Publication 103, Elsevier (2007).

event

In the context of the reporting and analysis of events, an event is any occurrence unintended by the operator, including operating error, equipment failure or other mishap, and deliberate action on the part of others, the consequences or potential consequences of which are not negligible from the point of view of protection and safety.

exemption

The determination by a regulatory body that a source or practice need not be subject to some or all aspects of regulatory control on the basis that the exposure and the potential exposure due to the source or practice are too small to warrant the application of those aspects or that this is the optimum option for protection irrespective of the actual level of the doses or risks.

exemption level

A value, established by a regulatory body and expressed in terms of activity concentration, total activity, dose rate or radiation energy, at or below which a source of radiation need not be subject to some or all aspects of regulatory control.

existing exposure situation

A situation of exposure that already exists when a decision on the need for control needs to be taken.

- ① Existing exposure situations include exposure to natural background radiation that is amenable to control; exposure due to residual radioactive material that derives from past practices that were never subject to regulatory control or exposure due to residual radioactive material deriving from a nuclear or radiological emergency after an emergency has been declared to be ended.

exposure

The state or condition of being subject to irradiation.

external exposure. Exposure to radiation from a source outside the body.

internal exposure. Exposure to radiation from a source within the body.

exposure pathway

A route by which radiation or radionuclides can reach humans and cause exposure.

facilities and activities³

A general term encompassing nuclear facilities, uses of all sources of ionizing radiation, all radioactive waste management activities, transport of radioactive material and any other practice or circumstances in which people may be subject to exposure to radiation from naturally occurring or artificial sources.

- ① ‘Facilities’ includes: nuclear facilities; irradiation installations; some mining and raw material processing facilities such as uranium mines; radioactive waste management facilities; and any other places where radioactive material is produced, processed, used, handled, stored or disposed of — or where radiation generators are installed — on such a scale that consideration of protection and safety is required.
- ① ‘Activities’ includes: the production, use, import and export of radiation sources for industrial, research and medical purposes; the transport of radioactive material; the decommissioning of facilities; radioactive waste management activities such as the discharge of effluents; and some aspects of the remediation of sites affected by residues from past activities.
- ① This term is intended to provide an alternative to the terminology of sources and practices (or intervention) to refer to general categories of situations. For example, a practice may involve many different facilities and/or activities, whereas the general definition (1) of ‘source’ is too broad in some cases: a facility or activity might constitute a source, or might involve the use of many sources, depending upon the interpretation used.
- ① The term ‘facilities and activities’ is very general, and includes those for which little or no regulatory control may be necessary or achievable: the more specific terms ‘authorized facility’ and ‘authorized activity’ should be used to distinguish those facilities and activities for which any form of authorization has been given.

³ A small number of ‘catch-all’ terms — e.g. ‘facilities and activities’; ‘protection and safety’; ‘radiation risks’; and ‘structures, systems and components’ — are defined in the IAEA Safety Glossary. These terms may be used in exactly the form listed to describe a whole group of things without cumbersome repetition, or slight variations of the terms may be used to refer to particular subgroups. Although the definitions include an indication of the meanings of the separate elements of the terms, these are not intended to be applied rigidly: if precise reference is needed to particular items covered by the ‘catch-all’ term, more precise terms should be used.

- ① In the Fundamental Safety Principles (Safety Fundamentals), the term ‘facilities and activities — existing and new — utilized for peaceful purposes’ is abbreviated for convenience to ‘facilities and activities’ as a general term encompassing any human activity that may cause people to be exposed to radiation risks arising from naturally occurring or artificial sources (see para. 1.9 of IAEA Safety Standards Series No. SF-1⁴).

feed

Any single material or multiple materials, whether processed, semi-processed or raw, that is or are intended to be fed directly to food producing animals.

fluence

- ① A measure of the strength of a radiation field. Commonly used without qualification to mean particle fluence.

energy fluence, Ψ . A measure of the energy density of a radiation field, defined as:

$$\Psi = \frac{dR}{da}$$

where dR is the radiation energy incident on a sphere of cross-sectional area da .

- ① See International Commission on Radiological Protection Publication 74⁵.

⁴ EUROPEAN ATOMIC ENERGY COMMUNITY, FOOD AND AGRICULTURE ORGANIZATION OF THE UNITED NATIONS, INTERNATIONAL ATOMIC ENERGY AGENCY, INTERNATIONAL LABOUR ORGANIZATION, INTERNATIONAL MARITIME ORGANIZATION, OECD NUCLEAR ENERGY AGENCY, PAN AMERICAN HEALTH ORGANIZATION, UNITED NATIONS ENVIRONMENT PROGRAMME, WORLD HEALTH ORGANIZATION, Fundamental Safety Principles, IAEA Safety Standards Series No. SF-1, IAEA, Vienna (2006).

⁵ INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Conversion Coefficients for Use in Radiological Protection against External Radiation, ICRP Publication 74, Pergamon Press, Oxford and New York (1997).

particle fluence, Φ . A measure of the density of particles in a radiation field, defined as:

$$\Phi = \frac{dN}{da}$$

where dN is the number of particles incident on a sphere of cross-sectional area da .

① See International Commission on Radiological Protection Publication 74⁵.

food

Any substance, whether processed, semi-processed or raw, that is intended for human consumption.

① This includes drink (other than fresh water), chewing gum and substances used in the preparation or processing of food; it does not include cosmetics, tobacco or drugs. Consumption in this context refers to ingestion.

graded approach

For a system of control, such as a regulatory system or a safety system, a process or method in which the stringency of the control measures and conditions to be applied is commensurate, to the extent practicable, with the likelihood and possible consequences of, and the level of risk associated with, a loss of control.

hazard assessment

Assessment of hazards associated with facilities, activities or sources within or beyond the borders of a State in order to identify:

- (a) Those events and the associated areas for which protective actions may be required within the State;
- (b) The actions that would be effective in mitigating the consequences of such events.

health authority

A governmental authority (at the national, regional or local level) that is responsible for policies and interventions, including the development of standards and the provision of guidance, for maintaining or improving human health, and that has the legal power of enforcing such policies and interventions.

health professional

An individual who has been formally recognized through appropriate national procedures to practise a profession related to health (e.g. medicine, dentistry, chiropractic, podiatry, nursing, medical physics, medical radiation technology, radiopharmacy, occupational health).

health screening programme

A programme in which health tests or medical examinations are performed for the purpose of early detection of disease.

health surveillance

See workers' health surveillance.

hereditary effect

A radiation induced health effect that occurs in a descendant of the exposed person.

- ① Hereditary effects are usually stochastic effects.
- ① Contrasting term: somatic effect.

ICRU sphere

A sphere of 30 cm diameter made of tissue equivalent material with a density of 1 g/cm³ and a mass composition of 76.2% oxygen, 11.1% carbon, 10.1% hydrogen and 2.6% nitrogen.

- ① The International Commission on Radiation Units and Measurements ICRU sphere is used as a reference phantom in defining dose equivalent quantities.⁶

⁶ INTERNATIONAL COMMISSION ON RADIATION UNITS AND MEASUREMENTS, Radiation Quantities and Units, Rep. 33, ICRU, Bethesda, MD (1980).

incident

Any unintended event, including operating errors, equipment failures, initiating events, accident precursors, near misses or other mishaps, or unauthorized act, malicious or non-malicious, the consequences or potential consequences of which are not negligible from the point of view of protection and safety.

individual monitoring

Monitoring using measurements by equipment worn by individuals, or measurements of quantities of radioactive substances in or on, or taken into, the bodies of individuals, or measurements of quantities of radioactive substances excreted from the body by individuals.

- ① Usually contrasted with workplace monitoring.

inspection imaging device

An imaging device designed specifically for imaging persons or cargo conveyances for the purpose of detecting concealed objects on or within the human body or within cargo or a vehicle.

- ① In some types of inspection imaging device, ionizing radiation is used to produce images by backscatter, transmission or both. Other types of inspection imaging device utilize imaging by means of electrical and magnetic fields, ultrasound and sonar waves, nuclear magnetic resonance, microwaves, terahertz rays, millimetre waves, infrared radiation or visible light.

intake

1. The act or process of taking radionuclides into the body by inhalation or ingestion or through the skin.

- ① Other intake pathways are injection (important in nuclear medicine) and intake via a wound, as distinguished from intake through (intact) skin.

2. The activity of a radionuclide taken into the body in a given time period or as a result of a given event.

interested party

A person, company, etc., with a concern or interest in the activities and performance of an organization, business, system, etc.

- ① The term ‘interested party’ is used in a broad sense to mean a person or group having an interest in the performance of an organization. Those who can influence events may effectively become interested parties — whether their ‘interest’ is regarded as ‘genuine’ or not — in the sense that their views need to be considered. Interested parties have typically included the following: customers, owners, operators, employees, suppliers, partners, trade unions; the regulated industry or professionals; scientific bodies; governmental agencies or regulatory bodies (national, regional and local) whose responsibilities may cover nuclear energy; the media; the public (individuals, community groups and interest groups); and other States, especially neighbouring States that have entered into agreements providing for an exchange of information concerning possible transboundary impacts, or States involved in the import or export of certain technologies or materials.

investigation level

The value of a quantity such as effective dose, intake or contamination per unit area or volume at or above which an investigation would be conducted.

ionizing radiation

See radiation.

justification

1. The process of determining for a planned exposure situation whether a practice is, overall, beneficial; i.e. whether the expected benefits to individuals and to society from introducing or continuing the practice outweigh the harm (including radiation detriment) resulting from the practice.

2. The process of determining for an emergency exposure situation or an existing exposure situation whether a proposed protective action or remedial action is likely, overall, to be beneficial; i.e. whether the expected benefits to individuals and to society (including the reduction in radiation detriment) from introducing or continuing the protective action or remedial action outweigh the cost of such action and any harm or damage caused by the action.

kerma, K

The quantity K , defined as:

$$K = \frac{dE_{tr}}{dm}$$

where dE_{tr} is the sum of the initial kinetic energies of all charged ionizing particles liberated by uncharged ionizing particles in a material of mass dm .

- ① The SI unit for kerma is joule per kilogram (J/kg), termed the gray (Gy).

air kerma. The kerma value for air.

- ① Under charged particle equilibrium conditions, the air kerma (in grays) is numerically approximately equal to the absorbed dose in air (in grays).

reference air kerma rate. The kerma rate to air, in air, at a reference distance of 1 m, corrected for air attenuation and scattering.

- ① This quantity is expressed in $\mu\text{Gy/h}$ at 1 m.

licence

A legal document issued by the regulatory body granting authorization to perform specified activities relating to a facility or activity.

- ① A licence is a product of the authorization process and a practice with a current licence is an authorized practice.
- ① Authorization may take other forms, such as registration.
- ① The licensee is the person or organization having overall responsibility for a facility or activity.

licensee

The holder of a current licence.

limit

The value of a quantity used in certain specified activities or circumstances that must not be exceeded.

authorized limit. A limit on a measurable quantity, established or formally accepted by a regulatory body.

operational limits and conditions. A set of rules setting forth parameter limits, the functional capability and the performance levels of equipment and personnel approved by the regulatory body for safe operation of an authorized facility.

linear–no threshold (LNT) hypothesis

The hypothesis that the risk of stochastic effects is directly proportional to the dose for all levels of dose and dose rate below those levels at which deterministic effects occur.

- ① This means that any non-zero dose implies a non-zero risk of stochastic effects.
- ① This is the working hypothesis on which the IAEA's safety standards are based. It is not proven — indeed it is probably not provable — for low doses and dose rates, but it is considered the most radiobiologically defensible assumption on which to base safety standards.

lung absorption type

A classification used to distinguish between the different rates at which inhaled radionuclides are transferred from the respiratory tract to the blood.

- ① International Commission on Radiological Protection Publication 71⁷ classifies materials into four lung absorption types:
 - (a) Type V (very fast) are materials that, for dosimetric purposes, are assumed to be instantaneously absorbed into the blood;
 - (b) Type F (fast) are materials that are readily absorbed into the blood;
 - (c) Type M (moderate) are materials that have intermediate rates of absorption into the blood;
 - (d) Type S (slow) are materials that are relatively insoluble and are only slowly absorbed into the blood.
- ① See also gut transfer factor, a similar concept for ingested radionuclides in the gastrointestinal tract.

⁷ INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Age-dependent Doses to Members of the Public from Intakes of Radionuclides: Part 4 Inhalation Dose Coefficients, Publication 71, Pergamon Press, Oxford and New York (1995).

management system

A set of interrelated or interacting elements (system) for establishing policies and objectives and enabling the objectives to be achieved in an efficient and effective manner.

- ① The component parts of the management system include the organizational structure, resources and organizational processes. Management is defined (in ISO 9000) as coordinated activities to direct and control an organization.
- ① The management system integrates all elements of an organization into one coherent system to enable all of the organization's objectives to be achieved. These elements include the organizational structure, resources and processes. Personnel, equipment and organizational culture as well as the documented policies and processes are parts of the management system. The organization's processes have to address the totality of the requirements on the organization as established in, for example, IAEA safety standards and other international codes and standards.

medical exposure

Exposure incurred by patients for the purposes of medical or dental diagnosis or treatment; by carers and comforters; and by volunteers subject to exposure as part of a programme of biomedical research.

- ① A patient is an individual who is a recipient of services of health care professionals and/or their agents that are directed at (a) promotion of health; (b) prevention of illness and injury; (c) monitoring of health; (d) maintaining health; and (e) medical treatment of diseases, disorders and injuries in order to achieve a cure or, failing that, optimum comfort and function. Some asymptomatic individuals are included. For the purpose of the requirements on medical exposure in the IAEA safety standards, the term 'patient' refers only to those individuals undergoing radiological procedures.

medical physicist

A health professional with specialist education and training in the concepts and techniques of applying physics in medicine and competent to practise independently in one or more of the subfields (specialties) of medical physics.

- ① Competence of persons is normally assessed by the State by having a formal mechanism for registration, accreditation or certification of medical physicists in the various specialties (e.g. diagnostic radiology, radiation therapy, nuclear medicine). States that have yet to develop such a mechanism would need to assess the education, training and competence of any individual proposed by the licensee to act as a medical physicist and to decide, on the basis of either

international accreditation standards or standards of a State where such an accreditation system exists, whether such an individual could undertake the functions of a medical physicist, within the required specialty.

medical radiation facility

A medical facility in which radiological procedures are performed.

medical radiation technologist

A health professional, with specialist education and training in medical radiation technology, competent to perform radiological procedures, on delegation from the radiological medical practitioner, in one or more of the specialties of medical radiation technology.

- ① Competence of persons is normally assessed by the State by having a formal mechanism for registration, accreditation or certification of medical radiation technologists in the various specialties (e.g. diagnostic radiology, radiation therapy, nuclear medicine). States that have yet to develop such a mechanism would need to assess the education, training and competence of any individual proposed by the licensee to act as a medical radiation technologist and to decide, on the basis of either international standards or standards of a State where such a system exists, whether such an individual could undertake the functions of a medical radiation technologist, within the required specialty.

medical radiological equipment

Radiological equipment used in medical radiation facilities to perform radiological procedures that either delivers an exposure of an individual or directly controls or influences the extent of such exposure. The term applies to radiation generators, such as X ray machines or medical linear accelerators; to devices containing sealed sources, such as ^{60}Co teletherapy units; to devices used in medical imaging to capture images, such as gamma cameras, image intensifiers or flat panel detectors, and to hybrid systems such as positron emission tomography–computed tomography scanners.

member of the public

For purposes of protection and safety, in a general sense, any individual in the population except when subject to occupational exposure or medical exposure. For the purpose of verifying compliance with the annual dose limit for public exposure, this is the representative person.

monitoring

The measurement of dose, dose rate or activity for reasons relating to the assessment or control of exposure to radiation or exposure due to radioactive substances, and the interpretation of the results.

- ① ‘Measurement’ is used somewhat loosely here. The ‘measurement’ of dose often means the measurement of a dose equivalent quantity as a proxy (i.e. substitute) for a dose quantity that cannot be measured directly. Also, sampling may be involved as a preliminary step to measurement.
- ① Measurements may actually be made of radiation levels, airborne activity concentrations, levels of contamination, quantities of radioactive material, or individual doses. The results of these measurements may be used to assess radiological hazards or doses resulting or potentially resulting from exposure.
- ① Monitoring may be subdivided in two different ways: according to where the measurements are made, into individual monitoring, workplace monitoring, source monitoring and environmental monitoring; and, according to the purpose of the monitoring, into routine monitoring, task related monitoring and special monitoring.

natural background

The doses, dose rates or activity concentrations associated with natural sources, or any other sources in the environment that are not amenable to control.

- ① This is normally considered to include doses, dose rates or activity concentrations associated with natural sources, global fallout (but not local fallout) from atmospheric nuclear weapon tests and the Chernobyl accident.

notification

A document submitted to the regulatory body by a person or organization to notify an intention to carry out a practice or other use of a source.

nuclear fuel cycle

All operations associated with the production of nuclear energy.

- ① These include:
 - (a) Mining and processing of uranium ores or thorium ores;
 - (b) Enrichment of uranium;
 - (c) Manufacture of nuclear fuel;
 - (d) Operation of nuclear reactors (including research reactors);
 - (e) Reprocessing of spent fuel;

- (f) All waste management activities (including decommissioning) relating to operations associated with the production of nuclear energy;
- (g) Any related research and development activities.

nuclear installation

Any nuclear facility subject to authorization that is part of the nuclear fuel cycle, except facilities for the mining or processing of uranium ores or thorium ores and radioactive waste disposal facilities.

- ① This definition thus includes: nuclear power plants; research reactors (including subcritical and critical assemblies) and any adjoining radioisotope production facilities; spent fuel storage facilities; facilities for the enrichment of uranium; nuclear fuel fabrication facilities; conversion facilities; facilities for the reprocessing of spent fuel; facilities for the predisposal management of radioactive waste arising from nuclear fuel cycle facilities; and nuclear fuel cycle related research and development facilities.

nuclear or radiological emergency

See emergency.

(nuclear) safety

The achievement of proper operating conditions, prevention of accidents or mitigation of accident consequences, resulting in protection of workers, the public and the environment from undue radiation hazards.

See also protection and safety, safety.

(nuclear) security

The prevention of, detection of, and response to, criminal or intentional unauthorized acts involving or directed at nuclear material, other radioactive material, associated facilities, or associated activities.

- ① There is not an exact distinction between the general terms ‘safety’ and ‘security’. In general, ‘security’ is concerned with criminal acts or intentional unauthorized acts by humans that could cause or threaten harm to other humans; ‘safety’ is concerned with the broader issue of harm to humans (or to the environment) from radiation, whatever the cause. The precise interaction between security and safety depends on the context. Security of nuclear material for reasons relating to non-proliferation is outside the scope of the IAEA safety standards.

occupancy factor

A typical fraction of the time for which a location is occupied by an individual or group.

occupational exposure

Exposure of workers incurred in the course of their work.

operational intervention level (OIL)

A set level of a measurable quantity that corresponds to a generic criterion.

- ① Operational intervention levels are typically expressed in terms of dose rates or of activity of radioactive material released, time integrated air activity concentrations, ground or surface concentrations, or activity concentrations of radionuclides in environmental, food or water samples. An operational intervention level is used immediately and directly (without further assessment) to determine the appropriate protective actions on the basis of an environmental measurement.

optimization of protection and safety

The process of determining what level of protection and safety would result in the magnitude of individual doses, the number of individuals (workers and members of the public) subject to exposure and the likelihood of exposure being “as low as reasonably achievable, economic and social factors being taken into account” (ALARA).

For medical exposures of patients, the optimization of protection and safety is the management of the radiation dose to the patient commensurate with the medical purpose.

- ① “Protection and safety is optimized” means that optimization of protection and safety has been applied and the result of that process has been implemented.

personal dose equivalent, $H_p(d)$

The dose equivalent in soft tissue below a specified point on the body at an appropriate depth d .

- ① Parameter used as a directly measurable proxy (i.e. substitute) for equivalent dose in tissues or organs or (with $d = 10$ mm) for effective dose, in individual monitoring of external exposure.
- ① The recommended values of d are 10 mm for strongly penetrating radiation and 0.07 mm for weakly penetrating radiation for whole body monitoring.
- ① $H_p(0.07)$ is used for monitoring for hands and feet for all radiation types.
- ① $H_p(3)$ is used for monitoring exposure of the lens of the eye.
- ① ‘Soft tissue’ is commonly interpreted as the International Commission on Radiation Units and Measurements ICRU sphere.

planned exposure situation

The situation of exposure that arises from the planned operation of a source or from a planned activity that results in an exposure due to a source.

- ① Since provision for protection and safety can be made before embarking on the activity concerned, associated exposures and their probabilities of occurrence can be restricted from the outset. The primary means of controlling exposure in planned exposure situations is by good design of installations, equipment and operating procedures. In planned exposure situations, a certain level of exposure is expected to occur.

planning target volume

A geometrical concept used in radiation therapy for planning medical treatment with consideration of the net effect of movements of the patient and of the tissues to be irradiated, variations in size and shape of the tissues, and variations in beam geometry such as beam size and beam direction.

potential exposure

Prospectively considered exposure that is not expected to be delivered with certainty but that may result from an anticipated operational occurrence or accident at a source or owing to an event or sequence of events of a probabilistic nature, including equipment failures and operating errors.

- ① Potential exposure includes prospectively considered (hypothetical) exposures due to a source in an event or sequence of events of a probabilistic nature, including exposures resulting from an accident, equipment failures, operating errors, natural phenomena (such as hurricanes, earthquakes and floods) and inadvertent human intrusion (such as an intrusion into a near surface waste disposal facility after institutional control is removed).

practice

Any human activity that introduces additional sources of exposure or additional exposure pathways, or modifies the network of exposure pathways from existing sources, so as to increase the exposure or the likelihood of exposure of people or the number of people exposed.

- ! Radioactive waste is generated as a result of practices that involve some beneficial effect, such as the generation of electricity by nuclear means or the diagnostic application of radioisotopes. The management of this waste is therefore only one part of the overall practice.

projected dose

The dose that would be expected to be received if planned protective actions were not taken.

protection (against radiation)

radiation protection (also radiological protection). The protection of people from harmful effects of exposure to ionizing radiation, and the means for achieving this.

protection and safety

The protection of people against exposure to ionizing radiation or exposure due to radioactive material and the safety of sources, including the means for achieving this, and the means for preventing accidents and for mitigating the consequences of accidents if they do occur.

- ① For the purposes of the IAEA safety standards, ‘protection and safety’ includes the protection of people against ionizing radiation and safety; it does not include non-radiation-related aspects of safety. ‘Protection and safety’ is concerned with both radiation risks under normal circumstances and radiation risks as a consequence of incidents, as well as with other possible direct consequences of a loss of control over a nuclear reactor core, nuclear chain

reaction, radioactive source or any other source of radiation. Safety measures include actions to prevent incidents and arrangements put in place to mitigate their consequences if they were to occur.

protection of the environment

See environment.

protective action

An action for the purposes of avoiding or reducing doses that might otherwise be received in an emergency exposure situation or an existing exposure situation.

public exposure

Exposure incurred by members of the public due to sources in planned exposure situations, emergency exposure situations and existing exposure situations, excluding any occupational exposure or medical exposure.

qualified expert

An individual who, by virtue of certification by appropriate boards or societies, professional licence or academic qualifications and experience, is duly recognized as having expertise in a relevant field of specialization, e.g. medical physics, radiation protection, occupational health, fire safety, quality management or any relevant engineering or safety specialty.

quality assurance

The function of a management system that provides confidence that specified requirements will be fulfilled.

- ① Planned and systematic actions are necessary to provide adequate confidence that an item, process or service will satisfy given requirements for quality; for example, those specified in the licence. This statement has been slightly modified from that in the International Organization for Standardization's publication ISO 921:1997⁸ to 'an item, process or service' instead of 'a product or service' and to add the example. A more general definition of quality assurance (all those planned and systematic actions necessary to provide

⁸ INTERNATIONAL ORGANIZATION FOR STANDARDIZATION, Nuclear Energy: Vocabulary (Second Edition), ISO 921:1997, ISO, Geneva (1997).

confidence that a structure, system or component will perform satisfactorily in service) and definitions of related terms can be found in the International Organization for Standardization's publication ISO 8402:1994⁹.

radiation

- ! When used in IAEA publications, the term 'radiation' refers only to ionizing radiation unless otherwise stated. The IAEA has no statutory responsibilities in relation to non-ionizing radiation.

ionizing radiation. For the purposes of radiation protection, radiation capable of producing ion pairs in biological material(s).

- ① For most practical purposes, it may be assumed that strongly penetrating radiation includes photons of energy above about 12 keV, electrons of energy more than about 2 MeV, and neutrons.
- ① For most practical purposes, it may be assumed that weakly penetrating radiation includes photons of energy below about 12 keV, electrons of energy less than about 2 MeV, and massive charged particles such as protons and alpha particles.

radiation detriment

The total harm that would eventually be incurred by a group that is subject to exposure and by its descendants as a result of the group's exposure to radiation from a source.

radiation generator

See source (1).

radiation protection

See protection.

radiation protection officer

A person technically competent in radiation protection matters relevant for a given type of practice who is designated by the registrant, licensee or employer to oversee the application of regulatory requirements.

⁹ INTERNATIONAL ORGANIZATION FOR STANDARDIZATION, Quality Management and Quality Assurance — Vocabulary, ISO 8024:1994, ISO, Geneva (1994).

radiation risks

Detrimental health effects of exposure to radiation (including the likelihood of such effects occurring), and any other safety related risks (including those to the environment) that might arise as a direct consequence of:

- (a) Exposure to radiation;
- (b) The presence of radioactive material (including radioactive waste) or its release to the environment;
- (c) A loss of control over a nuclear reactor core, nuclear chain reaction, radioactive source or any other source of radiation.

radiation weighting factor, w_R

A number by which the absorbed dose in a tissue or organ is multiplied to reflect the relative biological effectiveness of the radiation in inducing stochastic effects at low doses, the result being the equivalent dose.

- ① Values are selected to be representative of the relevant relative biological effectiveness and are broadly compatible with the values previously recommended for quality factors in the definition of dose equivalent. The radiation weighting factor values¹⁰ are:

Type of radiation	w_R
Photons	1
Electrons and muons	1
Protons and charged pions	2
Alpha particles, fission fragments, heavy ions	20
Neutrons	A continuous function of neutron energy: $w_R = \begin{cases} 2.5 + 18.2 e^{-[\ln(E_n)]^2/6}, & E_n < 1 \text{ MeV} \\ 5.0 + 17.0 e^{-[\ln(2E_n)]^2/6}, & 1 \text{ MeV} \leq E_n \leq 50 \text{ MeV} \\ 2.5 + 3.25 e^{-[\ln(0.04E_n)]^2/6}, & E_n > 50 \text{ MeV} \end{cases}$

Note: All values relate to radiation incident on the body or, for internal radiation sources, radiation emitted from the incorporated radionuclide(s).

¹⁰ INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, The 2007 Recommendations of the International Commission on Radiological Protection, Publication 103, Elsevier (2007).

radioactive (adjective)

1. Exhibiting radioactivity; emitting or relating to the emission of ionizing radiation or particles.

- ① This is the ‘scientific’ definition, and should not be confused with the ‘regulatory’ definition (2).

2. Designated in national law or by a regulatory body as being subject to regulatory control because of its radioactivity.

- ① This is the ‘regulatory’ definition, and should not be confused with the ‘scientific’ definition (1).

radioactive material

Material designated in national law or by a regulatory body as being subject to regulatory control because of its radioactivity.

- ① This is the ‘regulatory’ meaning of radioactive (2), and should not be confused with the ‘scientific’ meaning of radioactive (1): ‘exhibiting radioactivity; emitting or relating to the emission of ionizing radiation or particles’. The ‘scientific’ meaning of radioactive — as in radioactive substance — refers only to the presence of radioactivity, and gives no indication of the magnitude of the hazard involved.

radioactive source

A source containing radioactive material that is used as a source of radiation.

radioactive substance

- ① This is the ‘scientific’ meaning of radioactive (1), and should not be confused with the ‘regulatory’ meaning of radioactive (2): ‘Designated in national law or by a regulatory body as being subject to regulatory control because of its radioactivity.’ The ‘scientific’ meaning of radioactive refers only to the presence of radioactivity, and gives no indication of the magnitude of the hazard involved.

radioactive waste

For legal and regulatory purposes, material for which no further use is foreseen that contains, or is contaminated with, radionuclides at activity concentrations greater than clearance levels as established by the regulatory body.

- ! It should be recognized that this definition is purely for regulatory purposes, and that material with activity concentrations equal to or less than clearance levels is radioactive from a physical viewpoint, although the associated radiological hazards are considered negligible.

radioactive waste management

All administrative and operational activities involved in the handling, pretreatment, treatment, conditioning, transport, storage and disposal of radioactive waste.

predisposal management of radioactive waste. Any waste management steps carried out prior to disposal, such as pretreatment, treatment, conditioning, storage and transport activities.

- ① Predisposal management is used as a contraction of ‘pre-disposal management of radioactive waste’; ‘predisposal’ is not a form of disposal.

processing. Any operation that changes the characteristics of waste, including pretreatment, treatment and conditioning.

radioactive waste management facility

Facility specifically designed to handle, treat, condition, store or permanently dispose of radioactive waste.

radiological medical practitioner

A health professional with specialist education and training in the medical uses of radiation, who is competent to perform independently or to oversee radiological procedures in a given specialty.

- ① Competence of persons is normally assessed by the State by having a formal mechanism for registration, accreditation or certification of radiological medical practitioners in the given specialty (e.g. radiology, radiation therapy, nuclear medicine, dentistry, cardiology). States that have yet to develop such a mechanism need to assess the education, training and competence of any individual proposed by the licensee to act as a radiological medical practitioner and to decide, on the basis of either international standards or standards of a State where such a system exists, whether such an individual could undertake the functions of a radiological medical practitioner, within the required specialty.

radiological procedure

A medical imaging procedure or therapeutic procedure that involves ionizing radiation — such as a procedure in diagnostic radiology, nuclear medicine or radiation therapy, or a planning procedure, image guided interventional procedure or other interventional procedure involving radiation — delivered by a radiation generator, a device containing a sealed source or an unsealed source, or by means of a radiopharmaceutical administered to a patient.

radionuclides of natural origin

Radionuclides that occur naturally on Earth in significant quantities.

- ① The term is usually used to refer to the primordial radionuclides ^{40}K , ^{235}U , ^{238}U , ^{232}Th and their radioactive decay products.
- ① Contrasted with radionuclides of artificial origin; also artificial radionuclides, anthropogenic radionuclides and human-made radionuclides.

radiopharmacist

A health professional, with specialist education and training in radiopharmacy, who is competent to prepare and dispense radiopharmaceuticals used for the purposes of medical diagnosis and radionuclide therapy.

- ① Competence of persons is normally assessed by the State by having a formal mechanism for registration, accreditation or certification of radiopharmacists. States that have yet to develop such a mechanism need to assess the education, training and competence of any individual proposed by the licensee to act as a radiopharmacist and to decide, on the basis of either international standards or standards of a State where such a system exists, whether such an individual could undertake the functions of a radiopharmacist.

radon

Any combination of isotopes of the element radon.

- ① For the purposes of these Standards, radon refers to ^{220}Rn and ^{222}Rn .

radon progeny

The short lived radioactive decay products of ^{220}Rn and of ^{222}Rn .

- ① For ^{222}Rn , this includes the decay chain up to but not including ^{210}Pb , namely ^{218}Po , ^{214}Pb , ^{214}Bi and ^{214}Po , plus traces of ^{218}At , ^{210}Tl and ^{209}Pb . ^{210}Pb , which has a half-life of 22.3 a, and its radioactive progeny — ^{210}Bi and ^{210}Po , plus traces of ^{206}Hg and ^{206}Tl — are, strictly, progeny of ^{222}Rn , but they are not included in this listing because they will not normally be present in significant amounts in airborne form. For ^{220}Rn , this includes ^{216}Po , ^{212}Pb , ^{212}Bi , ^{212}Po and ^{208}Tl .

reference level

For an emergency exposure situation or an existing exposure situation, the level of dose, risk or activity concentration above which it is not appropriate to plan to allow exposures to occur and below which optimization of protection and safety would continue to be implemented.

- ① The value chosen for a reference level will depend upon the prevailing circumstances for the exposure under consideration.

referring medical practitioner

A health professional who, in accordance with national requirements, may refer individuals to a radiological medical practitioner for medical exposure.

registrant

The holder of a current registration.

- ① Other derivative terms should not be needed; a registration is a product of the authorization process, and a practice with a current registration is an authorized practice.

registration

A form of authorization for practices of low or moderate risks whereby the person or organization responsible for the practice has, as appropriate, prepared and submitted a safety assessment of the facilities and equipment to the regulatory body. The practice or use is authorized with conditions or limitations as appropriate.

- ① The requirements for safety assessment and the conditions or limitations applied to the practice would be less severe for registration than those for licensing.

- ① Typical practices that are amenable to registration are those for which:
(a) safety can largely be ensured by the design of the facilities and equipment;
(b) the operating procedures are simple to follow; (c) the safety training requirements are minimal; and (d) there is a history of few problems with safety in operations. Registration is best suited to those practices for which operations do not vary significantly.

regulatory body

An authority or a system of authorities designated by the government of a State as having legal authority for conducting the regulatory process, including issuing authorizations, and thereby regulating nuclear, radiation, radioactive waste and transport safety.

- ① The national competent authority for the regulation of radioactive material transport safety (see IAEA Safety Standards Series No. SSR-6¹¹) is included in this description, as is the regulatory body for protection and safety.

regulatory control

See control.

relative biological effectiveness (RBE)

A relative measure of the effectiveness of different radiation types at inducing a specified health effect, expressed as the inverse ratio of the absorbed doses of two different radiation types that would produce the same degree of a defined biological end point.

- ① Values of relative biological effectiveness in inducing stochastic effects are represented by the radiation weighting factor w_R .
- ① Values of relative biological effectiveness in inducing deterministic effects are selected to be representative of the severe deterministic effects that are significant to emergency preparedness and response. The tissue or organ specific and radiation specific values of $RBE_{T,R}$ for the development of selected severe deterministic effects are as shown in the table.

¹¹ INTERNATIONAL ATOMIC ENERGY AGENCY, Regulations for the Safe Transport of Radioactive Material, 2012 Edition, IAEA Safety Standards Series No. SSR-6, IAEA, Vienna (2012).

Health effect	Critical tissue or organ	Exposure ^a	RBE _{T,R}
Haematopoietic syndrome	Red marrow	External and internal γ	1
		External and internal n	3
		Internal β	1
		Internal α	2
Pneumonitis	Lung ^b	External and internal γ	1
		External and internal n	3
		Internal β	1
		Internal α	7
Gastrointestinal syndrome	Colon	External and internal γ	1
		External and internal n	3
		Internal β	1
		Internal α	0 ^c
Necrosis	Tissue ^d	External β, γ	1
		External n	3
Moist desquamation	Skin ^e	External β, γ	1
		External n	3
Hypothyroidism	Thyroid	Intake of iodine isotopes ^f	0.2
		Other thyroid seekers	1

^a External β, γ exposure includes exposure to bremsstrahlung produced within the material of the source.

^b Tissue of the alveolar–interstitial region of the respiratory tract.

^c For alpha emitters uniformly distributed in the contents of the colon, it is assumed that irradiation of the walls of the intestine is negligible.

^d Tissue at a depth of 5 mm below the skin surface over an area of more than 100 cm².

^e Tissue at a depth of 0.4 mm below the skin surface over an area of more than 100 cm².

^f Uniform irradiation of the tissue of the thyroid gland is considered to be five times more likely to produce deterministic effects than internal exposure due to low energy beta emitting isotopes of iodine such as ¹³¹I, ¹²⁹I, ¹²⁵I, ¹²⁴I and ¹²³I. Thyroid seeking radionuclides have a heterogeneous distribution in thyroid tissue. The isotope ¹³¹I emits low energy beta particles, which leads to a reduced effectiveness of irradiation of critical thyroid tissue owing to the dissipation of the energy of the particles within other tissues.

relative biological effectiveness (RBE) weighted absorbed dose, AD_T

The quantity $AD_{T,R}$, defined as:

$$AD_{T,R} = D_{T,R} \times RBE_{T,R}$$

where $D_{T,R}$ is the absorbed dose delivered by radiation of type R averaged over a tissue or organ T and $RBE_{T,R}$ is the relative biological effectiveness for radiation of type R in the production of severe deterministic effects in a tissue or organ T. When the radiation field is composed of different radiation types with different values of $RBE_{T,R}$, the RBE weighted absorbed dose is given by:

$$AD_T = \sum_R D_{T,R} \times RBE_{T,R}$$

- ① The SI unit for RBE weighted absorbed dose is joule per kilogram (J/kg), termed the gray (Gy).
- ① RBE weighted absorbed dose is a measure of the dose to a tissue or organ, intended to reflect the risk of development of severe deterministic effects.
- ① Values of RBE weighted absorbed dose to a specified tissue or organ from any type(s) of radiation can be compared directly.

remedial action

The removal of a source or the reduction of its magnitude (in terms of activity or amount) for the purposes of preventing or reducing exposures that might otherwise occur in an existing exposure situation.

- ① Remedial actions could also be termed longer term protective action, but longer term protective actions are not necessarily remedial actions.

remediation

Any measures that may be carried out to reduce the radiation exposure due to existing contamination of land areas through actions applied to the contamination itself (the source) or to the exposure pathways to humans.

- ① Complete removal of the contamination is not implied.

See decontamination.

representative person

An individual receiving a dose that is representative of the doses to the more highly exposed individuals in the population.

- ① The International Commission on Radiological Protection's Publication 101¹² indicates that the dose to the representative person "is the equivalent of, and replaces, the mean dose in the 'critical group'", and provides guidance on assessing doses to the representative person. The concept of critical group remains valid.

See member of the public.

residual dose

The dose expected to be incurred after protective actions have been terminated (or after a decision has been taken not to take protective actions).

- ① This applies for an existing exposure situation or an emergency exposure situation.

response organization

An organization designated or otherwise recognized by a State as being responsible for managing or implementing any aspect of an emergency response.

risk

A multiattribute quantity expressing hazard, danger or chance of harmful or injurious consequences associated with exposures or potential exposure. It relates to quantities such as the probability that specific deleterious consequences may arise and the magnitude and character of such consequences.

See radiation risks.

risk constraint

See constraint.

¹² INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Assessing Dose of the Representative Person for the Purpose of Radiation Protection of the Public and the Optimisation of Radiological Protection: Broadening the Process, ICRP Publication 101, Elsevier (2006).

safety

See (nuclear) safety, protection and safety.

- ① In the Fundamental Safety Principles (Safety Fundamentals)¹³, the generalized usage in this particular text of the term ‘safety’ (i.e. to mean protection and safety) is explained as follows (in paras 3.1 and 3.2):

“3.1. For the purposes of this publication, ‘safety’ means the protection of people and the environment against radiation risks, and the safety of facilities and activities that give rise to radiation risks. ‘Safety’ as used here and in the IAEA safety standards includes the safety of nuclear installations, radiation safety, the safety of radioactive waste management and safety in the transport of radioactive material; it does not include non-radiation-related aspects of safety.

“3.2. Safety is concerned with both radiation risks under normal circumstances and radiation risks as a consequence of incidents, as well as with other possible direct consequences of a loss of control over a nuclear reactor core, nuclear chain reaction, radioactive source or any other source of radiation. Safety measures include actions to prevent incidents and arrangements put in place to mitigate their consequences if they were to occur.”

safety assessment

Assessment of all aspects of a practice that are relevant to protection and safety; for an authorized facility, this includes siting, design and operation of the facility.

¹³ EUROPEAN ATOMIC ENERGY COMMUNITY, INTERNATIONAL ATOMIC ENERGY AGENCY, FOOD AND AGRICULTURE ORGANIZATION OF THE UNITED NATIONS, INTERNATIONAL LABOUR ORGANIZATION, INTERNATIONAL MARITIME ORGANIZATION, OECD NUCLEAR ENERGY AGENCY, PAN AMERICAN HEALTH ORGANIZATION, UNITED NATIONS ENVIRONMENT PROGRAMME, WORLD HEALTH ORGANIZATION, Safety Fundamentals: Fundamental Safety Principles, IAEA Safety Standards Series No. SF-1, IAEA, Vienna (2006).

safety culture

The assembly of characteristics and attitudes in organizations and individuals which establishes that, as an overriding priority, protection and safety issues receive the attention warranted by their significance.

safety measure

Any action that might be taken, condition that might be applied or procedure that might be followed to fulfil the requirements of Safety Requirements.

safety standards

Standards issued pursuant to Article III(A)(6)¹⁴ of the Statute of the IAEA¹⁵.

- ① IAEA publications issued since 1997 in the IAEA Safety Standards Series are designated as Safety Fundamentals, Safety Requirements or Safety Guides.

scenario

A postulated or assumed set of conditions and/or events.

- ① Most commonly used in analysis or assessment to represent possible future conditions and/or events to be modelled, such as possible accidents at a nuclear facility, or the possible future evolution of a repository and its surroundings. A scenario may represent the conditions at a single point in time or a single event, or a time history of conditions and/or events (including processes).
- ① See event.

security

See (nuclear) security.

somatic effect

A radiation induced health effect that occurs in the exposed person.

¹⁴ “[The Agency is authorized...] To establish or adopt, in consultation and, where appropriate, in collaboration with the competent organs of the United Nations and with the specialized agencies concerned, standards of safety for protection of health and minimization of danger to life and property (including such standards for labour conditions)...”

¹⁵ Statute of the International Atomic Energy Agency, IAEA, Vienna (1990).

- ① This includes effects occurring after birth that are attributable to exposure in utero.
- ① Deterministic effects are normally also somatic effects; stochastic effects may be somatic effects or hereditary effects.
- ① Contrasting term: hereditary effect.

source

1. Anything that may cause radiation exposure — such as by emitting ionizing radiation or by releasing radioactive substances or radioactive material — and can be treated as a single entity for purposes of protection and safety.

- ① For example, materials emitting radon are sources in the environment; a sterilization gamma irradiation unit is a source for the practice of irradiation preservation of food and sterilization of other products; an X ray unit may be a source for the practice of radiodiagnosis; a nuclear power plant is part of the practice of generating electricity by nuclear fission, and may be regarded as a source (e.g. with respect to discharges to the environment) or as a collection of sources (e.g. for occupational radiation protection purposes). A complex or multiple installation situated at one location or site may, as appropriate, be considered a single source for the purposes of application of the safety standards.

natural source. A naturally occurring source of radiation, such as the sun and stars (sources of cosmic radiation) and rocks and soil (terrestrial sources of radiation), or any other material whose radioactivity is for all intents and purposes due only to radionuclides of natural origin, such as products or residues from the processing of minerals; but excluding radioactive material for use in a nuclear installation and radioactive waste generated in a nuclear installation.

radiation generator. A device capable of generating ionizing radiation, such as X rays, neutrons, electrons or other charged particles, that may be used for scientific, industrial or medical purposes.

2. Radioactive material used as a source of radiation.

- ① Such as those sources used for medical applications or in industrial instruments. These are, of course, sources as defined in (1), but this usage in (2) is less general.

dangerous source. A source that could, if not under control, give rise to exposure sufficient to cause severe deterministic effects. This categorization

is used for determining the need for emergency arrangements and is not to be confused with categorizations of sources for other purposes.

- ① The term ‘dangerous source’ relates to dangerous quantities (D values) as recommended in Dangerous Quantities of Radioactive Material (D-Values), Emergency Preparedness and Response, EPR-D-VALUES 2006¹⁶.

radioactive source. A source containing radioactive material that is used as a source of radiation.

sealed source. A radioactive source in which the radioactive material is (a) permanently sealed in a capsule, or (b) closely bonded and in a solid form.

spent source. A source that is no longer suitable for its intended purpose as a result of radioactive decay.

! Note that a spent source may still represent a radiological hazard.

unsealed source. A radioactive source in which the radioactive material is neither (a) permanently sealed in a capsule nor (b) closely bonded and in a solid form.

source monitoring

The measurement of activity in radioactive material being released to the environment or of external dose rates due to sources within a facility or activity.

- ① Contrasted with environmental monitoring.

spent fuel

Nuclear fuel removed from a reactor following irradiation that is no longer usable in its present form because of depletion of fissile material, poison buildup or radiation damage.

¹⁶ INTERNATIONAL ATOMIC ENERGY AGENCY, Dangerous Quantities of Radioactive Material (D-Values), Emergency Preparedness and Response, EPR-D-VALUES, IAEA, Vienna (2006).

- ① The adjective ‘spent’ suggests that spent fuel cannot be used as fuel in its present form (e.g. as in spent source). In practice, however, spent fuel is commonly used to refer to fuel that has been used as fuel but will no longer be used, whether or not it could be used (and that might more accurately be termed ‘disused fuel’).

standards dosimetry laboratory

A laboratory, designated by the relevant national authority, that possesses certification or accreditation necessary for the purpose of developing, maintaining or improving primary or secondary standards for radiation dosimetry.

stochastic effect

A radiation induced health effect, the probability of occurrence of which is greater for a higher radiation dose and the severity of which (if it occurs) is independent of dose.

- ① Stochastic effects may be somatic effects or hereditary effects, and generally occur without a threshold level of dose. Examples include solid cancers and leukaemia.
- ① Contrasting term: deterministic effect.

storage

The holding of radioactive sources, radioactive material, spent fuel or radioactive waste in a facility that provides for their/its containment, with the intention of retrieval.

structures, systems and components

A general term encompassing all of the elements (items) of a facility or activity that contribute to protection and safety, except human factors.

- ① Structures are the passive elements: buildings, vessels, shielding, etc. A system comprises several components, assembled in such a way as to perform a specific (active) function. A component is a discrete element of a system. Examples of components are wires, transistors, integrated circuits, motors, relays, solenoids, pipes, fittings, pumps, tanks and valves.
- ① Human factors may be reflected in structures, systems and components in so far as ergonomics — the study of people’s efficiency in their working environment — is an element in their design.

supervised area

A defined area not designated as a controlled area but for which occupational exposure conditions are kept under review, even though specific protection measures or safety provisions are not normally needed.

supplier (of a source)

Any person or organization to whom a registrant or licensee assigns duties, totally or partially, in relation to the design, manufacture, production or construction of a source.

- ① The term ‘supplier’ of a source includes designers, manufacturers, producers, constructors, assemblers, installers, distributors, sellers, importers and exporters of a source.

tissue weighting factor, w_T

Multiplier of the equivalent dose to a tissue or organ used for purposes of radiation protection to account for the different sensitivities of different tissues or organs to the induction of stochastic effects of radiation.

- ① Tissue weighting factors used for calculating effective dose are as follows.

Tissue or organ	w_T	$\sum w_T$
Bone marrow (red), colon, lung, stomach, breast, remainder tissues ^a	0.12	0.72
Gonads	0.08	0.08
Bladder, oesophagus, liver, thyroid	0.04	0.16
Bone surface, brain, salivary glands, skin	0.01	0.04
	Total	1.00

^a The w_T for remainder tissues (0.12) applies to the arithmetic mean dose to these 13 tissues and organs for each sex: adrenals, extrathoracic region, gall bladder, heart, kidneys, lymphatic nodes, muscle, oral mucosa, pancreas, prostate (male), small intestine, spleen, thymus, uterus/cervix (female).

transboundary exposure

Exposure of members of the public in one State due to radioactive material released via accidents, discharges or waste disposal in another State.

transport

The deliberate physical movement of radioactive material (other than that forming part of the means of propulsion) from one place to another.

urgent protective action

See protective action.

worker

Any person who works, whether full time, part time or temporarily, for an employer and who has recognized rights and duties in relation to occupational radiation protection.

- ① A self-employed person is regarded as having the duties of both an employer and a worker.

workers' health surveillance

Medical supervision intended to ensure the initial and continuing fitness of workers for their intended tasks.

workplace monitoring

Monitoring using measurements made in the working environment.

- ① Usually contrasted with individual monitoring.

CONTRIBUTORS TO DRAFTING AND REVIEW

Abu-Eid, R.	Nuclear Regulatory Commission, United States of America
Ahier, B.	OECD Nuclear Energy Agency
Akhadi, M.	National Nuclear Energy Agency, Indonesia
Al-Arfaj, A.	Institute of Atomic Energy Research, Saudi Arabia
Ali, H.	Atomic Licensing Board, Malaysia
Ali, M.	Pakistan Nuclear Regulatory Authority, Pakistan
Amaral, E.	International Atomic Energy Agency
Amor Calvo, I.	Nuclear Safety Council, Spain
Ampuero Flores, C.	Peruvian Institute of Nuclear Energy, Peru
Andersen, R.	World Nuclear Association
Arvela, H.	Radiation and Nuclear Safety Authority, Finland
Awatsuji, Y.	Ministry of Education, Culture, Sports, Science and Technology, Japan
Bäckström, T.	Swedish Radiation Protection Authority, Sweden
Badulin, V.	Ministry of Health, Bulgaria
Baeklandt, L.	Federal Agency for Nuclear Control, Belgium
Basurto Cázares, J.	National Commission for Nuclear Safety and Safeguards, Mexico
Berkovskyy, V.	International Atomic Energy Agency
Boal, T.	International Atomic Energy Agency
Bochichhio, F.	Higher Institute of Health, Italy
Bologna, L.	APAT, Italy
Borras, C.	Consultant, Radiological Physics and Health Services, United States of America

Böttger, A.	Federal Ministry for the Environment, Nature Conservation and Nuclear Safety, Germany
Bourguignon, M.	World Health Organization
Brewer, S.	Atomic Energy of Canada Limited, Canada
Buglova, E.	International Atomic Energy Agency
Bundy, K.	Canadian Nuclear Safety Commission, Canada
Burns, P.	Australian Radiation Protection and Nuclear Safety Agency, Australia
Byron, D.	Food and Agriculture Organization of the United Nations
Cabral Molina, W.	Ministry of Industry, Energy and Mining, Uruguay
Calamosca, M.	Institute for Radiation Protection, Italy
Cancio, D.	Unidad de Protección Radiológica, Spain
Canoba, A.	Nuclear Regulatory Authority, Argentina
Carboneras Martinez, P.	National Company for Radioactive Waste, Spain
Carr, Z.	World Health Organization
Cernohlavek, N.	Austrian Agency for Health and Food Safety, Austria
Chambers, D.	SENES Consultants Limited, Canada
Charette, M.	International Source Suppliers and Producers Association
Cherf, A.	International Atomic Energy Agency
Cheung, K.	International Organization for Medical Physics
Chi, C.	China Institute for Radiation Protection, China
Cho, K.	Korea Institute of Nuclear Safety, Republic of Korea
Christofides, S.	International Organization for Medical Physics
Clement, C.	International Commission on Radiological Protection
Colgan, T.	International Atomic Energy Agency

Cool, D.	Nuclear Regulatory Commission, United States of America
Cooper, J.	Health Protection Agency, United Kingdom
Coppee, G.	International Labour Organization
Crick, M.	United Nations Scientific Committee on the Effects of Atomic Radiation
Cripwell, B.	International Labour Organization
Cruz-Suarez, R.	International Atomic Energy Agency
Currivan, L.	Radiological Protection Institute of Ireland, Ireland
Czarwinski, R.	International Atomic Energy Agency
de la Fuente Puch, A.	National Nuclear Safety Centre, Cuba
Deboodt, P.	International Atomic Energy Agency
Delattre, D.	International Atomic Energy Agency
Delves, D.	International Atomic Energy Agency
Dimitriou, P.	Greek Atomic Energy Commission, Greece
Ditto, M.	Federal Ministry for Health and Women, Austria
Ebdon-Jackson, S.	Health Protection Agency, United Kingdom
Fenton, D.	Radiological Protection Institute of Ireland, Ireland
Fischer, H.	Federal Ministry of Agriculture, Forestry, Environment and Water Management, Austria
Frullani, S.	Higher Institute of Health, Italy
Fujii, K.	Ministry of Education, Culture, Sports, Science and Technology, Japan
Fundarek, P.	Canadian Nuclear Safety Commission, Canada
Garcia-Talavera, M.	Nuclear Safety Council, Spain
Garcier, Y.	World Nuclear Association

Gaunt, M.	International Labour Organization, International Organization of Employers
Ghovanlou, A.	Health and Physics Safety, United States of America
Gilley, N.	Florida Department of Health Bureau of Radiation Control, United States of America
Gomaa, M.	Atomic Energy Authority, Egypt
Gonzalez, A.	Nuclear Regulatory Authority, Argentina
Griebel, J.	Federal Office for Radiation Protection, Germany
Groth, S.	World Health Organization
Gruson, M.	Federal Office of Public Health, Switzerland
Guven, M.	Turkish Atomic Energy Authority, Turkey
Hamani, W.	National Centre for Nuclear Science and Technology, Tunisia
Hammer, J.	Swiss Federal Nuclear Safety Inspectorate, Switzerland
Hanninen, R.	Radiation and Nuclear Safety Authority, Finland
Hattori, T.	Central Research Institute of Electric Power Industry, Japan
Havukainen, R.	Radiation and Nuclear Safety Authority, Finland
Hedemann, P.	Danish Decommissioning, Denmark
Helming, M.	Federal Ministry for the Environment, Nature Conservation and Nuclear Safety, Germany
Hesse, J.	World Nuclear Association
Hoffmann, B.	Federal Office for Radiation Protection, Germany
Homma, T.	Japan Atomic Energy Agency, Japan
Huffman, D.	AREVA Resources Canada Inc., Canada
Hugron, R.	National Defence Headquarters, Canada

Hulka, J.	SÚRO National Radiation Protection Institute, Czech Republic
Hunt, J.	International Atomic Energy Agency
Iimoto, T.	University of Tokyo, Japan
Inokuchi, T.	Nuclear Safety Commission, Japan
Ishikawa, N.	Nuclear Safety Commission, Japan
Ito, K.	Japan Atomic Energy Agency, Japan
Janssens, A.	European Commission
Janzekovic, H.	Slovenian Nuclear Safety Administration, Slovenia
Jensen, L.	National Institute of Radiation Protection, Denmark
Jerachanchai, S.	Office of Atoms for Peace, Thailand
Jimenez, P.	Pan American Health Organization
Jones, G.	International Atomic Energy Agency
Jung, T.	Federal Office for Radiation Protection, Germany
Jurina, V.	Public Health Authority, Slovakia
Kamenopoulou, V.	Greek Atomic Energy Commission, Greece
Kardan, M.	Atomic Energy Organization of Iran, Islamic Republic of Iran
Kelly, N.	Consultant, United Kingdom
Kenigsberg, J.	National Commission of Radiation Protection, Belarus
Kirchner, G.	Federal Office for Radiation Protection, Germany
Koblinger, L.	Hungarian Atomic Energy Authority, Hungary
Koc, J.	Temelin Nuclear Power Plant, Czech Republic
Koch, J.	Soreq Nuclear Research Center, Israel
Kolovou, M.	Greek Atomic Energy Commission, Greece
Kralik, I.	State Office for Radiation Protection, Croatia

Krca, S.	State Office for Radiation Protection, Croatia
Kuhlen, J.	Federal Ministry for the Environment, Nature Conservation and Nuclear Safety, Germany
Kulich, V.	Dukovany Nuclear Power Plant, Czech Republic
Kutkov, V.	Russian Research Centre “Kurchatov Institute”, Russian Federation
Landfermann, H.	Federal Ministry for the Environment, Nature Conservation and Nuclear Safety, Germany
Larsson, C.	Swedish Radiation Protection Authority, Sweden
Lazo, E.	OECD Nuclear Energy Agency
Le Guen, B.	World Nuclear Association
Le Heron, J.	International Atomic Energy Agency
Lecomte, J.-F.	Institute for Radiological Protection and Nuclear Safety, France
Lindvall, C.	Barsebäck Kraft AB, Sweden
Lipsztein, J.	Institute of Radiation Protection and Dosimetry, Brazil
Long, K.	Food and Agriculture Organization of the United Nations
Long, W.	Center for Radon and Air Toxics, United States of America
Lopes Gonzalez, F.	Universidad Nacional Autonoma de Nicaragua, Nicaragua
Lorenz, B.	World Nuclear Association
Louvat, D.	International Atomic Energy Agency
Lund, I.	Swedish Radiation Protection Authority, Sweden
Magnusson, S.	Icelandic Radiation Protection Institute, Iceland
Makarovska, O.	State Nuclear Regulatory Committee, Ukraine
Mansoux, H.	International Atomic Energy Agency

Marechal, N.	Comissão Nacional de Energia Nuclear, Brazil
Marengo, M.	World Federation of Nuclear Medicine and Biology
Maringer, F.	Low-level Counting Laboratory Arsenal, Austria
Markkanen, M.	Radiation and Nuclear Safety Authority, Finland
Martin Calvarro, J.	Nuclear Safety Council, Spain
Martincic, R.	International Atomic Energy Agency
Mason, C.	International Atomic Energy Agency
Massera, G.	Nuclear Regulatory Authority, Argentina
Mayya, Y.	Bhabha Atomic Research Centre, India
McClelland, V.	Department of Energy, United States of America
McKenna, T.	International Atomic Energy Agency
McLaughlin, J.	University College Dublin, Ireland
Meghzifene, A.	International Atomic Energy Agency
Merta, A.	National Atomic Energy Agency, Poland
Metcalf, P.	International Atomic Energy Agency
Mirsaidov, U.	Nuclear and Radiation Safety Agency, Tajikistan
Miyazaki, S.	Kansai Electric Power Co. Inc., Japan
Mizumachi, W.	Japan Nuclear Energy Safety Organization, Japan
Mokrani, Z.	Nuclear Research Centre, Algeria
Mrabit, K.	International Atomic Energy Agency
Mundigl, S.	European Commission
Naegele, J.	European Commission
Nandakumar, A.	International Atomic Energy Agency
Niu, S.	International Labour Organization
Owen, D.	International Labour Organization

Parkes, R.	Health and Safety Executive, United Kingdom
Pather, T.	National Nuclear Regulator, South Africa
Paynter, R.	Health Protection Agency, United Kingdom
Peñalosa, A.	International Organization of Employers
Perez, M.	World Health Organization
Perrin, M.	French Nuclear Safety Authority, France
Petrova, K.	State Office for Nuclear Safety, Czech Republic
Philpott, L.	Health and Safety Executive, United Kingdom
Pinak, M.	OECD Nuclear Energy Agency
Poffijn, A.	Federal Agency for Nuclear Control, Belgium
Purvis, C.	Canadian Nuclear Safety Commission, Canada
Radolic, V.	University Josip Juraj Strossmayer, Croatia
Rannou, A.	Institute for Radiological Protection and Nuclear Safety, France
Rehani, M.	International Atomic Energy Agency
Ringertz, H.	International Society of Radiology
Robinson, I.	Health and Safety Executive, United Kingdom
Rochedo, E.	Institute of Radiation Protection and Dosimetry, Brazil
Rotaru, I.	National Commission for Nuclear Activities Control, Romania
Rudjord, A.	Norwegian Radiation Protection Authority, Norway
Runova, J.	Scientific and Engineering Centre for Nuclear and Radiation Safety, Russian Federation
Ryder, G.	International Trade Union Confederation, Belgium
Saint-Pierre, S.	World Nuclear Association, United Kingdom
Sallit, G.	Department for Transport, United Kingdom

Salomon, S.	Australian Radiation Protection and Nuclear Safety Agency, Australia
Sanz Alduan, M.	Nuclear Safety Council, Spain
Schmitt-Hannig, A.	Federal Office for Radiation Protection, Germany
Sefzig, R.	Federal Ministry for the Environment, Nature Conservation and Nuclear Safety, Germany
Shannoun, F.	World Health Organization
Simeonov, G.	European Commission
Sinaga, M.	Nuclear Energy Regulatory Agency, Indonesia
Stasiunaitiene, R.	Ministry of Health, Lithuania
Stephen, P.	Nuclear Directorate, United Kingdom
Stern, W.	International Atomic Energy Agency
Storrie, R.	International Source Suppliers and Producers Association
Sugier, A.	Institute for Radiological Protection and Nuclear Safety, France
Suman, H.	International Atomic Energy Agency
Sutej, T.	Ministry of Health, Romania
Svensson, H.	International Organization for Medical Physics
Syahrir, S.	National Nuclear Energy Agency, Indonesia
Telleria, D.	International Atomic Energy Agency
Thomas, G.	Health and Safety Executive, United Kingdom
Tirmarche, M.	Institute for Radiological Protection and Nuclear Safety, France
Todorov, N.	Bulgarian Nuclear Regulatory Agency, Bulgaria
Tokonami, S.	National Institute of Radiological Sciences, Japan

Tomasek, L.	SÚRO National Radiation Protection Institute, Czech Republic
Tonhauser, W.	International Atomic Energy Agency
Ugleveit, F.	Norwegian Radiation Protection Authority, Norway
Valentin, J.	International Commission on Radiological Protection
Van der Steen, J.	Nuclear Research & Consultancy Group, Netherlands
Viktorsson, C.	International Atomic Energy Agency
Wambersie, A.	International Commission on Radiation Units, Belgium
Wangler, M.	International Atomic Energy Agency
Weiss, W.	Federal Office for Radiation Protection, Germany
Wheatley, J.	International Atomic Energy Agency
Wiklund, A.	European Commission
Wirth, E.	Federal Office for Radiation Protection, Germany
Wood, P.	International Society of Radiographers and Radiological Technologists
Wrixon, A.	International Atomic Energy Agency
Wymer, D.	International Atomic Energy Agency
Xiao, X.	China Institute of Atomic Energy, China
Yonehara, H.	National Institute of Radiological Sciences, Japan
Zafmanjato, J.	Ministère de l'éducation national et de la recherche scientifique, Madagascar
Zeeb, H.	World Health Organization
Zodlates, T.	International Labour Organization, International Trade Union Confederation
Zuur, C.	Ministry of Housing, Spatial Planning and Environment, Netherlands



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Suweco CZ, spol. S.r.o.

Klecakova 347, 180 21 Prague 9, CZECH REPUBLIC

Telephone: +420 242 459 202 • Fax: +420 242 459 203

Email: nakup@suweco.cz • Web site: <http://www.suweco.cz>

FINLAND

Akateeminen Kirjakauppa

PO Box 128 (Keskuskatu 1), 00101 Helsinki, FINLAND

Telephone: +358 9 121 41 • Fax: +358 9 121 4450

Email: akatilaus@akateeminen.com • Web site: <http://www.akateeminen.com>

FRANCE

Form-Edit

5 rue Janssen, PO Box 25, 75921 Paris CEDEX, FRANCE

Telephone: +33 1 42 01 49 49 • Fax: +33 1 42 01 90 90

Email: fabien.boucard@formedit.fr • Web site: <http://www.formedit.fr>

Lavoisier SAS

14 rue de Provigny, 94236 Cachan CEDEX, FRANCE

Telephone: +33 1 47 40 67 00 • Fax: +33 1 47 40 67 02

Email: livres@lavoisier.fr • Web site: <http://www.lavoisier.fr>

L'Appel du livre

99 rue de Charonne, 75011 Paris, FRANCE

Telephone: +33 1 43 07 50 80 • Fax: +33 1 43 07 50 80

Email: livres@appeldulivre.fr • Web site: <http://www.appeldulivre.fr>

GERMANY

Goethe Buchhandlung Teubig GmbH

Schweitzer Fachinformationen

Willstätterstrasse 15, 40549 Düsseldorf, GERMANY

Telephone: +49 (0) 211 49 8740 • Fax: +49 (0) 211 49 87428

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